



**Northeast
Nuclear Energy**

Rope Ferry Rd. (Route 156), Waterford, CT 06385

Millstone Nuclear Power Station
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385-0128
(860) 447-1791
Fax (860) 444-4277

The Northeast Utilities System

AUG 28 2000

Docket Nos. 50-336
50-423
B18211

Re: 10 CFR 50.90

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

**Millstone Nuclear Power Station, Unit Nos. 2 and 3
Proposed Revision to Technical Specifications
Radiological Effluent Technical Specifications**

In a letter dated February 22, 2000,⁽¹⁾ Northeast Nuclear Energy Company (NNECO) proposed to amend Operating Licenses DPR-65 and NPF-49 by incorporating changes into the Technical Specifications of Millstone Unit Nos. 2 and 3. The proposed changes will relocate selected Technical Specifications related to Radiological Effluent and the associated Bases to the Millstone Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMDCM). The purpose of this letter is to provide the Nuclear Regulatory Commission (NRC) with updated retyped pages of the affected Technical Specifications and to provide clarifications to the discussion of Technical Specifications changes in the February 22, 2000, letter.

Since February 22, 2000, several Unit Nos. 2 and 3 license amendments have been issued, which impacted the retyped pages. Therefore, NNECO is providing updated retyped pages of the affected Technical Specifications to reflect the changes introduced by these license amendments. Attachments 1 and 2 provide the updated retyped pages of the Technical Specifications for Unit Nos. 2 and 3, respectively. The pages provided in Attachments 1 and 2 should replace the pages provided in Attachments 5 and 6 of the letter dated February 22, 2000, respectively.

Additionally, this letter provides clarifications to the discussion of Technical Specifications changes in the letter dated February 22, 2000. Item 4.d on page 11 of

⁽¹⁾ R. P. Necci letter to the Nuclear Regulatory Commission, "Millstone Nuclear Power Station, Unit Nos. 2 and 3, Proposed Revision to Technical Specifications, Radiological Effluent Technical Specifications," dated February 22, 2000.

Handwritten signature/initials: A009

Attachment 1 discusses the removal of reference to Specifications 3.11.1.2, 3.11.2.2, 3.11.2.3 and 3.11.4 from Technical Specification 6.9.2.h. Technical Specification 3.11.4 does not exist in Millstone Unit No. 2 Technical Specifications. The reference should have been to Specification 3.11.3 instead. The reference in Specification 6.9.2.h to Specification 3.11.4 was introduced by an error in Amendment No. 104,⁽²⁾ which added to the Millstone Unit No. 2 Technical Specifications sections defining limiting conditions for operation and surveillance requirements associated with radioactive liquid and gaseous effluent monitoring; concentration, dose and treatment of liquid, and gaseous wastes; and total dose. Please also note that item number 6 on page 11 should have been number 5.

There are no regulatory commitments contained within this letter.

If you should have any questions on the above, please contact Mr. Ravi Joshi at (860) 440-2080.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY



Raymond P. Necci
Vice President - Nuclear Technical Services

Sworn to and subscribed before me

this 28 day of August, 2000

Donna Lynne Williams
Notary Public

My Commission expires Nov 30, 2001

cc: See next page

⁽²⁾ D. I. Osborne to J. F. Opeka, "Issuance of Amendment No. 104 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2," dated September 16, 1985.

Attachments (2)

cc: H. J. Miller, Region I Administrator
J. I. Zimmerman, NRC Project Manager, Millstone Unit No. 2
S. R. Jones, Senior Resident Inspector, Millstone Unit No. 2
V. Nerses, NRC Senior Project Manager, Millstone Unit No. 3
A. C. Cerne, Senior Resident Inspector, Millstone Unit No. 3

Director
Bureau of Air Management
Monitoring and Radiation Division
Department of Environmental Protection
79 Elm Street
Hartford, CT 06106-5127

Docket Nos. 50-336
50-423
B18211

Attachment 1

Millstone Nuclear Power Station, Unit No. 2

Proposed Revision to Technical Specifications
Radiological Effluent Technical Specifications
Retyped Pages

ATTACHMENT TO LICENSE AMENDMENT NO.
FACILITY OPERATING LICENSE NO. DPR-65
DOCKET NO. 50-336

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
II	II
V	V
X	X
XIV	XIV
XVII	XVII
1-6	1-6
3/4 3-50	3/4 3-50
3/4 3-51	3/4 3-51
3/4 3-52	3/4 3-52
3/4 3-53	3/4 3-53
3/4 3-54	3/4 3-54
3/4 3-55	3/4 3-55
3/4 3-56	3/4 3-56
3/4 3-57	3/4 3-57
3/4 3-58	3/4 3-58
3/4 3-59	3/4 3-59
3/4 3-60	3/4 3-60
3/4 10-2	3/4 10-2
3/4 10-5	3/4 10-5
3/4 11-1	3/4 11-1
3/4 11-2	3/4 11-2
3/4 11-3	3/4 11-3
3/4 11-4	3/4 11-4
3/4 11-5	3/4 11-5
3/4 11-6	3/4 11-6
B3/4 3-5	B3/4 3-5
B3/4 3-6	B3/4 3-6
B3/4 11-1	B3/4 11-1
B3/4 11-2	B3/4 11-2
B3/4 11-3	B3/4 11-3
B3/4 11-4	B3/4 11-4
6-15	6-15
6-18	6-18

ATTACHMENT TO LICENSE AMENDMENT NO.
FACILITY OPERATING LICENSE NO. DPR-65
DOCKET NO. 50-336

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
6-18a	6-18a
6-19	6-19
6-20	6-20
6-24	6-24
6-25	6-25
6-26	6-26
-----	6-27
-----	6-28

INDEX

DEFINITIONS

<u>SECTION</u>	<u>PAGE</u>
Reactor Trip System Response Time	1-5
Engineered Safety Feature Response Time	1-5
Physics Tests	1-6
Unrodded Integrated Radial Peaking Factor - F_r	1-6
Source Check	1-6
Purge - Purging	1-6
Venting	1-8
Member(s) of the Public	1-8
Site Boundary	1-8
Unrestricted Area	1-8
Storage Pattern	1-8

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>		<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>		
3/4.2.1	LINEAR HEAT RATE	3/4 2-1
3/4.2.2	Deleted	
3/4.2.3	TOTAL INTEGRATED RADIAL PEAKING FACTOR - F_r^T	3/4 2-9
3/4.2.4	AZIMUTHAL POWER TILT	3/4 2-10
3/4.2.5	Deleted	
3/4.2.6	DNB MARGIN	3/4 2-13
<u>3/4.3 INSTRUMENTATION</u>		
3/4.3.1	REACTOR PROTECTIVE INSTRUMENTATION	3/4 3-1
3/4.3.2	ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION	3/4 3-10
3/4.3.3	MONITORING INSTRUMENTATION	3/4 3-26
	Radiation Monitoring	3/4 3-26
	Remote Shutdown Instrumentation	3/4 3-39
	Accident Monitoring	3/4 3-46
3/4.3.4	CONTAINMENT PURGE VALVE ISOLATION SIGNAL	3/4 3-61
<u>3/4.4 REACTOR COOLANT SYSTEM</u>		
3/4.4.1	COOLANT LOOPS AND COOLANT CIRCULATION	3/4 4-1
	Startup and Power Operation	3/4 4-1
	Hot Standby	3/4 4-1a
	Shutdown	3/4 4-1b
	Reactor Coolant Pumps - Shutdown	3/4 4-1d

INDEX

LIMITING CONDITION FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>		<u>PAGE</u>
3/4.10.4	PHYSICS TESTS	3/4 10-4
3/4.10.5	CENTER CEA MISALIGNMENT	3/4 10-5
 <u>3/4.11 DELETED</u>		
3/4.11.1	DELETED	
3/4.11.2	DELETED	
3/4.11.3	DELETED	

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
3/4.9.9 and 3/4.9.10 DELETED	B 3/4 9-2
3/4.9.11 and 3/4.9.12 WATER LEVEL - REACTOR VESSEL AND STORAGE POOL WATER LEVEL	B 3/4 9-2a
3/4.9.13 DELETED	B 3/4 9-3
3/4.9.14 DELETED	B 3/4 9-3
3/4.9.15 STORAGE POOL AREA VENTILATION SYSTEM	B 3/4 9-3
3/4.9.16 SHIELDED CASK	B 3/4 9-3
3/4.9.17 MOVEMENT OF FUEL IN SPENT FUEL POOL	B 3/4 9-3
3/4.9.18 SPENT FUEL POOL - REACTIVITY CONDITION	B 3/4 9-3
3/4.9.19 SPENT FUEL POOL - STORAGE PATTERN	B 3/4 9-4
3/4.9.20 SPENT FUEL POOL - CONSOLIDATION	B 3/4 9-4
 <u>3/4.10 SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1 SHUTDOWN MARGIN	B 3/4 10-1
3/4.10.2 GROUP HEIGHT AND INSERTION LIMITS	B 3/4 10-1
3/4.10.3 PRESSURE/TEMPERATURE LIMITATION - REACTOR CRITICALITY	B 3/4 10-1
3/4.10.4 PHYSICS TESTS	B 3/4 10-1
3/4.10.5 CENTER CEA MISALIGNMENT	B 3/4 10-1
 <u>3/4.11 DELETED</u>	
3/4.11.1 DELETED	B 3/4 11-1
3/4.11.2 DELETED	B 3/4 11-1
3/4.11.3 DELETED	B 3/4 11-1

INDEX

ADMINISTRATIVE CONTROLS

<u>SECTION</u>	<u>PAGE</u>
<u>6.9 REPORTING REQUIREMENTS</u>	
6.9.1 ROUTINE REPORTS	6-16
STARTUP REPORTS	6-16
ANNUAL REPORTS	6-17
ANNUAL RADIOLOGICAL REPORTS	6-18
MONTHLY OPERATING REPORT	6-18
CORE OPERATING LIMITS REPORT	6-18
6.9.2 SPECIAL REPORTS	6-19
<u>6.10 DELETED</u>	
6.11 <u>RADIATION PROTECTION PROGRAM</u>	6-22
6.12 <u>HIGH RADIATION AREA</u>	6-22
6.13 <u>SYSTEMS INTEGRITY</u>	6-23
6.14 <u>IODINE MONITORING</u>	6-23
6.15 <u>RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE</u> <u>CALCULATION MANUAL (REMODCM)</u>	6-24
6.16 <u>RADIOACTIVE WASTE TREATMENT</u>	6-24
6.17 <u>SECONDARY WATER CHEMISTRY</u>	6-25
6.18 <u>PASS/SAMPLING AND ANALYSIS OF PLANT EFFLUENTS</u>	6-25
6.19 <u>CONTAINMENT LEAKAGE RATE TESTING PROGRAM</u>	6-26
6.20 <u>RADIOACTIVE EFFLUENT CONTROLS PROGRAM</u>	6-26
6.21 <u>RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM</u>	6-27

DEFINITIONS

ENGINEERED SAFETY FEATURE RESPONSE TIME (Continued)

performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable.

PHYSICS TESTS

1.28 PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation and 1) described in Chapter 13.0 of the FSAR, 2) authorized under the provisions of 10 CFR 50.59, or 3) otherwise approved by the Commission.

TOTAL UNRODDED INTEGRATED RADIAL PEAKING FACTOR - F_r^T

1.29 The TOTAL UNRODDED INTEGRATED RADIAL PEAKING FACTOR is the ratio of the peak pin power to the average pin power in an unrodded core. This value includes the effect of AZIMUTHAL POWER TILT.

SOURCE CHECK

1.30 A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to radiation.

PURGE - PURGING

1.34 PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the containment.

This page intentionally left blank

SPECIAL TEST EXCEPTIONS

GROUP HEIGHT AND INSERTION LIMITS

LIMITING CONDITION FOR OPERATION

3.10.2 The requirements of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.2.3 and 3.2.4 may be suspended during the performance of PHYSICS TESTS provided:

- a. The THERMAL POWER is restricted to the test power plateau which shall not exceed 85% of RATED THERMAL POWER, and
- b. The limits of Specification 3.2.1 are maintained and determined as specified in Specification 4.10.2 below.

APPLICABILITY: MODES 1 and 2.

ACTION:

With any of the limits of Specification 3.2.1 being exceeded while the requirements of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.2.3 and 3.2.4 are suspended, immediately:

- a. Reduce THERMAL POWER sufficiently to satisfy the requirements of Specification 3.2.1 or
- b. Be in HOT STANDBY within 2 hours.

SURVEILLANCE REQUIREMENTS

4.10.2.1 The THERMAL POWER shall be determined at least once per hour during PHYSICS TESTS in which the requirements of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.2.3 or 3.2.4 are suspended and shall be verified to be within the test power plateau.

4.10.2.2 The linear heat rate shall be determined to be within the limits of Specification 3.2.1 by monitoring it continuously with the Incore Detector Monitoring System pursuant to the requirements of Specification 4.2.1.3 during PHYSICS TESTS above 5% of RATED THERMAL POWER in which the requirements of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.2.3 or 3.2.4 are suspended.

SPECIAL TEST EXCEPTIONS

CENTER CEA MISALIGNMENT

LIMITING CONDITION FOR OPERATION

3.10.5 The requirements of Specifications 3.1.3.1 and 3.1.3.6 may be suspended during the performance of PHYSICS TESTS to determine the isothermal temperature coefficient and power coefficient provided:

- a. Only the center CEA (CEA #1) is misaligned, and
- b. The limits of Specification 3.2.1 are maintained and determined as specified in Specification 4.10.5 below.

APPLICABILITY: MODES 1 and 2.

ACTION:

With any of the limits of Specification 3.2.1 being exceeded while the requirements of Specification 3.1.3.1 and 3.1.3.6 are suspended, immediately:

- a. Reduce THERMAL POWER sufficiently to satisfy the requirements of Specification 3.2.1, or
- b. Be in HOT STANDBY within 2 hours.

SURVEILLANCE REQUIREMENTS

4.10.5.1 The THERMAL POWER shall be determined at least once per hour during PHYSICS TESTS in which the requirements of Specifications 3.1.3.1 and/or 3.1.3.6 are suspended and shall be verified to be within the test power plateau.

4.10.5.2 The linear heat rate shall be determined to be within the limits of Specification 3.2.1 by monitoring it continuously with the Incore Detector Monitoring System pursuant to the requirements of Specification 4.2.1.3 during PHYSICS TESTS above 5% of RATED THERMAL POWER in which the requirements of Specifications 3.1.3.1 and/or 3.1.3.6 are suspended.

This page intentionally left blank

INSTRUMENTATION

BASES

3/4.3.3.9 - DELETED

3/4.3.3.10 - DELETED

3/4.3.4 Containment Purge Valve Isolation Signal

The containment purge valve may remain open during CORE ALTERATIONS and the movement of irradiated fuel assemblies inside containment provided the automatic closure of the purge valves on high containment radiation is OPERABLE. A high airborne radioactivity level inside containment will be detected by the containment airborne radiation monitors (gaseous and particulate). The actuation logic for this function is one out of four. High radioactivity inside containment, detected by any one of the four radiation detectors (two gaseous and two particulate), will automatically isolate containment purge.

An OPERABLE system capable of generating a Containment Purge Valve Isolation Signal consists of at least one containment gaseous radiation monitor channel, at least one containment particulate radiation monitor channel, and the associated automatic actuation logic train. An actuation logic train consists of the detectors, sensor channels, and logic circuits up to and including the Engineered Safeguards Actuation System actuation module.

The analysis of a fuel handling accident inside containment credits automatic isolation of containment purge. Therefore, this function is required to be OPERABLE if the containment purge valves are to remain open during CORE ALTERATIONS and the movement of irradiated fuel assemblies inside containment. If this automatic isolation function is not OPERABLE, the containment purge valves must be maintained closed to perform CORE ALTERATIONS or the movement of irradiated fuel assemblies inside containment.

The ACTION requirements to immediately suspend CORE ALTERATIONS and the movement of irradiated fuel assemblies inside containment, or to immediately close the containment purge valves and maintain them closed, does not preclude completion of the movement of a component to a safe position.

3/4.11 DELETED

BASES

3/4.11.1 - DELETED

3/4.11.2 - DELETED

3/4.11.3 - DELETED

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

ADMINISTRATIVE CONTROLS

- f. Fire Protection Program implementation.
 - g. Quality Control for effluent monitoring using the guidance in Regulatory Guide 1.21 Rev. 1, June 1974.
 - h. Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMDCM) implementation, except for Section I.E., Radiological Environmental Monitoring.
- 6.8.2
- a. The designated manager or designated officer or designated senior officer may designate specific procedures and programs, or classes of procedures and programs to be reviewed in accordance with the Quality Assurance Program Topical Report.
 - b. Procedures and programs listed in Specification 6.8.1, and changes thereto, shall be approved by the designated manager or designated officer or by cognizant Manager or Directors who are designated as the Approval Authority by the designated manager or designated officer, as specified in administrative procedures. The Approval Authority for each procedure and program or class of procedure and program shall be specified in administrative procedures.
 - c. Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed and approved in accordance with the Quality Assurance Program Topical Report, prior to implementation. Each procedure of Specification 6.8.1 shall be reviewed periodically as set forth in administrative procedures.
- 6.8.3
- Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
 - c. The change is documented, reviewed and approved in accordance with the Quality Assurance Program Topical Report within 14 days of implementation.
- 6.8.4
- Written procedures shall be established, implemented and maintained covering Section I.E, Radiological Environmental Monitoring, of the REMDCM.

ADMINISTRATIVE CONTROLS

- d. Documentation of all failures (inability to lift or reclose within the tolerances allowed by the design basis) and challenges to the pressurizer PORVs or safety valves.

ANNUAL RADIOLOGICAL REPORTS

6.9.1.6a ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT

-----NOTE-----

A single submittal may be made for a multiple unit station. The submittal shall combine sections common to all units at the station.

The Annual Radiological Environmental Operating Report covering the operation of the unit during the previous calendar year shall be submitted by May 1 of each year. The report shall include summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in the Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMDCM), and in 10 CFR Part 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C.

The Annual Radiological Environmental Operating Report shall include the results of analyses of all radiological environmental samples and of all environmental radiation measurements taken during the period pursuant to the locations specified in the table and figures in the REMDCM, as well as summarized and tabulated results of these analyses and measurements. In the event that some individual results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted in the next annual report.

6.9.1.6b RADIOACTIVE EFFLUENT RELEASE REPORT

-----NOTE-----

A single submittal may be made for a multiple unit station. The submittal shall combine sections common to all units at the station; however, for units with separate radwaste systems, the submittal shall specify the releases of radioactive material from each unit.

The Radioactive Effluent Release Report covering the operation of the unit in the previous year shall be submitted prior to May 1 of each year in accordance with 10 CFR 50.36a. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided shall be consistent with the objectives outlined in the REMDCM and in conformance with 10 CFR 50.36a and 10 CFR Part 50, Appendix I, Section IV.B.1.

MONTHLY OPERATING REPORT

- 6.9.1.7 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the U.S. Nuclear Regulatory Commission, Document Control Desk, Washington, D.C. 20555, one copy to the Regional

ADMINISTRATIVE CONTROLS

MONTHLY OPERATING REPORT (Con't)

Administrator, Region I, and one copy to the NRC Resident Inspector, no later than the 15th of each month following the calendar month covered by the report.

CORE OPERATING LIMITS REPORT

6.9.1.8 a. Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle.

- 3/4.1.1.1 SHUTDOWN MARGIN - $T_{avg} > 200^{\circ}\text{F}$
- 3/4.1.1.2 SHUTDOWN MARGIN - $T_{avg} \leq 200^{\circ}\text{F}$
- 3/4.1.1.4 Moderator Temperature Coefficient
- 3/4.1.3.6 Regulating CEA Insertion Limits
- 3/4.2.1 Linear Heat Rate
- 3/4.2.3 Total Integrated Radial Peaking Factor - F_r^T
- 3/4.2.6 DNB Margin

b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

- 1) EMF-96-029(P)(A) Volumes 1 and 2, "Reactor Analysis System for PWRs Volume 1 - Methodology Description, Volume 2 - Benchmarking Results," Siemens Power Corporation, January 1997.
- 2) ANF-84-73 Revision 5 Appendix B (P)(A), "Advanced Nuclear Fuels Methodology for Pressurized Water Reactors: Analysis of Chapter 15 Events," Advanced Nuclear Fuels, July 1990.
- 3) XN-NF-82-21(P)(A) Revision 1, "Application of Exxon Nuclear Company PWR Thermal Margin Methodology to Mixed Core Configurations," Exxon Nuclear Company, September 1983.
- 4) EMF-84-093(P)(A) Revision 1, "Steamline Break Methodology for PWRs," Siemens Power Corporation, February 1999.
- 5) XN-75-32(P)(A) Supplements 1 through 4, "Computational Procedure for Evaluating Fuel Rod Bowing," Exxon Nuclear Company, October 1983.
- 6) XN-NF-82-49(P)(A) Revision 1, "EXXON Nuclear Company Evaluation Model EXEM PWR Small Break Model," Advanced Nuclear Fuels Corporation, April 1989.
- 7) XN-NF-82-49(P)(A) Revision 1 Supplement 1, "Exxon Nuclear Company Evaluation Model Revised EXEM PWR Small Break Model," Siemens Power Corporation, December 1994.
- 8) EMF-2087(P)(A), "SEM/PWR-98: ECCS Evaluation Model for PWR LBLOCA Applications," Siemens Power Corporation, June 1999.

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (CONT.)

- 9) XN-NF-78-44(NP)(A), "A Generic Analysis of the Control Rod Ejection Transient for Pressurized water reactors," Exxon Nuclear Company, October 1983.
 - 10) XN-NF-621(P)(A) Revision 1, "Exxon Nuclear DNB Correlation for PWR Fuel Designs," Exxon Nuclear Company, September 1983.
 - 11) XN-NF-82-06(P)(A) Revision 1 and Supplements 2, 4, and 5, "Qualification of Exxon Nuclear Fuel for Extended Burnup," Exxon Nuclear Company, October 1986.
 - 12) ANF-88-133(P)(A) and Supplement 1, "Qualification of Advanced Nuclear Fuels PWR Design Methodology for Rod Burnups of 62 Gwd/MTU," Advanced Nuclear Fuels Corporation, December 1991.
 - 13) XN-NF-85-92(P)(A), "Exxon Nuclear Uranium Dioxide/Gadolinia Irradiation Examination and Thermal Conductivity Results," Exxon Nuclear Company, November 1988.
 - 14) ANF-89-151(P)(A), "ANF-RELAP Methodology for Pressurized Water Reactors: Analysis of Non-LOCA Chapter 15 Events," Advanced Nuclear Fuels Corporation, May 1992.
 - 15) XN-NF-507(P)(A) Supplements 1 and 2, "ENC Setpoint Methodology for C.E. Reactors: Statistical Setpoint Methodology," Exxon Nuclear Company, September 1986.
- c. The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.
- d. The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

- 6.9.2 Special reports shall be submitted to the U.S. Nuclear Regulatory Commission, Document Control Desk, Washington, D.C. 20555, one copy to the Regional Administrator, Region I, and one copy to the NRC Resident Inspector within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:
- a. Deleted

ADMINISTRATIVE CONTROLS

SPECIAL REPORTS (CONT.)

- b. Deleted
- c. Deleted
- d. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- e. Deleted
- f. Deleted
- g. RCS Overpressure Mitigation, Specification 3.4.9.3.
- h. Deleted
- i. Degradation of containment structure, Specification 4.6.1.6.4.
- j. Steam Generator Tube Inspection, Specification 4.4.5.1.5.
- k. Accident Monitoring Instrumentation, Specification 3.3.3.8.
- l. Radiation Monitoring Instrumentation, Specification 3.3.3.1.
- m. Reactor Coolant System Vents, Specification 3.4.11.

6.10 Deleted.

ADMINISTRATIVE CONTROLS

6.15 RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE CALCULATION MANUAL (REMODCM)

- a. The REMODCM shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm and trip setpoints, and in the conduct of the radiological environmental monitoring program; and
- b. The REMODCM shall also contain the radioactive effluent controls and radiological environmental monitoring activities and descriptions of the information that should be included in the Annual Radiological Environmental Operating, and Radioactive Effluent Release, reports required by Specification 6.9.1.6a and Specification 6.9.1.6b.

Licensee initiated changes to the REMODCM:

- a. Shall be documented and records of reviews performed shall be retained. This documentation shall contain:
 - 1) sufficient information to support the change(s) together with the appropriate analyses or evaluations justifying the change(s), and
 - 2) a determination that the change(s) will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I of 10 CFR 50, and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations;
- b. Shall become effective after review and acceptance by SORC and the approval of the designated officer; and
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire REMODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the REMODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

ADMINISTRATIVE CONTROLS

6.16 RADIOACTIVE WASTE TREATMENT

Procedures for liquid and gaseous radioactive effluent discharges from the Unit shall be prepared, approved, maintained, and adhered to for all operations involving offsite releases of radioactive effluents. These procedures shall specify the use of appropriate* waste treatment utilizing the guidance provided in the REMODCM.

6.17 SECONDARY WATER CHEMISTRY

A program shall be maintained for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

1. Identification of a sampling schedule for the critical variables and control points for these variables.
2. Identification of the procedures used to measure the values of the critical variables,
3. Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage.
4. Procedures for the recording and management of data.
5. Procedures defining corrective actions for all off-control point chemistry conditions, and
6. A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

6.18 PASS/Sampling and Analysis of Plant Effluents

A program shall be established, implemented and maintained which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for sampling and analysis,
- (iii) Provisions for maintenance of sampling & analysis equipment.

*The Solid Radioactive Waste Treatment System shall be operated in accordance with the Process Control Program to process wet radioactive wastes to meet shipping and burial ground requirements.

ADMINISTRATIVE CONTROLS

6.19 CONTAINMENT LEAKAGE RATE TESTING PROGRAM

A program shall be established to implement the leakage rate testing of the primary containment as required by 10CFR50.54(o) and 10CFR50, Appendix J, Option B as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.

The peak calculated primary Containment internal pressure for the design basis loss of coolant accident is P_a .

The maximum allowable primary containment leakage rate, L_a , at P_a , is 0.5% of primary containment air weight per day.

Leakage rate acceptance criteria are:

- a. Primary containment overall leakage rate acceptance criterion is $< 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $< 0.60 L_a$ for the combined Type B and Type C tests, and $< 0.75 L_a$ for Type A tests;
- b. Air lock testing acceptance criteria are:
 1. Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 2. For each door, pressure decay is ≤ 0.1 psig when pressurized to ≥ 25 psig for at least 15 minutes.

The provisions of SR 4.0.2 do not apply for test frequencies specified in the Primary Containment Leakage Rate Testing Program.

The provisions of SR 4.0.3 are applicable to the Primary Containment Leakage Rate Testing Program.

6.20 RADIOACTIVE EFFLUENT CONTROLS PROGRAM

This program conforms to 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable. The program shall be contained in the REMODCM, shall be implemented by procedures, and shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- a. Limitations on the functional capability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the REMODCM;
- b. Limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas, conforming to 10 CFR 20, Appendix B, Table II, Column 2 (1993 version);

ADMINISTRATIVE CONTROLS

- c. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the REMODCM;
- d. Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released from each unit to unrestricted areas, conforming to 10 CFR 50, Appendix I;
- e. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the REMODCM at least every 31 days;
- f. Limitations on the functional capability and use of the liquid and gaseous effluent treatment systems to ensure that appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a period of 31 days would exceed 2% of the guidelines for the annual dose or dose commitment, conforming to 10 CFR 50, Appendix I;
- g. Limitations on the dose rate resulting from radioactive material released in gaseous effluents to areas beyond the site boundary to a dose rate which, if the release were to occur for a full year, would cause a dose of 500 mrem. This conforms to the dose associated with the 1993 version of 10 CFR 20, Appendix B, Table II, Column I;
- h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;
- i. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and
- j. Limitations on the annual dose or dose commitment to any member of the public due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.

6.21 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

A program shall be provided to monitor the radiation and radionuclides in the environs of the plant. The program shall provided (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in the REMODCM, (2) conform to that guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

- a. Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the REMODCM.

ADMINISTRATIVE CONTROLS

- b. A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census, and
- c. Participation in a Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

Docket Nos. 50-336
50-423
B18211

Attachment 2

Millstone Nuclear Power Station, Unit No. 3

**Proposed Revision to Technical Specifications
Radiological Effluent Technical Specifications
Retyped Pages**

ATTACHMENT TO LICENSE AMENDMENT NO.
FACILITY OPERATING LICENSE NO. NPF-49
DOCKET NO. 50-423

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
i	i
vi	vi
vii	vii
xii	xii
xvi	xvi
xix	xix
1-5	1-5
3/4 3-69	3/4 3-69
3/4 3-70	3/4 3-70
3/4 3-71	3/4 3-71
3/4 3-72	3/4 3-72
3/4 3-73	3/4 3-73
3/4 3-74	3/4 3-74
3/4 3-75	3/4 3-75
3/4 3-76	3/4 3-76
3/4 3-77	3/4 3-77
3/4 3-78	3/4 3-78
3/4 3-79	3/4 3-79
3/4 3-80	3/4 3-80
3/4 11-1	3/4 11-1
3/4 11-2	3/4 11-2
3/4 11-3	3/4 11-3
3/4 11-4	3/4 11-4
3/4 11-5	3/4 11-5
3/4 11-6	3/4 11-6
B3/4 3-6	B3/4 3-6
B3/4 11-1	B3/4 11-1
B3/4 11-2	B3/4 11-2
B3/4 11-3	B3/4 11-3
6-19	6-19

ATTACHMENT TO LICENSE AMENDMENT NO.
FACILITY OPERATING LICENSE NO. NPF-49
DOCKET NO. 50-423

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

Insert

6-24

6-19a
6-24
6-25
6-26

INDEX

DEFINITIONS

<u>SECTION</u>	<u>PAGE</u>
1.0 DEFINITIONS	
1.1 ACTION	1-1
1.2 ACTUATION LOGIC TEST	1-1
1.3 ANALOG CHANNEL OPERATIONAL TEST	1-1
1.4 AXIAL FLUX DIFFERENCE	1-1
1.5 CHANNEL CALIBRATION	1-1
1.6 CHANNEL CHECK	1-1
1.7 CONTAINMENT INTEGRITY	1-2
1.8 CONTROLLED LEAKAGE	1-2
1.9 CORE ALTERATIONS	1-2
1.10 DOSE EQUIVALENT I-131	1-2
1.11 E-AVERAGE DISINTEGRATION ENERGY	1-2
1.12 DELETED	
1.13 ENGINEERED SAFETY FEATURES RESPONSE TIME	1-3
1.14 DELETED	
1.15 FREQUENCY NOTATION	1-3
1.16 IDENTIFIED LEAKAGE	1-3
1.17 MASTER RELAY TEST	1-3
1.18 MEMBER(S) OF THE PUBLIC	1-4
1.19 OPERABLE - OPERABILITY	1-4
1.20 OPERATIONAL MODE - MODE	1-4
1.21 PHYSICS TESTS	1-4
1.22 PRESSURE BOUNDARY LEAKAGE	1-4
1.23 PURGE - PURGING	1-4
1.24 QUADRANT POWER TILT RATIO	1-5
1.25 DELETED	
1.26 DELETED	
1.27 RATED THERMAL POWER	1-5
1.28 REACTOR TRIP SYSTEM RESPONSE TIME	1-5
1.29 REPORTABLE EVENT	1-5
1.30 SHUTDOWN MARGIN	1-5
1.31 SITE BOUNDARY	1-5

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
TABLE 3.3-5 DELETED	
TABLE 4.3-2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-36
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring for Plant Operations	3/4 3-42
TABLE 3.3-6 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS	3/4 3-43
TABLE 4.3-3 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS SURVEILLANCE REQUIREMENTS	3/4 3-45
Movable Incore Detectors	3/4 3-46
Seismic Instrumentation	3/4 3-47
TABLE 3.3-7 SEISMIC MONITORING INSTRUMENTATION	3/4 3-48
TABLE 4.3-4 SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-49
Meteorological Instrumentation	3/4 3-50
TABLE 3.3-8 METEOROLOGICAL MONITORING INSTRUMENTATION	3/4 3-51
TABLE 4.3-5 METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-52
Remote Shutdown Instrumentation	3/4 3-53
TABLE 3.3-9 REMOTE SHUTDOWN INSTRUMENTATION	3/4 3-54
TABLE 4.3-6 REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-58
Accident Monitoring Instrumentation	3/4 3-59
TABLE 3.3-10 ACCIDENT MONITORING INSTRUMENTATION	3/4 3-60
TABLE 4.3-7 ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-62
TABLE 3.3-11 DELETED	
Loose-Part Detection System	3/4 3-68
TABLE 3.3-12 DELETED	
TABLE 4.3-8 DELETED	

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
TABLE 3.3-13 DELETED	
TABLE 4.3-9 DELETED	
3/4.3.4 TURBINE OVERSPEED PROTECTION	3/4 3-81
3/4.3.5 SHUTDOWN MARGIN MONITOR	3/4 3-82
 <u>3/4.4 REACTOR COOLANT SYSTEM</u>	
<u>3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION</u>	
Startup and Power Operation	3/4 4-1
Hot Standby	3/4 4-2
Hot Shutdown	3/4 4-3
Cold Shutdown - Loops Filled	3/4 4-5
Cold Shutdown - Loops Not Filled	3/4 4-6
Isolated Loop	3/4 4-7
Isolated Loop Startup	3/4 4-8
 <u>3/4.4.2 SAFETY VALVES</u>	
Shutdown	3/4 4-9
Operating	3/4 4-10
 <u>3/4.4.3 PRESSURIZER</u>	
Startup and Power Operation	3/4 4-11
FIGURE 3.4-5 PRESSURIZER LEVEL CONTROL	3/4 4-11a
Hot Standby	3/4 4-11b
3/4.4.4 RELIEF VALVES	3/4 4-12
3/4.4.5 STEAM GENERATORS	3/4 4-14
TABLE 4.4-1 MINIMUM NUMBER OF STEAM GENERATORS TO BE INSPECTED DURING INSERVICE INSPECTION	3/4 4-19
TABLE 4.4-2 STEAM GENERATOR TUBE INSPECTION	3/4 4-20
 <u>3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE</u>	
Leakage Detection Systems	3/4 4-21
Operational Leakage	3/4 4-22
TABLE 3.4-1 REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES	3/4 4-24
3/4.4.7 CHEMISTRY	3/4 4-25
TABLE 3.4-2 REACTOR COOLANT SYSTEM CHEMISTRY LIMITS	3/4 4-26
TABLE 4.4-3 REACTOR COOLANT SYSTEM CHEMISTRY LIMITS SURVEILLANCE REQUIREMENTS	3/4 4-27
3/4.4.8 SPECIFIC ACTIVITY	3/4 4-28

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
3/4.9.6	REFUELING MACHINE 3/4 9-6
3/4.9.7	CRANE TRAVEL - SPENT FUEL STORAGE AREAS 3/4 9-7
3/4.9.8	RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION
	High Water Level 3/4 9-8
	Low Water Level 3/4 9-9
3/4.9.9	CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM 3/4 9-10
3/4.9.10	WATER LEVEL - REACTOR VESSEL 3/4 9-11
3/4.9.11	WATER LEVEL - STORAGE POOL 3/4 9-12
3/4.9.12	FUEL BUILDING EXHAUST FILTER SYSTEM 3/4 9-13
3/4.9.13	SPENT FUEL POOL - REACTIVITY 3/4 9-16
3/4.9.14	SPENT FUEL POOL - STORAGE PATTERN 3/4 9-17
FIGURE 3.9-1	FUEL ASSEMBLY MINIMUM BURNUP VERSUS INITIAL U235
	ENRICHMENT FOR STORAGE IN REGION II SPENT FUEL RACKS . 3/4 9-18
FIGURE 3.9-2	REGION I THREE OR FOUR FUEL ASSEMBLY LOADING
	SCHEMATIC FOR A TYPICAL 6X6 STORAGE MODULE 3/4 9-19
 <u>3/4.10 SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1	SHUTDOWN MARGIN 3/4 10-1
3/4.10.2	GROUP HEIGHT, INSERTION, AND POWER DISTRIBUTION LIMITS
	Four Loops Operating 3/4 10-2
	Three Loops Operating 3/4 10-3
3/4.10.3	PHYSICS TESTS 3/4 10-4
3/4.10.4	REACTOR COOLANT LOOPS 3/4 10-5
3/4.10.5	POSITION INDICATION SYSTEM - SHUTDOWN 3/4 10-6
 <u>3/4.11 DELETED</u>	
3/4.11.1	DELETED
3/4.11.2	DELETED
3/4.11.3	DELETED

INDEX

BASES

SECTION

PAGE

3/4.11 DELETED

3/4.11.1 DELETED

3/4.11.2 DELETED

3/4.11.3 DELETED

INDEX

ADMINISTRATIVE CONTROLS

<u>SECTION</u>	<u>PAGE</u>
<u>6.8 PROCEDURES AND PROGRAMS</u>	6-14
<u>6.9 REPORTING REQUIREMENTS</u>	6-17
6.9.1 ROUTINE REPORTS	6-17
Startup Report	6-17
Annual Reports	6-18
Annual Radiological Environmental Operating Report	6-19
Annual Radioactive Effluent Release Report	6-19
Monthly Operating Reports	6-19
Core Operating Limits Report	6-19a
6.9.2 SPECIAL REPORTS	6-21
<u>6.10 DELETED</u>	
<u>6.11 RADIATION PROTECTION PROGRAM</u>	6-22
<u>6.12 HIGH RADIATION AREA</u>	6-23
<u>6.13 RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE CALCULATION MANUAL (REMODCM)</u>	6-24
<u>6.14 RADIOACTIVE WASTE TREATMENT</u>	6-24
<u>6.15 RADIOACTIVE EFFLUENT CONTROLS PROGRAM</u>	6-25
<u>6.16 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM</u>	6-26

DEFINITIONS

QUADRANT POWER TILT RATIO

1.24 QUADRANT POWER TILT RATIO shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper excore detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater. With one excore detector inoperable, the remaining three detectors shall be used for computing the average.

RATED THERMAL POWER

1.27 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3411 Mwt.

REACTOR TRIP SYSTEM RESPONSE TIME

1.28 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its Trip Setpoint at the channel sensor until loss of stationary gripper coil voltage.

REPORTABLE EVENT

1.29 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 of 10 CFR Part 50.

SHUTDOWN MARGIN

1.30 SHUTDOWN MARGIN shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming all full-length rod cluster assemblies (shutdown and control) are fully inserted except for the single rod cluster assembly of highest reactivity worth which is assumed to be fully withdrawn.

This page intentionally left blank

INSTRUMENTATION

BASES

3/4.3.3.8 LOOSE-PART DETECTION SYSTEM

The OPERABILITY of the Loose-Part Detection System ensures that sufficient capability is available to detect loose metallic parts in the Reactor System and avoid or mitigate damage to Reactor System components. The allowable out-of-service times and surveillance requirements are consistent with the recommendations of Regulatory Guide 1.133, "Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors," May 1981.

3/4.3.3.9 DELETED

3/4.3.3.10 DELETED

3/4.3.4 TURBINE OVERSPEED PROTECTION

This specification is provided to ensure that the turbine overspeed protection instrumentation and the turbine speed control valves are OPERABLE and will protect the turbine from excessive overspeed. Protection from turbine excessive overspeed is required since excessive overspeed of the turbine could generate potentially damaging missiles which could impact and damage safety-related components, equipment, or structures.

3/4.11 DELETED

BASES

3/4.11.1 - DELETED

3/4.11.2 - DELETED

3/4/11/3 - DELETED

This page intentionally left blank

This page intentionally left blank

ADMINISTRATIVE CONTROLS

ANNUAL REPORTS (Continued)

duration of the specific activity above the steady-state level; and
(5) The time duration when the specific activity of the reactor coolant exceeded the radioiodine limit.

- c. Documentation of all challenges to the pressurizer power-operated relief valves (PORVs) and safety valves; and

ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT

-----NOTE-----

A single submittal may be made for a multiple unit station. The submittal shall combine sections common to all units at the station.

6.9.1.3 The Annual Radiological Environmental Operating Report covering the operation of the unit during the previous calendar year shall be submitted by May 1 of each year. The report shall include summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in the Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMDCM), and in 10 CFR Part 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C.

The Annual Radiological Environmental Operating Report shall include the results of analyses of all radiological environmental samples and of all environmental radiation measurements taken during the period pursuant to the locations specified in the table and figures in the REMDCM, as well as summarized and tabulated results of these analyses and measurements. In the event that some individual results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted in the next annual report.

RADIOACTIVE EFFLUENT RELEASE REPORT

-----NOTE-----

A single submittal may be made for a multiple unit station. The submittal shall combine sections common to all units at the station; however, for units with separate radwaste systems, the submittal shall specify the releases of radioactive material from each unit.

6.9.1.4 The Radioactive Effluent Release Report covering the operation of the unit in the previous year shall be submitted prior to May 1 of each year in accordance with 10 CFR 50.36a. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided shall be consistent with the objectives outlined in the REMDCM and in conformance with 10 CFR 50.36a and 10 CFR Part 50, Appendix I, Section IV.B.1.

ADMINISTRATIVE CONTROLS

MONTHLY OPERATING REPORTS

6.9.1.5 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the U.S. Nuclear Regulatory Commission, Document Control Desk, Washington, D.C. 20555, one copy to the Regional Administrator Region I, and one copy to the NRC Resident Inspector, no later than the 15th of each month following the calendar month covered by the report.

CORE OPERATING LIMITS REPORT

6.9.1.6.a Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle for the following:

1. Moderator Temperature Coefficient BOL and EOL limits and 300 ppm surveillance limit for Specification 3/4.1.1.3,

ADMINISTRATIVE CONTROLS

6.13 RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE CALCULATION MANUAL (REMODCM)

- a. The REMODCM shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm and trip setpoints, and in the conduct of the radiological environmental program; and
- b. The REMODCM shall also contain the radioactive effluent controls and radiological environmental monitoring activities and descriptions of the information that should be included in the Annual Radiological Environmental Operating, and Radioactive Effluent Release, reports required by Specification 6.9.1.3 and Specification 6.9.1.4.

Licensee initiated changes to the REMODCM:

- a. Shall be documented and records of reviews performed shall be retained. This documentation shall contain:
 - 1) sufficient information to support the change(s) together with the appropriate analyses or evaluations justifying the change(s), and
 - 2) a determination that the change(s) will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I of 10 CFR 50, and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations;
- b. Shall become effective after review and acceptance by SORC and the approval of the designated officer; and
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire REMODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the REMODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

6.14 RADIOACTIVE WASTE TREATMENT

Procedures for liquid and gaseous radioactive effluent discharges from the Unit shall be prepared, approved, maintained and adhered to for all operations involving offsite releases of radioactive effluents. These procedures shall specify the use of appropriate waste treatment systems utilizing the guidance provided in the REMODCM.

The Solid Radioactive Waste Treatment System shall be operated in accordance with the Process Control Program to process wet radioactive wastes to meet shipping and burial ground requirements.

ADMINISTRATIVE CONTROLS

6.15 RADIOACTIVE EFFLUENT CONTROLS PROGRAM

This program conforms to 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable. The program shall be contained in the REMODCM, shall be implemented by procedures, and shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- a. Limitations on the functional capability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the REMODCM;
- b. Limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas, conforming to 10 CFR 20, Appendix B, Table II, Column 2 (1993 version);
- c. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the REMODCM;
- d. Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released from each unit to unrestricted areas, conforming to 10 CFR 50, Appendix I;
- e. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the REMODCM at least every 31 days;
- f. Limitations on the functional capability and use of the liquid and gaseous effluent treatment systems to ensure that appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a period of 31 days would exceed 2% of the guidelines for the annual dose or dose commitment, conforming to 10 CFR 50, Appendix I;
- g. Limitations on the dose rate at any time resulting from radioactive material released in gaseous effluents to areas beyond the site boundary to a dose rate which, if the release were to occur for a full year, would cause a dose of 500 mrem. This conforms to the dose associated with the 1993 version of 10 CFR 20, Appendix B, Table II, Column I;
- h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;
- i. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and

ADMINISTRATIVE CONTROLS

- j. Limitations on the annual dose or dose commitment to any member of the public due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.

6.16 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

A program shall be provided to monitor the radiation and radionuclides in the environs of the plant. The program shall provided (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in the REMODCM, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

- a. Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the REMODCM.
- b. A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census, and
- c. Participation in a Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.