

Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616-465-5001



September 8, 2000

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating License DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

LER 316/1998-007-01, "High Energy Line Break Effects On Auxiliary Feedwater System."

The condition described in this LER is one of a number of issues related to inadequacies associated with the high energy line break (HELB) program that were combined into a single report, Donald C. Cook Nuclear Plant LER 315/1999-026-00, "High Energy Line Break Programmatic Inadequacies Result in Unanalyzed Conditions," submitted in November 1999. The combined report better characterizes the aggregate impact of the HELB programmatic inadequacies that led to these individual conditions, and provides an evaluation of each condition. As such, LER 316/1998-007-01 is being submitted to close out this individual HELB issue to LER 315/1999-026-00.

There are no commitments identified in this submittal.

Should you have any questions regarding this correspondence, please contact Mr. Wayne J. Kropp, Director Regulatory Affairs, at 616/697-5056.

Sincerely,

A handwritten signature in black ink, appearing to read 'A. Christopher Bakken, III'. The signature is stylized and cursive.

A. Christopher Bakken, III
Site Vice President

/srd
Attachment

c: J. E. Dyer, Region III
D. Hahn
B. A. McIntyre
T. P. Noonan
R. P. Powers
R. Whale
NRC Resident Inspector
Records Center, INPO

NRC Form 366 (6-1998)	U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)	APPROVED BY OMB NO. 3150-0104 EXPIRES 06/30/2001 <small>ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-8 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503</small>
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FACILITY NAME (1) Donald C. Cook Nuclear Plant Unit 2	DOCKET NUMBER (2) 05000-316	PAGE (3) 1 of 1
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TITLE (4)

High Energy Line Break Effects On Auxiliary Feedwater System

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
11	06	1998	1998	-- 007 --	01	09	08	2000	FACILITY NAME	DOCKET NUMBER	
OPERATING MODE (9)		5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
POWER LEVEL (10)		--	20.2201 (b)			20.2203(a)(2)(v)			50.73(a)(2)(i)	50.73(a)(2)(viii)	
			20.2203(a)(1)			20.2203(a)(3)(i)			<input checked="" type="checkbox"/> 50.73(a)(2)(ii)	50.73(a)(2)(x)	
			20.2203(a)(2)(i)			20.2203(a)(3)(ii)			50.73(a)(2)(iii)	73.71	
			20.2203(a)(2)(ii)			20.2203(a)(4)			50.73(a)(2)(iv)	OTHER	
			20.2203(a)(2)(iii)			50.36(c)(1)			50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A	
			20.2203(a)(2)(iv)			50.36(c)(2)			50.73(a)(2)(vii)		

LICENSEE CONTACT FOR THIS LER (12)

NAME Mr. Ron Gaston, Regulatory Affairs	TELEPHONE NUMBER (Include Area Code) 616 / 465-5901, x1366
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If Yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/>	NO						

Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)
This supplement is submitted to close out this individual high energy line break (HELB) issue to LER 315/1999-026-00.

In July 1995, the fire door/High Energy Line Break (HELB) door for the startup blowdown flash tank room was blocked open for approximately 39 hours to allow draining of the Essential Service Water (ESW) header. At the time, the condition was evaluated and it was concluded that the condition was acceptable for up to 72 hours. This conclusion was based upon the assumption that the HELB would be detected and isolated within 10 minutes and that the line break would not result in automatic actions. As part of the recent Auxiliary Feedwater (AFW) System Safety System Functional Inspection, the assumption that the break would be isolated within 10 minutes was revisited and determined not to be valid. A subsequent review of Westinghouse WCAP 10961 revealed that automatic action, a reactor trip, would occur at 12 minutes into the event. Based on the new information, on November 6, 1998 it was determined that a HELB in the startup blowdown flash tank room would potentially expose the motor control centers (MCC) in the area to a steam environment and higher temperatures than they are qualified to withstand. Auxiliary Feedwater would potentially be lost because the valves which supply ESW to the AFW pumps are located on these MCCs. It was determined that this condition represented a condition outside the design bases, reportable under 10 CFR 50.73 (a)(2)(ii)(B).

This condition is one of a number of issues related to inadequacies associated with the HELB program that were combined into a single report, LER 315/1999-026-00, "High Energy Line Break Programmatic Inadequacies Result in Unanalyzed Conditions," submitted in November 1999. The combined report better characterizes the aggregate impact of the HELB programmatic inadequacies that led to these individual conditions, and provides an evaluation of each condition.