

September 21, 2000

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: DRESDEN - ISSUANCE OF AMENDMENTS CHANGING THE UNIT 2 MINIMUM
CRITICAL POWER RATIO AND ADDING THE RODEX2A METHODOLOGY
FOR UNITS 2 AND 3 (TAC NOS. MA6233 AND MA6234)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 180 to Facility Operating License No. DPR-19 and Amendment No. 175 to Facility Operating License No. DPR-25 for Dresden, Units 2 and 3. The amendments are in response to your application dated August 3, 1999, as supplemented on February 25, 2000.

The amendments revise Technical Specification (TS) Section 2.1.B to reflect a change to the Minimum Critical Power Ratio for Unit 2; add an approved analytical method to TS Section 6.9.A.6 for Units 2 and 3 for use in determining core operating limits; and add conditions to the Unit 2 and 3 licenses to limit the maximum rod average burnup for any rod to 60 GWD/MTU until the staff has completed an environmental assessment supporting a greater limit. Also, a typographic error on the last page of the Unit 3 license is corrected to give the correct Unit 3 license expiration date of January 12, 2011.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Lawrence W. Rossbach, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-237 and 50-249

- Enclosures: 1. Amendment No. 180 to DPR-19
- 2. Amendment No. 175 to DPR-25
- 3. Safety Evaluation

cc w/encls: See next page

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DATE	09/20/00	09/20/00	05/25/00	05/18/00	9/5/00	09/20/00	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 21, 2000

Mr. Oliver D. Kingsley, President /
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: DRESDEN - ISSUANCE OF AMENDMENTS CHANGING THE UNIT 2 MINIMUM CRITICAL POWER RATIO AND ADDING THE RODEX2A METHODOLOGY FOR UNITS 2 AND 3 (TAC NOS. MA6233 AND MA6234)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 180 to Facility Operating License No. DPR-19 and Amendment No.175 to Facility Operating License No. DPR-25 for Dresden, Units 2 and 3. The amendments are in response to your application dated August 3, 1999, as supplemented on February 25, 2000.

The amendments revise Technical Specification (TS) Section 2.1.B to reflect a change to the Minimum Critical Power Ratio for Unit 2; add an approved analytical method to TS Section 6.9.A.6 for Units 2 and 3 for use in determining core operating limits; and add conditions to the Unit 2 and 3 licenses to limit the maximum rod average burnup for any rod to 60 GWD/MTU until the staff has completed an environmental assessment supporting a greater limit. Also, a typographic error on the last page of the Unit 3 license is corrected to give the correct Unit 3 license expiration date of January 12, 2011.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Lawrence W. Rossbach, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
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Docket Nos. 50-237 and 50-249

Enclosures: 1. Amendment No. 180 to DPR-19
2. Amendment No. 175 to DPR-25
3. Safety Evaluation

cc w/encls: See next page

O. Kingsley
Commonwealth Edison Company

Dresden Nuclear Power Station
Units 2 and 3

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 180
License No. DPR-19

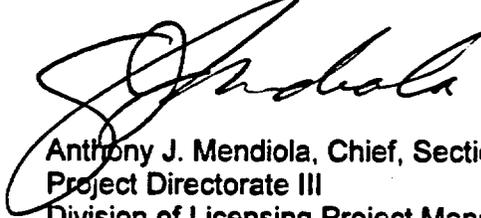
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated August 3, 1999, as supplemented on February 25, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Operating License and the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 180, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachments:
Changes to the Technical Specifications
and Operating License

Date of Issuance: September 21, 2000



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175
License No. DPR-25

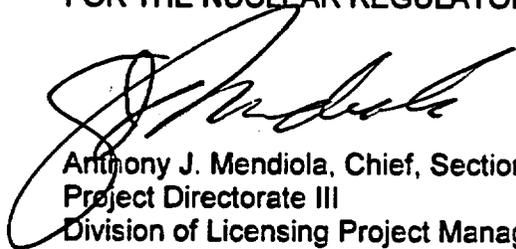
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated August 3, 1999, as supplemented on February 25, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Operating License and the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 175 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachments:
Changes to the Technical Specifications
and Operating License

Date of Issuance: September 21, 2000

ATTACHMENT TO LICENSE AMENDMENT NOS. 180 AND 175

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Replace the following page of Operating Licenses Nos. DPR-19 and DPR-25 with the attached revised page. The revised page is identified by amendment number and contains a vertical line in the margin indicating the area of change.

REMOVE

Unit 2 - page 3a

--

Unit 3 - page 6

--

INSERT

Unit 2 - page 3a

Unit 2 - page 3b

Unit 3 - page 6

Unit 3 - page 7

Revise the Appendix "A" Technical Specifications by replacing the pages identified below with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line in the margin indicating the area of change.

REMOVE

2-1

6-16

INSERT

2-1

6-16

- a. Surveillance Requirement 4.1.A.2 - RPS Logic System Functional Test
- b. Surveillance Requirement 4.2.A.2 - Primary & Secondary Containment Logic System Functional Test
- c. Surveillance Requirement 4.2.J.2 - Feedwater Pump Trip Logic System Functional Test
- d. Surveillance Requirement 4.6.F.1.b - Relief Valve Logic System Functional Test
- e. Surveillance Requirement 4.9.A.9 - Simultaneous Diesel Generator Start
- f. Surveillance Requirement 4.9.A.10 - Diesel Storage Tank Cleaning (Unit 3 and Unit 2/3 only)

Each of the above Surveillance Requirements shall be successfully demonstrated prior to entering into MODE 2 on the first plant startup following the fifteenth refueling outage (D2R15).

(7) Additional Conditions

The Additional Conditions contained in Appendix B, as revised through Amendment No. 163, are hereby incorporated into this license. Commonwealth Edison Company shall operate the facility in accordance with the Additional Conditions.

(8) Pressure-Temperature Limit Curves

The pressure-temperature (P-T) limit curves issued by Amendment No. 179 are approved for use until November 30, 2001, unless Commonwealth Edison Company, the licensee, obtains approval from the Nuclear Regulatory Commission staff for use beyond November 30, 2001.

(9) Fuel Burnup

The maximum rod average burnup for any rod shall be limited to 60 GWD/MTU until the completion of an NRC environmental assessment supporting an increased limit.

- D. The facility has been granted certain exemptions from the requirements of Section III.G of Appendix R to 10 CFR Part 50, "Fire Protection Program for Nuclear Power Facilities Operating Prior to January 1, 1979." This section relates to fire protection features for ensuring the systems and associated circuits used to achieve and maintain safe shutdown are free of fire damage. These exemptions were granted and sent to the licensee in letters dated February 2, 1983, September 28, 1987, July 6, 1989, and August 15, 1989.

In addition, the facility has been granted certain exemptions from Sections II and III of Appendix J to 10 CFR Part 50, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors." This section contains leakage test requirements, schedules and acceptance criteria for tests of the leak-tight integrity of the primary reactor containment and systems and components which penetrate the containment. These exemptions were granted and sent to the licensee in a letter dated June 25, 1982.

- L. Deleted. [Amdt. 87, 7-24-86]
- M. Deleted. [Amdt. 85, 12-12-85]
- N. By Amendment No. 144, the license is amended to allow, on a one time temporary basis, operation of Dresden, Unit 3, with the corner room structural steel members in the Low Pressure Coolant Injection Corner Rooms outside the Updated Final Safety Analysis Report (UFSAR) design parameters. Operation under these conditions is allowed up to and including the next scheduled refueling outage (D3R14).

The repairs to Dresden, Unit 3, corner room structural steel shall restore the steel design margins to the current UFSAR (updated through Revision 1A) design criteria. The design of the modifications to the Dresden, Unit 3, corner room structural steel members will be based on use of elastic section modules and the structural steel stresses will be limited to 1.6 of the American Institute of Steel Construction (AISC allowables). The modifications to Dresden, Unit 3, corner room structural steel will be implemented during the upcoming D3R14 refueling outage.

During this interim period of operation, should vibratory ground motion exceeding the UFSAR Operating Basis Earthquake (OBE) design parameters, Dresden, Unit 3, will be shut down for inspection and will not start up without prior NRC approval.

O. Additional Conditions

The Additional Conditions contained in Appendix B, as revised through Amendment No. 158, are hereby incorporated into this license. Commonwealth Edison Company shall operate the facility in accordance with the Additional Conditions.

P. Pressure-Temperature Limit Curves

The pressure-temperature (P-T) limit curves issued by Amendment No. 174 are approved for use until October 30, 2002, unless Commonwealth Edison Company, the licensee, obtains approval from the Nuclear Regulatory Commission staff for use beyond October 30, 2002.

Q. Fuel Burnup

The maximum rod average burnup for any rod shall be limited to 60 GWD/MTU until the completion of an NRC environmental assessment supporting an increased limit.

4. This license is effective as of the date of issuance and shall expire at Mid-night January 12, 2011.

FOR THE ATOMIC ENERGY COMMISSION

Original signed by:

Peter A. Morris, Director
Division of Licensing

Enclosures:

Appendix A - Technical Specifications
Appendix B - Additional Conditions

Date of Issuance: January 12, 1971

2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMITS**THERMAL POWER, Low Pressure or Low Flow**

2.1.A THERMAL POWER shall not exceed 25% of RATED THERMAL POWER with the reactor vessel steam dome pressure less than 785 psig or core flow less than 10% of rated flow.

APPLICABILITY: OPERATIONAL MODE(s) 1 and 2.

ACTION:

With THERMAL POWER exceeding 25% of RATED THERMAL POWER and the reactor vessel steam dome pressure less than 785 psig or core flow less than 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours and comply with the requirements of Specification 6.7.

THERMAL POWER, High Pressure and High Flow

2.1.B The MINIMUM CRITICAL POWER RATIO (MCPR) shall not be less than the following:

Unit 2: 1.09 for cycle exposures less than or equal to 13,800 MWd/MTU and 1.12 for cycle exposures greater than 13,800 MWd/MTU, and

Unit 3: 1.10

with the reactor vessel steam dome pressure greater than or equal to 785 psig and core flow greater than or equal to 10% of rated flow. During single recirculation loop operation, the MCPR limit shall be increased by 0.01.

APPLICABILITY: OPERATIONAL MODE(s) 1 and 2

ACTION:

With MCPR less than the above applicable limit and the reactor vessel steam dome pressure greater than or equal to 785 psig and core flow greater than or equal to 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours and comply with the requirements of Specification 6.7.

ADMINISTRATIVE CONTROLS

- (12) ANF-1125 (P)(A), ANFB Critical Power Correlation Determination of ATRIUM-93 Additive Constant Uncertainties, Supplement 1, Appendix E, Siemens Power Corporation, September 1998.
 - (13) EMF-85-74(P), RODEX2A (BWR) Fuel Rod Thermal Mechanical Evaluation Model, Supplement 1 (P)(A) and Supplement 2 (P)(A), Siemens Power Corporation, February 1998.
- c. The core operating limits report shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met. The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions of supplements thereto shall be provided on issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

6.9.B Special Reports

Special reports shall be submitted to the Regional Administrator of the NRC Regional Office within the time period specified for each report.

6.10 [INTENTIONALLY LEFT BLANK]



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 180 TO FACILITY OPERATING LICENSE NO. DPR-19
AND AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-25
COMMONWEALTH EDISON COMPANY
DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3
DOCKET NOS. 50-237 AND 50-249

1.0 INTRODUCTION

By letter dated August 3, 1999 (Reference 1), as clarified by letter dated February 25, 2000 (Reference 2), Commonwealth Edison Company (ComEd, the licensee) proposed changes to the Technical Specifications (TS) for the Dresden Nuclear Power Station, Units 2 and 3. The requested changes include revising the safety limit minimum critical power ratio (SLMCPR) in TS 2.1.B for Dresden, Unit 2, Cycle 17 (D2C17) operation and adding an approved licensing topical report, EMF-85-74(P), RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model, Supplement 1(P)(A) and Supplement 2(P)(A), Siemens Power Corporation (SPC), February 1998 to Section 6 for both Units 2 and 3. The Unit 2 Cycle 17 is a mixed core of 724 Siemens Power Corporation fuel assemblies, of which there are 248 fresh SPC ATRIUM-9B bundles, 168 once-burned SPC ATRIUM-9B bundles, 224 SPC 9x9-2 and 8 SPC ATRIUM-9B twice-burned bundles, and 76 thrice-burned SPC 9x9-2 bundles. The February 25, 2000, letter provided clarifying information and did not change the staff's initial proposed no significant hazards consideration determination. Also, a typographic error on Page 7 of the Unit 3 license is corrected to give the correct Unit 3 license expiration date of January 12, 2011.

2.0 EVALUATION

The licensee requested a change to the Dresden Nuclear Power Station, Units 2 and 3, Facility Operating Licenses in accordance with 10 CFR 50.90. The revised TSs were proposed as follows:

2.1 Specification 2.1.B - THERMAL POWER, High Pressure and High Flow

The proposed amendment involves revising the value of the safety limit minimum critical power ratio (MCPR) and its applicability in terms of the cycle exposure for D2C17 operation such that the SLMCPR shall not be less than 1.09 for cycle exposures less than or equal to 13,800 MWD/MTU and 1.12 for cycle exposures greater than 13,800 MWD/MTU with the reactor vessel steam dome pressure greater than 785 psig and core flow greater than or equal to 10 percent of rated flow. The current SLMCPR is 1.09.

The licensee described the methodology to calculate the new SLMCPR values for the TSs in its submittals. The Cycle 17 SLMCPR analysis was performed by SPC using the plant- and cycle-specific fuel and core parameters, NRC-approved methodologies including: (1) ANF-1125(P)(A), "Critical Power Correlation-ANFB"; (2) ANF-524(P)(A), "Advanced Nuclear Fuels Corporation Critical Power Methodology for Boiling Water Reactors/Advanced Nuclear Fuels Corporation Critical Power Methodology for Boiling Water Reactors: Methodology for Analysis of Assembly Channel Bowing Effects" Revision 2, Supplement 1, Revision 2, Supplement 2; and (3) ANF-1125(P)(A), "ANFB Critical Power Correlation Determination of ATRIUM 9B Additive Constant Uncertainties" Supplement 1, Appendix E.

The licensee explained that the higher SLMCPR value for D2C17 exposures greater than 13,999 MWD/MTU is due to increased bundle exposures in the core design, which result in a significant increase in the impact of channel bow on the SLMCPR calculation. Based on our review of the submittal and the response to the request for additional information, the staff has concluded that the SLMCPR analysis for D2C17 operation using the plant- and cycle-specific calculation in conjunction with the approved method is acceptable. The D2C17 SLMCPR will ensure that 99.9 percent of the fuel rods in the core will not experience boiling transition which satisfies the requirements of Generic Design Criterion 10 of Appendix A to 10 CFR Part 50 regarding acceptable fuel design limits. Therefore, the staff has concluded that the justification for analyzing and determining the SLMCPR values for D2C17 is acceptable since approved methodologies were used.

2.2 Specification 6.9.A.6.b - REPORTING REQUIREMENTS CORE OPERATING LIMITS REPORT

The licensee proposed to add an approved topical report, "EMF-85-74(P), Revision 0, RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model, Supplements 1 and 2 (P)(A), Siemens Power Corporation, February 1998," to TS 6.9.A.6.b for Unit 2 and 3.

The SPC has performed additional analyses using RODEX2A corresponding to the maximum acceptable ATRIUM 9B peak assembly exposure of 48 GWD/MTU and determined new peak fuel rod exposure of 55 GWD/MTU and peak fuel pellet exposure of 66 GWD/MTU for the ATRIUM 9B fuel. The licensee has stated in the response (Reference 2) for the Unit 3, Cycle 17, core design projects that the fuel assemblies will exceed 60 GWD/MTU peak pellet exposure, but the Unit 2 Cycle 17 design is not projected to exceed these exposure limits.

The burnup limits for ATRIUM-9B fuel using the ANF-89-14(P)(A) methodology currently in the Dresden, Units 2 and 3, TS are 60 GWD/MTU peak pellet, 50 GWD/MTU rod average and 48 GWD/MTU assembly average burnup. The staff has found that the RODEX2A methodology is acceptable for burnup limits of 62 GWD/MTU rod average and 54 GWD/MTU assembly-average exposures (Reference 3). However, the staff has not completed an environmental assessment for these higher burnup limits. The staff has previously performed related environmental assessments for fuel burnup to 60 GWD/MTU (Reference 4). Due to this limitation from the environmental perspective, the staff proposed a license condition to ComEd which reads: "The maximum rod average burnup for any rod shall be limited to 60 GWD/MTU until the completion of an NRC environmental assessment supporting an increased limit." On

May 24, 2000, ComEd agreed to this license condition. The staff used a previous license amendment as the precedence for this license condition (Reference 5). The staff finds the proposed license condition acceptable.

The staff has reviewed the proposed TS change in Core Operating Limits Report (COLR) requirements and found it acceptable because an approved topical report was used (Reference 3) that is appropriate for use at Dresden.

Based on our review, the staff concludes that the above described TS changes involving the SLMCPR value in TS 2.1.B for Dresden, Unit 2, and the addition of the approved topical report RODEX2A for both Dresden, Units 2 and 3, are acceptable because the changes were analyzed based on the NRC-approved methods and will ensure that all applicable core operating limits are met.

2.3 Unit 3 License Expiration Date Corrected

Amendment No. 106 to Dresden, Unit 3, issued April 24, 1990, established the Unit 3 license expiration date as January 12, 2011. The Unit 3 expiration date was shown correctly on page 6 to the Unit 3 license issued by Amendment No. 144, dated May 31, 1996. Unit 3 Amendment No. 147, dated January 28, 1997, introduced a typographic error on the last page of the Unit 3 license, page 6, by mistakenly indicating that the Unit 3 license expires on January 12, 2001. The last page of the Unit 3 license, now page 7, enclosed with this amendment, corrects the Unit 3 license expiration date to January 12, 2011.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact has been prepared and published in the Federal Register on September 19, 2000 (65 FR 56604) for the portion of the amendment adding the RODEX2A methodology.

Accordingly, based upon the environmental assessment, the staff has determined that the issuance of this amendment adding the RODEX2A methodology to the TSs will not have a significant effect on the quality of the human environment.

With respect to the changes in SLMCPR, the amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (64 FR 48859). Accordingly, the amendments meet the eligibility

criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the implementation of these changes.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

6.0 REFERENCES

1. Letter (JMHLTR: #99-0078) from J. M. Heffley to USNRC, "Request for Amendment to Appendix A, Minimum Critical Power Ratio," August 3, 1999.
2. Letter (PSLTR: #00-0060) from Preston Swafford to USNRC, "Request for Additional Information Technical Specifications Change," February 25, 2000.
3. Letter from T. H. Essig, USNRC, to H. D. Curet, Siemens Power Corporation, February 9, 1998.
4. 53 FR 6041, February 29, 1988, "Extended Burnup Fuel Use in Commercial LWR's; Environmental Assessment and Finding of No Significant Impact," and 53 FR 30355, August 11, 1988, "NRC Assessment of the Environmental Effects of Transportation Resulting from Extended Fuel Enrichment and Irradiation," as corrected by 53 FR 32322, August 24, 1988.
5. License Amendments 188 and 169 for the McGuire Nuclear Station, September 22, 1999.

Principal Contributors: T. Huang
L. Rossbach

Date: September 21, 2000