



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 29, 2000

Mr. H. L. Sumner, Jr.  
Vice President - Nuclear  
Hatch Project  
Southern Nuclear Operating  
Company, Inc.  
Post Office Box 1295  
Birmingham, Alabama 35201-1295

SUBJECT: EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2 RE: ISSUANCE OF  
AMENDMENTS (TAC NOS. MA9119 AND MA9120)

Dear Mr. Sumner:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 222 to Facility Operating License DPR-57 and Amendment No. 163 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated June 1, 2000.

The amendments revise the reactor vessel pressure and temperature limit curves that are in the Technical Specifications. In addition, an exemption from Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Appendix G was processed in conjunction with these amendments and issued separately.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Leonard N. Olshan, Senior Project Manager, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosures:

1. Amendment No. 222 to DPR-57
2. Amendment No. 163 to NPF-5
3. Safety Evaluation

cc w/encls: See next page

August 29, 2000

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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

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/RA/

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Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

DISTRIBUTION:

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Enclosures:

1. Amendment No. 222 to DPR-57

PDII-1 R/F

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2. Amendment No. 163 to NPF-5

RidsRgn2MailCenter

3. Safety Evaluation

WBeckner,TSB

cc w/encls: See next page

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Edwin I. Hatch Nuclear Plant

cc:

Mr. Ernest L. Blake, Jr.  
Shaw, Pittman, Potts  
and Trowbridge  
2300 N Street, NW.  
Washington, DC 20037

Mr. D. M. Crowe  
Manager, Licensing  
Southern Nuclear Operating  
Company, Inc.  
P. O. Box 1295  
Birmingham, Alabama 35201-1295

Resident Inspector  
Plant Hatch  
11030 Hatch Parkway N.  
Baxley, Georgia 31531

Mr. Charles H. Badger  
Office of Planning and Budget  
Room 610  
270 Washington Street, SW.  
Atlanta, Georgia 30334

Harold Reheis, Director  
Department of Natural Resources  
205 Butler Street, SE., Suite 1252  
Atlanta, Georgia 30334

Steven M. Jackson  
Senior Engineer - Power Supply  
Municipal Electric Authority  
of Georgia  
1470 Riveredge Parkway, NW  
Atlanta, Georgia 30328-4684

Charles A. Patrizia, Esquire  
Paul, Hastings, Janofsky & Walker  
10th Floor  
1299 Pennsylvania Avenue  
Washington, DC 20004-9500

Chairman  
Appling County Commissioners  
County Courthouse  
Baxley, Georgia 31513

Mr. J. D. Woodard  
Executive Vice President  
Southern Nuclear Operating  
Company, Inc.  
P. O. Box 1295  
Birmingham, Alabama 35201-1295

Mr. P. W. Wells  
General Manager, Edwin I. Hatch  
Nuclear Plant  
Southern Nuclear Operating  
Company, Inc.  
U.S. Highway 1 North  
P. O. Box 2010  
Baxley, Georgia 31515

Mr. L. M. Bergen  
Resident Manager  
Oglethorpe Power Corporation  
Edwin I. Hatch Nuclear Plant  
P. O. Box 2010  
Baxley, Georgia 31515



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 222  
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by Southern Nuclear Operating Company, Inc. (Southern Nuclear), acting for itself, Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated June 1, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

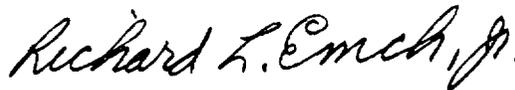
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No.222, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard L. Emch, Jr., Chief, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: August 29, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 222

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Insert

3.4-25

3.4-25

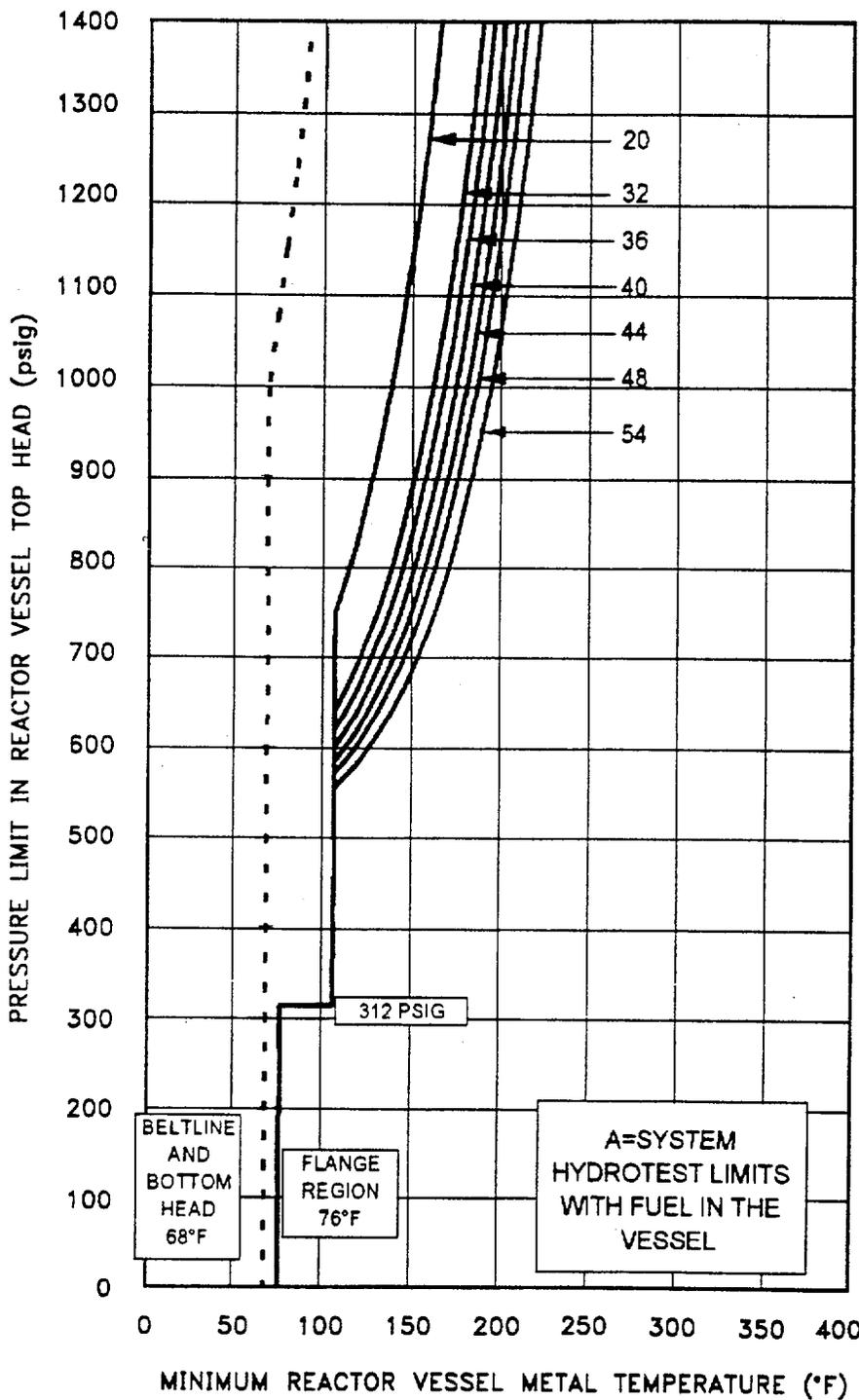
3.4-26

3.4-26

3.4-27

3.4-27

RCS P/T LIMITS  
3.4.9



INITIAL RTndt VALUES ARE  
-20°F FOR BELTLINE,  
40°F FOR UPPER VESSEL,  
AND  
10°F FOR BOTTOM HEAD

HEATUP/COOLDOWN  
RATE 20°F/HR

BELTLINE CURVES  
ADJUSTED AS SHOWN:  
EPPY SHIFT (°F)  
20 130.7

BELTLINE CURVES  
ADJUSTED AS SHOWN:  
EPPY SHIFT (°F)  
32 154.6

BELTLINE CURVES  
ADJUSTED AS SHOWN:  
EPPY SHIFT (°F)  
36 161.0

BELTLINE CURVES  
ADJUSTED AS SHOWN:  
EPPY SHIFT (°F)  
40 167.5

BELTLINE CURVES  
ADJUSTED AS SHOWN:  
EPPY SHIFT (°F)  
44 173.7

BELTLINE CURVES  
ADJUSTED AS SHOWN:  
EPPY SHIFT (°F)  
48 179.4

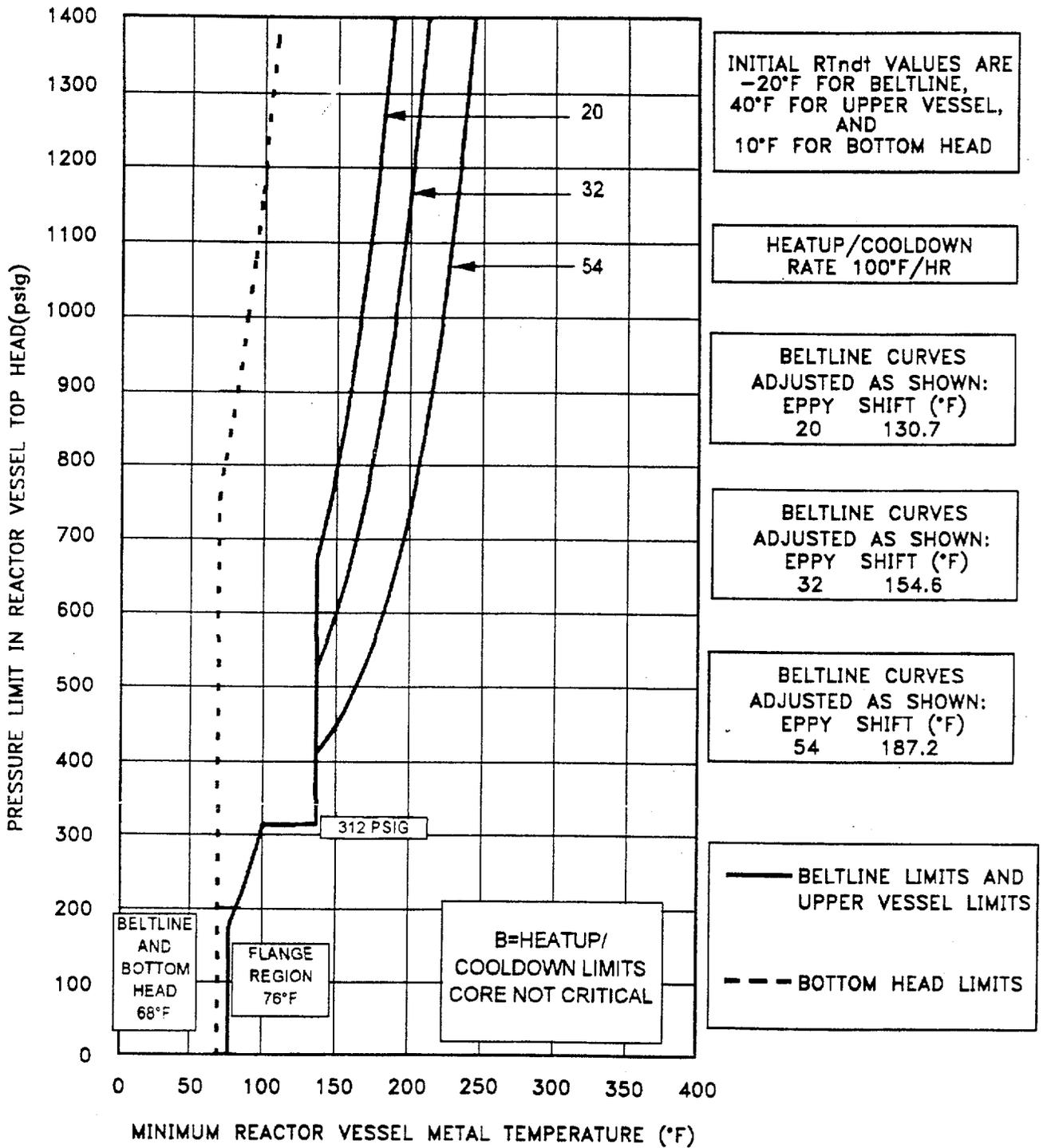
BELTLINE CURVES  
ADJUSTED AS SHOWN:  
EPPY SHIFT (°F)  
54 187.2

— BELTLINE LIMITS AND  
UPPER VESSEL LIMITS  
- - - BOTTOM HEAD LIMITS

ACAD F34911

Figure 3.4.9-1 (page 1 of 1)  
Pressure/Temperature Limits for  
Inservice Hydrostatic and Inservice Leakage Tests

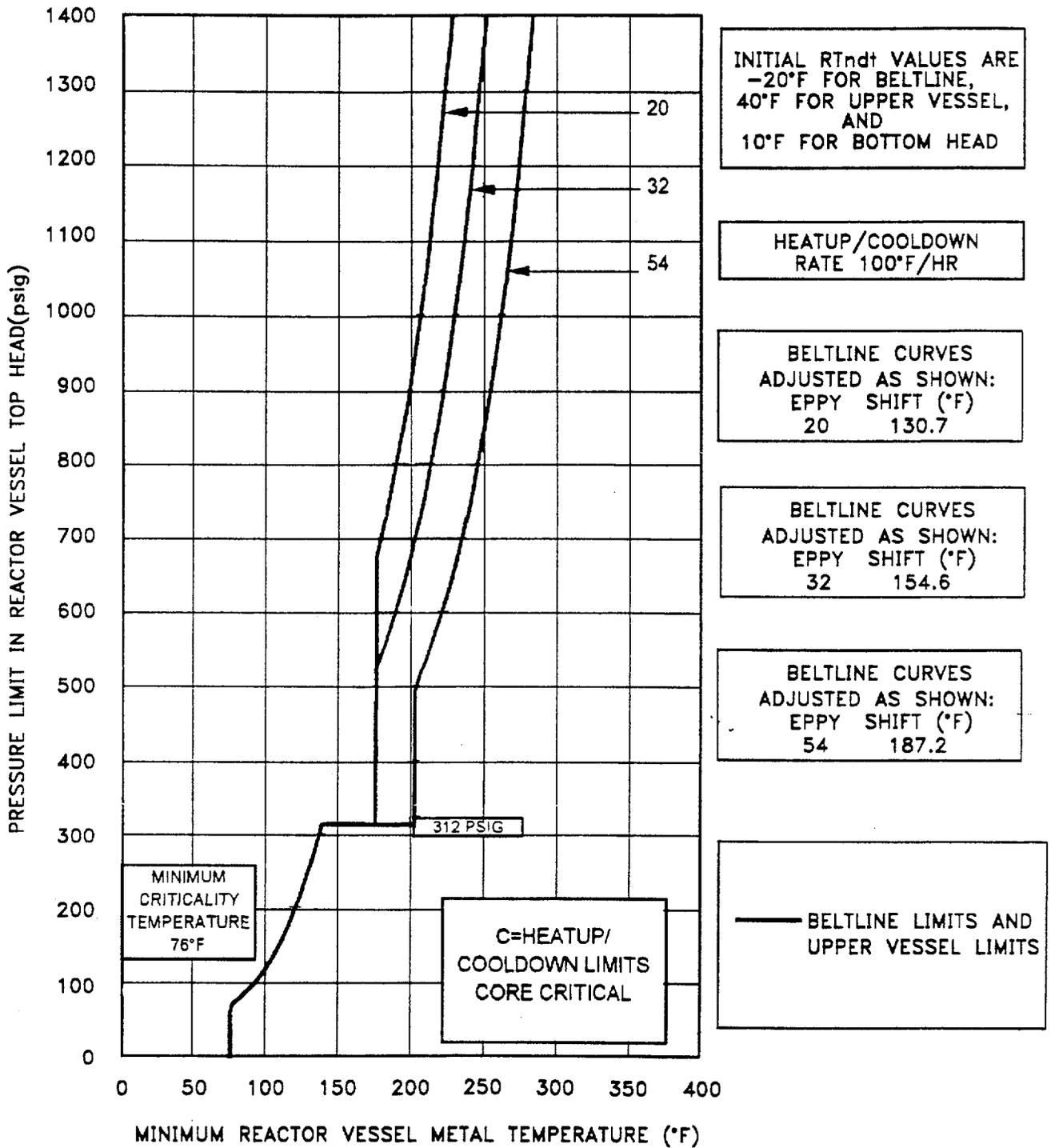
RCS P/T LIMITS  
3.4.9



ACAD F34921

Figure 3.4.9-2 (page 1 of 1)  
Pressure/Temperature Limits for Non-Nuclear Heatup,  
Low Power Physics Tests, and Cooldown Following a Shutdown

RCS P/T LIMITS  
3.4.9



ACAD F34931

Figure 3.4.9-3 (Page 1 of 1)  
Pressure/Temperature Limits for Criticality



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GEORGIA POWER COMPANY

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MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163  
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Southern Nuclear Operating Company, Inc. (Southern Nuclear), acting for itself, Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated June 1, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 163, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard L. Emch, Jr., Chief, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: August 29, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 163

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

Insert

3.4-25

3.4-25

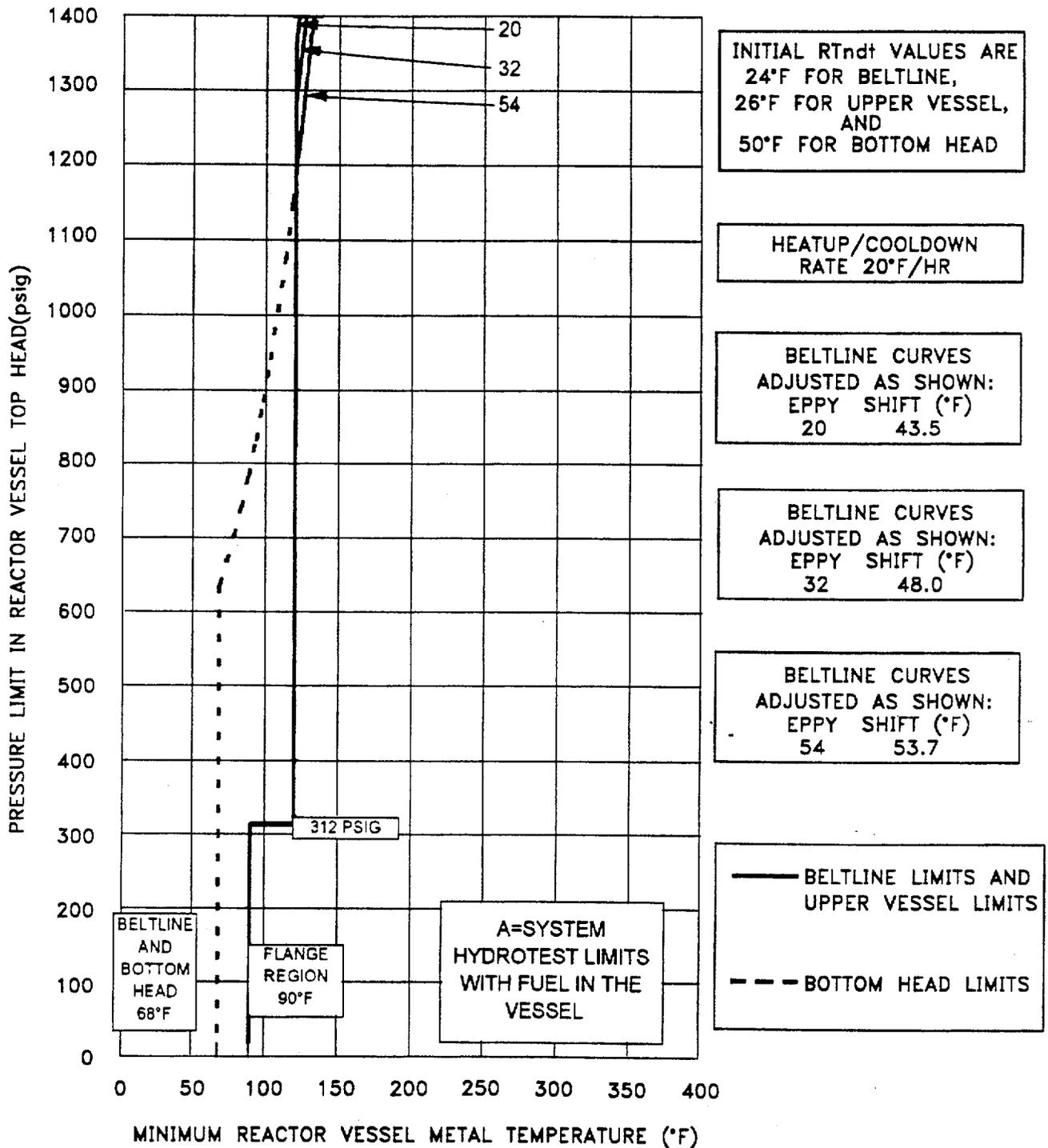
3.4-26

3.4-26

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3.4-27

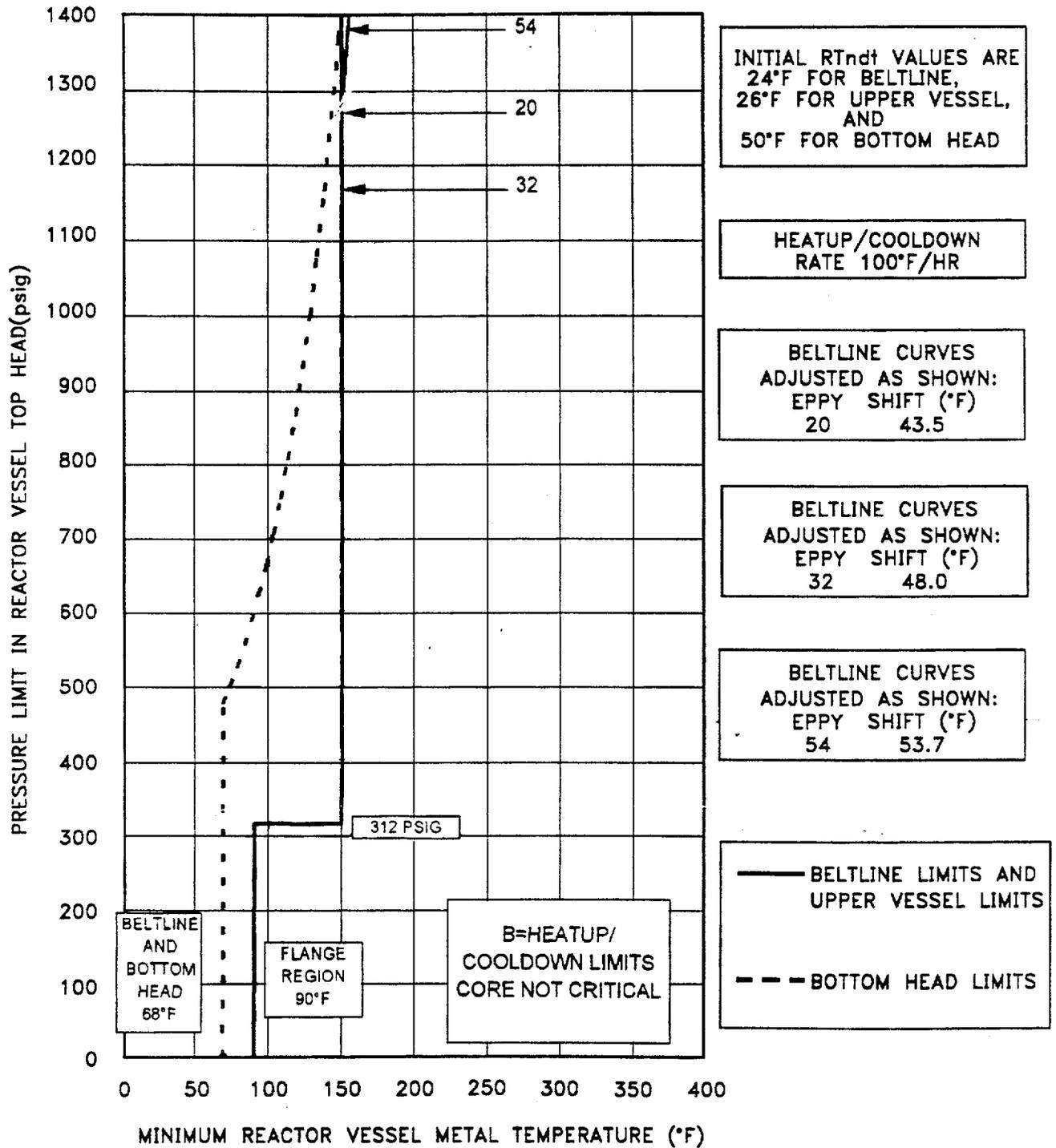
RCS P/T LIMITS  
3.4.9



ACAD F3491

Figure 3.4.9-1 (page 1 of 1)  
Pressure/Temperature Limits for  
Inservice Hydrostatic and Inservice Leakage Tests

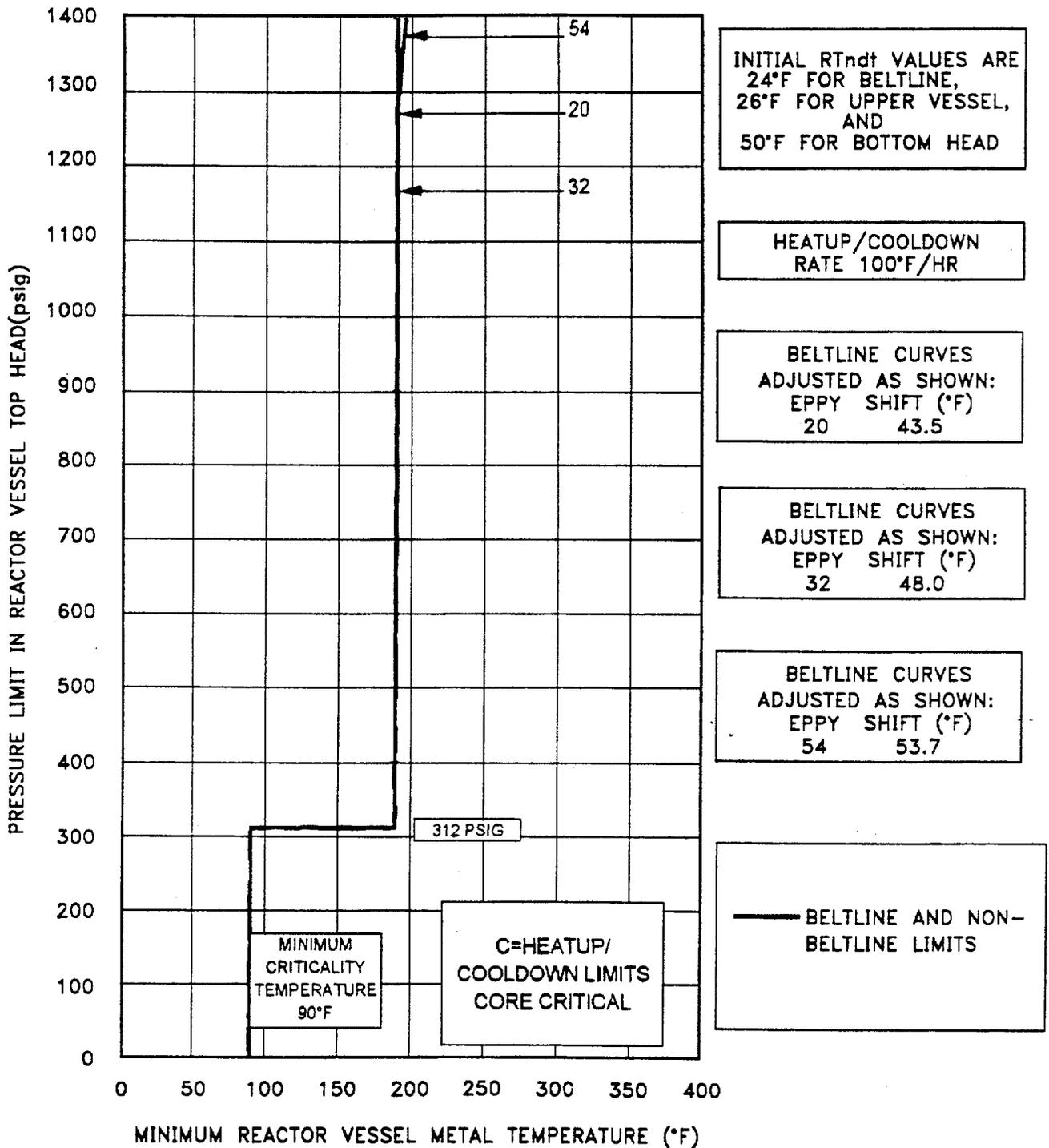
RCS P/T LIMITS  
3.4.9



ACAD F3492

Figure 3.4.9-2 (page 1 of 1)  
Pressure/Temperature Limits for Non-Nuclear Heatup,  
Low Power Physics Tests, and Cooldown Following a Shutdown

RCS P/T LIMITS  
3.4.9



ACAD F3493

Figure 3.4.9-3 (page 1 of 1)  
Pressure/Temperature Limits for Criticality



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 222 TO FACILITY OPERATING LICENSE DPR-57  
AND AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NPF-5  
SOUTHERN NUCLEAR OPERATING COMPANY, INC., ET AL.  
EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-321 AND 50-366

1.0 INTRODUCTION

By letter dated June 1, 2000, Southern Nuclear Operating Company, the licensee, submitted a Technical Specifications (TSs) change request to revise the pressure temperature (P-T) limits for a series of specified effective full power years (EFPYs) up to 54 EFPYs for Edwin I. Hatch Nuclear Plant, Units 1 and 2. The proposed methodology for P-T limits calculations is based on the 1989 American Society of Mechanical Engineers (ASME) Appendix G methodology with two modifications: first, the use of Code Cases N-588, which permits both the postulation of a circumferentially-oriented flaw in lieu of an axially-oriented flaw for the evaluation of reactor pressure vessel (RPV) circumferential welds and the use of the revised formula for stress intensity factors due to pressure and thermal gradient for axial flaws; and second, the use of Code Case N-640, which permits the use of the plane strain fracture toughness ( $K_{Ic}$ ) curve instead of the crack arrest fracture toughness ( $K_{Ia}$ ) curve for RPV materials in determining the P-T limits. As a result, the licensee also requested in this submittal an exemption from applying the current Appendix G methodology and to use the alternative methodology incorporating Code Cases N-588 and N-640.

The NRC has established requirements in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50 to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. The staff evaluates the P-T limit curves based on the following NRC regulations and guidance: 10 CFR Part 50, Appendix G; Generic Letter (GL) 88-11; GL 92-01, Revision 1; GL 92-01, Revision 1, Supplement 1; Regulatory Guide (RG) 1.99, Revision 2 (Rev. 2); and Standard Review Plan (SRP) Section 5.3.2. GL 88-11 advised licensees that the staff would use RG 1.99, Rev. 2 to review P-T limit curves. RG 1.99, Rev. 2 contains methodologies for determining the increase in transition temperature and the decrease in upper-shelf energy (USE) resulting from neutron radiation. GL 92-01, Rev. 1 requested that licensees submit their RPV data for their plants to the staff for review.

GL 92-01, Rev. 1, Supplement 1 requested that licensees provide and assess data from other licensees that could affect their RPV integrity evaluations. These data are used by the staff as the basis for the staff's review of P-T limit curves and as the basis for the staff's review of pressurized thermal shock (PTS) assessments (10 CFR 50.61 assessments). Appendix G to

10 CFR Part 50 requires that P-T limit curves for the RPV be at least as conservative as those obtained by applying the methodology of Appendix G to Section XI of the ASME Code.

SRP Section 5.3.2 provides an acceptable method of determining the P-T limit curves for ferritic materials in the beltline of the RPV based on the linear elastic fracture mechanics (LEFM) methodology of Appendix G to Section XI of the ASME Code. The basic parameter of this methodology is the stress intensity factor,  $K_I$ , which is a function of the stress state and flaw configuration. Appendix G requires a safety factor of 2.0 on stress intensities resulting from reactor pressure during normal and transient operating conditions and a safety factor of 1.5 for hydrostatic testing curves. The methods of Appendix G postulate the existence of a sharp surface flaw in the RPV that is normal to the direction of the maximum stress. This flaw is postulated to have a depth that is equal to 1/4 thickness (1/4T) of the RPV beltline thickness and a length equal to 1.5 times the RPV beltline thickness. The critical locations in the RPV beltline region for calculating heatup and cooldown P-T curves are the 1/4T and 3/4 thickness (3/4T) locations which correspond to the maximum depth of the postulated inside surface and outside surface defects, respectively.

The Appendix G ASME Code methodology requires that licensees determine the adjusted reference temperature (ART or adjusted  $RT_{NDT}$ ). ART is defined as the sum of the initial (unirradiated) reference temperature (initial  $RT_{NDT}$ ), the mean value of the adjustment in reference temperature caused by irradiation ( $\Delta RT_{NDT}$ ), and a margin (M) term.

$\Delta RT_{NDT}$  is a product of a chemistry factor and a fluence factor. The chemistry factor is dependent upon the amount of copper and nickel in the material and may be determined from tables in RG 1.99, Rev. 2 or from surveillance data. The fluence factor is dependent upon the neutron fluence at the maximum postulated flaw depth. The margin term is dependent upon whether the initial  $RT_{NDT}$  is a plant-specific or a generic value and whether the chemistry factor (CF) was determined using the tables in RG 1.99, Rev. 2 or surveillance data. The margin term is used to account for uncertainties in the values of the initial  $RT_{NDT}$ , the copper and nickel contents, the fluence and the calculational procedures. RG 1.99, Rev. 2 describes the methodology to be used in calculating the margin term.

## 2.0 EVALUATION

### 2.1 Licensee Evaluation

The licensee submitted detailed information for the ART of the limiting beltline material and P-T limit curves for a series of specified EFPYs up to 54 EFPYs for both Hatch units. For example, for 32 EFPYs for Unit 1, the licensee determined that the most limiting material for cooldown curves is the lower intermediate shell with heat no. C4114-2. The licensee employed the methodology in RG 1.99, Rev. 2 and calculated an ART of 134.6 °F at the 1/4T fluence of  $0.143E19$  n/cm<sup>2</sup> (32 EFPYs) for this limiting material based on a  $\Delta RT_{NDT}$  value of 120.6 °F, an initial  $RT_{NDT}$  of -20 °F, and a margin term of 34 °F ( $\sigma_I = 0^\circ\text{F}$  and  $\sigma_\Delta = 17^\circ\text{F}$ ). The  $\Delta RT_{NDT}$  value for this material was determined using surveillance data; however, the licensee elected not to use one-half the full margin allowed by the RG.

The licensee repeated the similar calculations for Unit 2 and found that ART for the most limiting material, the lower plate with heat no. C8553-1, is 72 °F at the 1/4T fluence of  $0.0991E19$  n/cm<sup>2</sup> (32 EFPYs) for the cooldown based on a  $\Delta RT_{NDT}$  value of 21.2 °F, an initial

$RT_{NDT}$  of 24 °F, and a margin term of 26.8 °F ( $\sigma_1 = 0$  °F and  $\sigma_\Delta = 13.4$  °F). The margin is determined by the  $\sigma_\Delta$  value which is limited to one half of the  $\Delta RT_{NDT}$  value according to the RG.

Based on the limiting ART of 134.6°F for Unit 1 and 72°F for Unit 2 for the cooldown, the licensee used the methodology of Appendix G in the 1989 Edition of Section XI of the ASME Code, as modified by Code Cases N-588 and N-640, to calculate the P-T limits for both units.

## 2.2 Staff Evaluation

The staff compared the licensee's material information in Tables 1 and 2 of Attachment 2 of the submittal with that in the NRC's reactor vessel integrity database (RVID) and determined that, except for the negligible difference in inner-diameter (ID) fluence for 32 EFPYs for both units, (0.197E19 n/cm<sup>2</sup> in the submittal compared to 0.194E19 n/cm<sup>2</sup> in the RVID for Unit 1, and 0.137E19 n/cm<sup>2</sup> in the submittal compared to 0.139E19 n/cm<sup>2</sup> in the RVID for Unit 2), the material data for the limiting beltline materials for both units is consistent with that in the RVID. The staff then performed an independent calculation of ART values for the limiting materials for both units using the methodology in RG 1.99, Revision 2 and verified the licensee's identification of the limiting materials and their ART values for the series of specified EFPYs up to 54 EFPYs for both units.

The licensee used the 1989 ASME Appendix G methodology, as modified by Code Cases N-588 and N-640, to generate the heatup and cooldown P-T limits for both Hatch units. Although the licensee did not use the primary function of Code Case N-588, it employed the new set of equations for calculating stress intensity factors for axial flaws. Since the 1989 ASME Appendix G methodology is more conservative than the Appendix G in the 1996 Addenda to the 1995 Edition of Section XI of the ASME Code, which is endorsed by the NRC, the licensee's methodology is acceptable. The staff performed calculations and confirmed the validity of the proposed P-T limits for beltline materials and for the bottom head and upper vessel. Although "HEATUP/COOLDOWN" appears in captions and titles of the proposed P-T curves, the tabulated P-T values in Attachment 2 are for cooldown curves. The staff performed calculations for the heatup curves and confirmed that the cooldown curves are bounding, and therefore, concluded that the proposed P-T curves are valid for both heatup and cooldown. The staff used the tabulated P and (T-RT<sub>NDT</sub>) results for the bottom head from the generic study reported in Attachment 3 of the submittal and adjusted the (T-RT<sub>NDT</sub>) values using the plane strain fracture toughness,  $K_{IC}$ , permitted by Code Case N-640. This exercise confirmed that the proposed P-T limits for the bottom head are consistent with those of the staff's. Hence, the staff determined that the licensee's proposed P-T limit curves meet the requirements of the ASME Code as modified by Code Cases N-588 and N-640.

In addition to beltline materials, Appendix G of 10 CFR Part 50 also imposes a minimum temperature at the closure head flange based on the reference temperature for the flange material. Section IV.A.2 of Appendix G states that when the pressure exceeds 20 percent of the preservice system hydrostatic test pressure, the temperature of the closure flange regions highly stressed by the bolt preload must exceed the reference temperature of the material in those regions by at least 120 °F for normal operation and by 90 °F for hydrostatic pressure tests and leak tests. When the pressure is less than or equal to 20 percent of the preservice system hydrostatic test pressure, the temperature of the closure flange regions highly stressed by the bolt preload must exceed the highest reference temperature of the material in the closure flange region that is highly stressed by the bolt for normal operation and for hydrostatic

pressure tests and leak tests. Based on the flange  $RT_{NDT}$  of 16 °F for Unit 1 and 30 °F for Unit 2, the staff has determined that the straight-line segments, 76 °F (16 °F+60 °F) line and 136 °F (16 °F+120 °F) line for Unit 1 and 90 °F (30 °F+60 °F) line and 150 °F (30 °F+120 °F) line for Unit 2, have satisfied the requirement for the closure flange region during normal operation and inservice leak and hydrostatic testing. The straight-line segment of 68 °F for the bottom head limits for both units is not required by Appendix G. It is a more conservative limit based on the water temperature of 68 °F assumed by the licensee in its calculation of the shutdown margin for the condition when the head is off while fuel is in the vessel.

### 2.3 RVID Updating

Since the discrepancies in ID fluence for both units are negligibly small, there is no need to update material information in the RVID program.

The staff concludes that the proposed P-T limits for the reactor coolant system for heatup, cooldown, hydrotest, and criticality satisfy the requirements in Appendix G to Section XI of the ASME Code, as modified by Code Cases N-588 and N-640, and Appendix G of 10 CFR Part 50 for a series of specified EFPYs up to 54 EFPYs for both Hatch units. The proposed P-T limits also satisfy GL 88-11, because the method in RG 1.99, Rev. 2 was used to calculate ART. Hence, the proposed P-T limit curves may be incorporated into the Hatch, Units 1 and 2 TSs.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (65 FR 39960). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by

operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Sheng /

Date: August 29, 2000