

September 14, 2000

Mr. Valeri Tolstykh  
Regulatory Activities Unit  
Safety Assessment Section  
Division of Nuclear Installation Safety  
International Atomic Energy Agency  
Wagramer Strasse 5  
P.O. Box 100, A-1400  
Vienna, Austria

Dear Mr. Tolstykh:

Enclosed are the following IRS reports:

- ANNUAL REPORT ON THE EFFECTIVENESS OF TRAINING IN THE NUCLEAR INDUSTRY FOR CALENDAR YEAR 1999 (NRC Regulatory Issue Summary 2000-13).
- RECOMMENDATIONS FOR ENSURING CONTINUED SAFE PLANT OPERATION AND MINIMIZING REQUESTS FOR ENFORCEMENT DISCRETION DURING EXTREME WEATHER CONDITIONS (NRC Regulatory Issue Summary 2000-15).
- AVAILABILITY OF THE REACTOR VESSEL INTEGRITY DATABASE VERSION 2.0.1 (NRC Regulatory Issue Summary 2000-16).

Each report is being submitted in the following two media: (1) a hard copy of the input file for the AIRS database; and (2) a 3.5-inch HD diskette containing the input file for the AIRS database in Microsoft Word 6.0 format.

If you have any questions regarding these reports, please call Eric J. Benner of my staff. He can be reached at (301) 415-1171.

Sincerely,

**/RA**  
Ledyard B. Marsh, Chief  
Events Assessment, Generic Communications  
and Non-Power Reactors Branch  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Enclosures: As stated

cc w/enclosures 1 and 2:  
Mr. Lennart Carlsson  
Nuclear Safety Division  
Nuclear Energy Agency  
Organization for Economic  
Cooperation and Development  
Le Seine Saint Germain  
12, Boulevard des Iles  
92130, Issy-les-Moulineaux, France

September 14, 2000

Mr. Valeri Tolstykh  
Regulatory Activities Unit  
Safety Assessment Section  
Division of Nuclear Installation Safety  
International Atomic Energy Agency  
Wagramer Strasse 5  
P.O. Box 100, A-1400  
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Ledyard B. Marsh, Chief  
Events Assessment, Generic Communications  
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Office of Nuclear Reactor Regulation

Enclosures: As stated

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Mr. Lennart Carlsson  
Nuclear Safety Division  
Nuclear Energy Agency  
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Le Seine Saint Germain  
12, Boulevard des Iles  
92130, Issy-les-Moulineaux, France

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DATE	09/14/2000		09/14/2000		09/14/2000	

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## INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b> 2000/08/18	<b>DATE RECEIVED</b>
<b>EVENT TITLE</b> ANNUAL REPORT ON THE EFFECTIVENESS OF TRAINING IN THE NUCLEAR INDUSTRY FOR CALENDAR YEAR 1999 (NRC Regulatory Issue Summary 2000-13)		
<b>COUNTRY</b> USA	<b>PLANT AND UNIT</b> Generic	<b>REACTOR TYPE</b> (BWR or PWR)
<b>INITIAL STATUS</b> N/A	<b>RATED POWER (MWe NET)</b> N/A	
<b>DESIGNER</b> (WEST, GE, CE, B&W)	<b>1st COMMERCIAL OPERATION</b> N/A	

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### ABSTRACT

This IRS report discusses the availability of the NRC's "Annual Report on the Effectiveness of Training in the Nuclear Industry for Calendar Year 1999" and to provide some insights from the information contained in the report. An analysis of the training-related information contained in inspection reports and LERs indicated that training-related issues identified in 1999 were concentrated in two distinct areas: "Training less than adequate (LTA)" and "Individual knowledge less than adequate (LTA)". While training-related issues have been concentrated in these same two areas since 1997, the 1999 information shows a decrease in the number of issues attributed to Training LTA and a contrasting increase in the number of issues attributed to Individual knowledge LTA. The 1999 information suggests that the causes of poor performance appear to be becoming more often attributed to the individual rather than to a group or a class of workers. The increasing trend in the number of issues attributed to Individual knowledge LTA does not appear to be an issue in and of itself but rather appears to signal a change in emphasis within licensee problem identification and corrective action programs.

ANNUAL REPORT ON THE EFFECTIVENESS OF TRAINING IN THE NUCLEAR INDUSTRY  
FOR CALENDAR YEAR 1999 (NRC Regulatory Issue Summary 2000-13)

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1.	Reporting Categories:	<u>1.4</u>	_____	_____
2.	Plant Status Prior to the Event:	<u>2.0</u>	_____	_____
3.	Failed/Affected Systems:	<u>3.Z</u>	_____	_____
4.	Failed/Affected Components:	<u>4.0</u>	_____	_____
5.	Cause of the Event:	<u>5.5.6</u>	_____	_____
			_____	_____
6.	Effects on Operation:	<u>6.0</u>	_____	_____
7.	Characteristics of the Incident:	<u>7.0</u>	_____	_____
8.	Nature of Failure or Error:	<u>8.0</u>	_____	_____
9.	Nature of Recovery Actions:	<u>9.0</u>	_____	_____

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF NUCLEAR REACTOR REGULATION  
WASHINGTON, D.C. 20555-0001

August 18, 2000

**NRC REGULATORY ISSUE SUMMARY 2000-13  
ANNUAL REPORT ON THE EFFECTIVENESS  
OF TRAINING IN THE NUCLEAR INDUSTRY  
FOR CALENDAR YEAR 1999**

ADDRESSEES

All holders of operating licenses for nuclear power reactors.

INTENT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this regulatory issue summary (RIS) to inform the addressees of the availability of the NRC's "Annual Report on the Effectiveness of Training in the Nuclear Industry for Calendar Year 1999" and to provide some insights from the information contained in the report. This RIS does not create any new or changed NRC requirements or staff positions, and it requires no specific action or written response.

BACKGROUND INFORMATION

Public health and safety depend on the proper operation, testing, and maintenance of nuclear power plant systems and components by plant personnel. Successful performance by nuclear power plant personnel is reasonably assured by having workers achieve and maintain job task qualifications through the systems approach to training (SAT-based training) and the continuous retraining required by 10 CFR Part 55 and 10 CFR 50.120. Favorable job performance and successful SAT implementation provide a reasonable assurance that the training of nuclear power plant workers is sufficient to maintain public health and safety.

The NRC monitored training-related activities in 1999 as indicators of worker performance and SAT implementation. The NRC monitored worker performance by reviewing licensee event reports (LERs) and inspection reports for training-related issues; inspecting training programs when an appropriate reason was identified; and administering, inspecting, and reviewing the results of operator licensing activities. The NRC also indirectly monitored SAT use by reviewing the effectiveness of the accreditation process administered by the Institute of Nuclear Power Operations. These activities provide an efficient and effective assessment of industry activities with a minimal impact on licensees. Although each activity can provide plant-specific information, the information is used in the composite within the annual report to assess the overall effectiveness of training in the nuclear industry.

ML003719812

The full text of the "Annual Report on the Effectiveness of Training in the Nuclear Industry for Calendar Year 1999" can be located in the Agencywide Documents Access and Management System (ADAMS) at ML003705668.

### SUMMARY OF ISSUE

An analysis of the training-related information contained in inspection reports and LERs indicated that training-related issues identified in 1999 were concentrated in two distinct areas: "Training less than adequate (LTA)"<sup>1</sup> and "Individual knowledge less than adequate (LTA)"<sup>2</sup>. While training-related issues have been concentrated in these same two areas since 1997, the 1999 information shows a decrease in the number of issues attributed to Training LTA and a contrasting increase in the number of issues attributed to Individual knowledge LTA. The 1999 information suggests that the causes of poor performance appear to be becoming more often attributed to the individual rather than to a group or a class of workers. The increasing trend in the number of issues attributed to Individual knowledge LTA does not appear to be an issue in and of itself but rather appears to signal a change in emphasis within licensee problem identification and corrective action programs.

A shift in focus from the training program to individual performance, however, should correspondingly also be reflected in the implementation of the SAT. Successful use of SAT requires that individual knowledge deficiencies be assessed during the program evaluation phase to determine if the group of individual performance issues indicates problems with the overall effectiveness of the training program. Corrective actions focused on separate individual performance deficiencies are likely to successfully address the immediate problem. However, incomplete evaluation of the group of individual performance problems within a training program is likely to be the cause of any ongoing increases in the number of performance problems attributed to insufficient individual knowledge. Therefore, an increase in the number of individual performance problems may signal a problem with the use of SAT in the area of training program evaluation.

Training program evaluation problems have been documented as a weakness during NRC for-cause inspections of training over the past four years and continued to challenge the industry in 1999. Licensees should understand the link between individual performance and training program evaluation in order to minimize the possibility that training program deficiencies will go uncorrected and will contribute to risk-significant operational events.

### BACKFIT DISCUSSION

This RIS does not request any action or written response; therefore, the staff did not perform a backfit analysis.

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<sup>1</sup>Training was provided and was attended by the worker, but the content of the training was incorrect or incomplete.

<sup>2</sup>Complete and accurate training was received by the worker, but the worker was unable to perform successfully on the job.

FEDERAL REGISTER NOTIFICATION

A notice of opportunity for public comment on this RIS was not published in the *Federal Register* because this RIS is informational only.

PAPERWORK REDUCTION ACT STATEMENT

This RIS does not request any information collection.

If there is any question about this RIS, please contact the person listed below.

**/RA/**Charles E. Ader For

David B. Matthews, Director  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Technical contact: Mary Ann M. Ashley, NRR  
301-415-1073  
E-mail: [mab@nrc.gov](mailto:mab@nrc.gov)

## INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b> 2000/09/07	<b>DATE RECEIVED</b>
<b>EVENT TITLE</b> RECOMMENDATIONS FOR ENSURING CONTINUED SAFE PLANT OPERATION AND MINIMIZING REQUESTS FOR ENFORCEMENT DISCRETION DURING EXTREME WEATHER CONDITIONS (NRC Regulatory Issue Summary 2000-15)		
<b>COUNTRY</b> USA	<b>PLANT AND UNIT</b> Generic	<b>REACTOR TYPE</b> (BWR or PWR)
<b>INITIAL STATUS</b> N/A	<b>RATED POWER (MWe NET)</b> N/A	
<b>DESIGNER</b> (WEST, GE, CE, B&W)	<b>1st COMMERCIAL OPERATION</b> N/A	

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### ABSTRACT

This IRS report discusses the implementation of anticipatory measures for ensuring continued safe plant operation during extreme weather conditions and to reduce the possibility of having to request enforcement discretion. The NRC's "General Statement of Policy and Procedures for NRC Enforcement Actions (Enforcement Policy)," published as NUREG-1600, establishes NRC policy for granting or denying requests for enforcement discretion. The exercise of enforcement discretion is accomplished through the issuance of a notice of enforcement discretion (NOED).

RECOMMENDATIONS FOR ENSURING CONTINUED SAFE PLANT OPERATION AND  
MINIMIZING REQUESTS FOR ENFORCEMENT DISCRETION DURING EXTREME WEATHER  
CONDITIONS (NRC Regulatory Issue Summary 2000-15)

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1.	Reporting Categories:	<u>1.4</u>	_____	_____
2.	Plant Status Prior to the Event:	<u>2.0</u>	_____	_____
3.	Failed/Affected Systems:	<u>3.Z</u>	_____	_____
4.	Failed/Affected Components:	<u>4.0</u>	_____	_____
5.	Cause of the Event:	<u>5.1.0</u>	_____	_____
			_____	_____
6.	Effects on Operation:	<u>6.0</u>	_____	_____
7.	Characteristics of the Incident:	<u>7.0</u>	_____	_____
8.	Nature of Failure or Error:	<u>8.0</u>	_____	_____
9.	Nature of Recovery Actions:	<u>9.0</u>	_____	_____

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF NUCLEAR REACTOR REGULATION  
WASHINGTON, D.C. 20555-0001

September 7, 2000

**NRC REGULATORY ISSUE SUMMARY 2000-15  
RECOMMENDATIONS FOR ENSURING CONTINUED SAFE PLANT  
OPERATION AND MINIMIZING REQUESTS FOR ENFORCEMENT  
DISCRETION DURING EXTREME WEATHER CONDITIONS**

ADDRESSEES

All holders of operating licenses for nuclear power reactors, except those who have permanently ceased operations and have certified that fuel has been permanently removed from the reactor vessel.

INTENT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this regulatory issue summary (RIS) to advise addressees regarding the implementation of anticipatory measures for ensuring continued safe plant operation during extreme weather conditions and to reduce the possibility of having to request enforcement discretion. The NRC's "General Statement of Policy and Procedures for NRC Enforcement Actions (Enforcement Policy)," published as NUREG-1600, establishes NRC policy for granting or denying requests for enforcement discretion. The exercise of enforcement discretion is accomplished through the issuance of a notice of enforcement discretion (NOED). This RIS does not transmit any new requirements or staff positions. No specific action or written response is required.

BACKGROUND INFORMATION

The NRC expects all nuclear power plant licensees to operate their facilities safely and in compliance with the NRC regulations, the plant license, including technical specifications (TS), and other requirements. Nevertheless, unusual circumstances may occur in which explicit compliance with an NRC requirement could result in an unnecessary plant transient or unnecessary delays in plant startup, or the performance of a test, inspection, or system realignment that is inappropriate for the prevailing plant conditions. In these circumstances, licensees may request that the NRC exercise discretion and refrain from enforcing applicable technical specifications before a violation occurs. This type of enforcement discretion is implemented through a NOED, which is addressed in Section VII.C of the Enforcement Policy. The NRC will issue a NOED only when it is clear that such an action is consistent with the agency's mission to protect public health and safety.

Although rare, prolonged severe weather conditions may result in issuance by another Government agency or a responsible independent entity of an advisory assessment regarding

ML003741172

the demand for electrical power and associated public (non-radiological) health and safety concerns. In these circumstances, licensees may request and the staff may grant a NOED after weighing the public health and safety implications of shutting down a nuclear power plant against the potential radiological or other safety hazards associated with continued plant operation even though such operation may not be in strict conformance with all license requirements.

### SUMMARY OF ISSUE

Electricity production and delivery systems are vitally important elements of the North American economic and social infrastructure that must remain dependable at all times. Most other critical elements of this infrastructure depend on the availability of an interconnected, stable and reliable electrical power supply network. This requirement is especially significant during extreme weather conditions when the need and demand for power can be high. Any potential noncompliance with the facility license that could result in an unexpected plant shutdown during such weather conditions could adversely affect the stability of the electrical power generation and transmission network, with a concomitant adverse impact on the public health and safety. Continued safe operation of nuclear power plants during severe weather conditions is essential to maintaining a stable and reliable electrical power supply system.

The Enforcement Policy provides for the exercise of enforcement discretion under circumstances in which maintaining the stability and reliability of the electrical power supply system is consistent with protecting the public health and safety. The enforcement policy describes the criteria that the NRC staff considers in granting NOED requests.

Through periodic communications with the Department of Energy, the Federal Energy Regulatory Commission, other Government agencies, the North American Electric Reliability Council, and independent grid operators, as applicable, both the NRC staff and licensees are made aware of the occurrence of extreme weather conditions and the critical need for power. As such, extreme weather conditions should not come as a surprise to either licensees or the NRC staff. In fact, it may be prudent to implement appropriate preventive measures to minimize the need for NOEDs when such conditions develop. Weather-related NOED requests usually involve a missed surveillance, an improperly scheduled surveillance, or inoperable equipment. Therefore, licensees may want to ensure that required surveillances are current, or are scheduled and completed well before anticipated extreme weather conditions develop, and to implement plans for periodic walkdown inspections or other measures to ensure that the condition of plant equipment is good and that potentially inoperable equipment does not prompt a NOED request. These anticipatory activities may preclude the subsequent identification of missed surveillances, the need to perform potential load-threatening surveillances, or the need to restore inoperable equipment to operable status within a short allowable outage time during extreme weather conditions. Alternatively, missed surveillances or inoperable equipment conditions could result in the occurrence of potential noncompliance conditions and the need for NOED requests.

In spite of every reasonable effort that a licensee may take, unanticipated situations may arise and a NOED may be necessary for continued operation of a facility. In this event, licensees are encouraged to engage the NRC staff early, particularly in time-critical situations, with complete

and relevant information, as specified in NRC Inspection Manual Part 9900, to permit the timely evaluation of NOED requests by the staff. In the absence of NRC approval of the NOED request, a licensee must take action in conformance with facility license conditions and applicable regulations. A licensee may take reasonable action that departs from a license condition or technical specification, however, if the conditions of 10 CFR 50.54(x) are satisfied.

#### BACKFIT DISCUSSION

This regulatory issue summary requires no action or written response. Any action on the part of addressees regarding the implementation of anticipatory measures for ensuring continued safe plant operation during extreme weather conditions and to reduce the possibility of having to request enforcement discretion, is strictly voluntary. Consequently, the staff did not perform a backfit analysis.

#### FEDERAL REGISTER NOTIFICATION

The staff did not publish a notice of opportunity for public comment in the *Federal Register* because this regulatory issue summary is informational and pertains to a staff procedure. The policy that the procedure implements was published in the *Federal Register* and the NRC staff considered all of the comments it received.

#### PAPERWORK REDUCTION ACT STATEMENT

This RIS does not request any information collection.

Please contact the lead project manager listed below if there are any questions about this matter.

**/RA/**Charles E. Ader for

David B. Matthews, Director  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Lead Project Manager: L. Raghavan, NRR  
301-415-1471  
E-mail: lxr1@nrc.gov

## INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b> 2000/09/07	<b>DATE RECEIVED</b>
<b>EVENT TITLE</b>		
AVAILABILITY OF THE REACTOR VESSEL INTEGRITY DATABASE VERSION 2.0.1 (NRC Regulatory Issue Summary 2000-16)		
<b>COUNTRY</b> USA	<b>PLANT AND UNIT</b> Generic	<b>REACTOR TYPE</b> (BWR or PWR)
<b>INITIAL STATUS</b> N/A	<b>RATED POWER (MWe NET)</b> N/A	
<b>DESIGNER</b> (WEST, GE, CE, B&W)	<b>1st COMMERCIAL OPERATION</b> N/A	

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### ABSTRACT

This IRS report discusses the availability of Version 2.0.1 of the Reactor Vessel Integrity Database (RVID). This regulatory issue summary (RIS) does not transmit any new requirements or staff positions. The RVID was developed following NRC staff review of licensee responses to Generic Letter (GL) 92-01, Revision 1, "Reactor Vessel Structural Integrity." The database was designed and developed to reflect the current status of reactor pressure vessel integrity, and the data are consolidated in a convenient and accessible manner. Some of the data categories represent inputs of docketed information; other data categories are representative of computed values that are not necessarily docketed. The programming logic used for calculations in the database follows the methodology in NRC Regulatory Guide (RG) 1.99, Revision 2, "Radiation Embrittlement of Reactor Vessel Materials." The RVID summarizes the properties of the reactor vessel beltline materials for each operating commercial nuclear power plant. For plants that ceased operation between the issuance of the initial version of the RVID and the current version, existing data have been maintained (i.e., no existing data for non-operating plants have been deleted).

AVAILABILITY OF THE REACTOR VESSEL INTEGRITY DATABASE VERSION 2.0.1 (NRC  
Regulatory Issue Summary 2000-16)

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1.	Reporting Categories:	<u>1.2.2</u>	_____	_____
2.	Plant Status Prior to the Event:	<u>2.0</u>	_____	_____
3.	Failed/Affected Systems:	<u>3.AC</u>	_____	_____
4.	Failed/Affected Components:	<u>4.2.5</u>	_____	_____
5.	Cause of the Event:	<u>5.1.1.1</u>	<u>5.1.1.7</u>	_____
		_____	_____	_____
6.	Effects on Operation:	<u>6.0</u>	_____	_____
7.	Characteristics of the Incident:	<u>7.2</u>	_____	_____
8.	Nature of Failure or Error:	<u>8.0</u>	_____	_____
9.	Nature of Recovery Actions:	<u>9.0</u>	_____	_____

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF NUCLEAR REACTOR REGULATION  
WASHINGTON, D.C. 20555-0001

September 7, 2000

**NRC REGULATORY ISSUE SUMMARY 2000-16**  
**AVAILABILITY OF THE REACTOR VESSEL INTEGRITY DATABASE**  
**VERSION 2.0.1**

ADDRESSEES

All holders of operating licenses for nuclear power reactors, except those licensees who have permanently ceased operations and have certified that fuel has been permanently removed from the reactor vessel.

INTENT

To inform addressees of the availability of Version 2.0.1 of the Reactor Vessel Integrity Database (RVID). This regulatory issue summary (RIS) does not transmit any new requirements or staff positions. Furthermore, no specific action or written response is required.

BACKGROUND INFORMATION

The RVID was developed following NRC staff review of licensee responses to Generic Letter (GL) 92-01, Revision 1, "Reactor Vessel Structural Integrity." The database was designed and developed to reflect the current status of reactor pressure vessel integrity, and the data are consolidated in a convenient and accessible manner. Some of the data categories represent inputs of docketed information; other data categories are representative of computed values that are not necessarily docketed. The programming logic used for calculations in the database follows the methodology in NRC Regulatory Guide (RG) 1.99, Revision 2, "Radiation Embrittlement of Reactor Vessel Materials."

The RVID summarizes the properties of the reactor vessel beltline materials for each operating commercial nuclear power plant. For plants that ceased operation between the issuance of the initial version of the RVID and the current version, existing data have been maintained (i.e., no existing data for non-operating plants have been deleted).

The original RVID was a DOS application developed with FoxPro™ software. The RVID was updated with new data and references based on the staff's review of the responses to GL 92-01, Revision 1, Supplement 1. The staff released the database in June 1999 as RVID Version 2 (RVID 2), a Windows 3.1 native application based on Microsoft Access 2.0™. The database runs in the higher Windows environments as well (e.g., Windows 95, Windows NT).

ML003739887



Contact:

If you have any questions, please contact the following person:

E-mail: [415.273.6000](mailto:415.273.6000), NRR

Regional Office of Regulation, Air Permit