

Indiana Michigan  
Power Company  
Cook Nuclear Plant  
One Cook Place  
Bridgman, MI 49106  
616-466-5901



September 8, 2000

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Operating License DPR-74  
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

LER 316/1998-005-01, "Potential for High Energy Line Break to Degrade Component Cooling Water System."

The condition described in this LER is one of a number of issues related to inadequacies associated with the high energy line break (HELB) program that were combined into a single report, Donald C. Cook Nuclear Plant LER 315/1999-026-00, "High Energy Line Break Programmatic Inadequacies Result in Unanalyzed Conditions," submitted in November 1999. The combined report better characterizes the aggregate impact of the HELB programmatic inadequacies that led to these individual conditions, and provides an evaluation of each condition. As such, LER 316/1998-005-01 is being submitted to close out this individual HELB issue to LER 315/1999-026-00.

There are no commitments identified in this submittal.

Should you have any questions regarding this correspondence, please contact Mr. Wayne J. Kropp, Director Regulatory Affairs, at 616/697-5056.

Sincerely,

A handwritten signature in black ink, appearing to read 'A. Bakken, III'. The signature is fluid and cursive.

A. Christopher Bakken, III  
Site Vice President

/srd  
Attachment

JE22

c: J. E. Dyer, Region III  
D. Hahn  
B. A. McIntyre  
T. P. Noonan  
R. P. Powers  
R. Whale  
NRC Resident Inspector  
Records Center, INPO

**LICENSEE EVENT REPORT (LER)**

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1) Donald C. Cook Nuclear Plant Unit 2	DOCKET NUMBER (2) 05000-316	PAGE (3) 1 of 1
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TITLE (4)  
Potential for High Energy Line Break to Degrade Component Cooling Water System

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
07	15	1998	1998	-- 005 --	01	09	08	2000	Cook Plant Unit 1	05000-315	
									FACILITY NAME	DOCKET NUMBER	

OPERATING MODE (9)	5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
POWER LEVEL (10)	--	20.2201 (b)		20.2203(a)(2)(v)					50.73(a)(2)(i)	50.73(a)(2)(viii)
		20.2203(a)(1)		20.2203(a)(3)(i)		X		50.73(a)(2)(ii)		50.73(a)(2)(x)
		20.2203(a)(2)(i)		20.2203(a)(3)(ii)				50.73(a)(2)(iii)		73.71
		20.2203(a)(2)(ii)		20.2203(a)(4)				50.73(a)(2)(iv)		OTHER
		20.2203(a)(2)(iii)		50.36(c)(1)				50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A
	20.2203(a)(2)(iv)		50.36(c)(2)				50.73(a)(2)(vii)			

LICENSEE CONTACT FOR THIS LER (12)	
NAME Mr. Ron Gaston, Regulatory Affairs	TELEPHONE NUMBER (Include Area Code) 616 / 465-5901, x1366

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		
YES (If Yes, complete EXPECTED SUBMISSION DATE)	X	NO		MONTH	DAY	YEAR

**Abstract** (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)  
 This supplement is submitted to close out this individual high energy line break (HELB) issue to LER 315/1999-026-00.

On July 15, 1998, with Unit 1 and Unit 2 in Cold Shutdown, it was determined that the potential existed for a postulated critical crack in a Unit 2 main steam line to degrade the ability of adjacent Component Cooling Water (CCW) pumps to perform their design function. The CCW pumps for both units are adjacent to one another in a semi-enclosed area in the Auxiliary Building. Adjacent to the pumps is a pipe chase enclosing two Unit 2 main steam lines and a main feedwater line, which can be accessed through any of 3 doors. Although the pipe chase walls provide a qualified HELB barrier, no calculation could be found which shows that the doors would withstand the energy released from a postulated critical crack. As the adjacent CCW pump motors and other equipment are not qualified for a high temperature/high humidity environment, this was determined to constitute an unanalyzed condition. An ENS notification was made on July 15 at 1521 hours EDT in accordance with 10CFR50.72(b)(2)(i). This LER is therefore submitted under 10CFR50.73(a)(2)(ii)(B) for an unanalyzed condition

This condition is one of a number of issues related to inadequacies associated with the HELB program that were combined into a single report, LER 315/1999-026-00, "High Energy Line Break Programmatic Inadequacies Result in Unanalyzed Conditions," submitted in November 1999. The combined report better characterizes the aggregate impact of the HELB programmatic inadequacies that led to these individual conditions, and provides an evaluation of each condition.