



RS-00-76

August 18, 2000

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Byron Station, Units 1 and 2
Facility Operating License Nos. NPF-37 and NPF-66
NRC Docket Nos. STN 50-454 and STN 50-455

Dresden Nuclear Power Station, Units 2 and 3
Facility Operating License Nos. DPR-19 and DPR-25
NRC Docket Nos. 50-237 and 50-249

LaSalle County Nuclear Power Station, Units 1 and 2
Facility Operating License Nos. NPF-11 and NPF-18
NRC Docket Nos. 50-373 and 50-374

Subject: Response to Request for Additional Information Concerning
Inservice Inspection Program Relief Requests Regarding Containment
Inspections by Approved Alternate Means

- References:
- (1) Letter from R.M. Krich (Commonwealth Edison Company) to U.S. NRC, "Request for Inservice Inspection Program Relief Regarding Containment Inspections by Approved Alternate Means," dated May 8, 2000
 - (2) Letter from George F. Dick, Jr. (U.S. NRC) to Commonwealth Edison Company, "Request for Additional Information Regarding Requests for Relief To Use the 1998 Edition of Subsections IWE and IWL of the ASME Code," dated August 11, 2000

In accordance with 10 CFR 50.55a, "Codes and standards," paragraph (a)(3)(i), Reference (1) was submitted as an alternative to the requirements of 10 CFR 50.55a(g), "Inservice inspection requirements," regarding the expedited examinations of containments for Byron Station, Units 1 and 2; Dresden Nuclear Power Station, Units 2 and 3; and LaSalle County Station, Units 1 and 2. On August 10, 2000, a teleconference was held with the NRC, its contractor and Commonwealth Edison

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(ComEd) Company personnel to discuss questions regarding the submittal. The NRC subsequently transmitted a Request for Additional Information (RAI) (i.e., Reference 2). This letter formally submits our response to the RAI.

Approval of our original submittal, as clarified in Attachment 1 of this response, will maintain the same level of assurance concerning the continued leak-tight and structural integrity of metallic and concrete containment components as would compliance with the American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," 1992 Edition with the 1992 Addenda.

We request that the relief requests be approved by September 8, 2000, to support the Fall 2000 refueling outages for Byron Station, Dresden Nuclear Power Station and LaSalle County Station.

Should you have any questions concerning this letter, please contact Ms. Marcia Lesniak at (630) 663-6484.

Respectfully,



R.M. Krich
Vice President – Regulatory Services

Attachment 1: Response to Request for Additional Information, Relief Request to Use the 1998 Edition of the American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section XI, Subsections IWE and IWL

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Byron Station
NRC Senior Resident Inspector – Dresden Nuclear Power Station
NRC Senior Resident Inspector – LaSalle County Station