Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Safeguards Material Control and Accountability **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The safeguards material control and accountability is the subsystem responsible for protecting and maintaining the inventories of nuclear material. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The safeguards material control and accountability is the subsystem responsible for protecting and maintaining the inventories of nuclear material. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The security and safeguards system - safeguards material control and accountability, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

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Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Badging & Records System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Badging & Records System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. Part of the purpose of this system is to provide for a method of detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Badging & Records System

Level 4: N/A

Level 3: N/A

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The security badging and record system maintains accurate personnel and visitor information, issues badges to support the automated surveillance, monitoring, and access control to all site area and facilities. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The security and safeguards system - security badging and record system, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: Material Access Area Barrier System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security barrier system - material access area barrier system controls access to areas where material is handled and stored. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security barrier system - material access area barrier system control assess to areas where material is stored. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: Material Access Area Barrier System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security barrier system - material access area barrier system control assess to areas where material is stored. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. Part of the purpose of this system is to provide detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: Material Access Area Barrier System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ Yes? Rationale:

The security barrier system - material access area barrier system controls access to areas where material is stored. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ Yes? Rationale:

The security and safeguards system - security barrier system - material access area barrier system, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: North Portal Support Area Barrier System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: North Portal Support Area Barrier System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ **Yes?** Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. Part of the purpose of this system is to provide detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: North Portal Support Area Barrier System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

The security barrier system - north portal support area barrier system controls access to areas where material is stored. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

The security barrier system - north portal support area barrier system, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4: N/A**

Level 3: Radiological Control Area Barrier System **Level 5: N/A**

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4: N/A**

Level 3: Radiological Control Area Barrier System **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where radiological material is handled and stored. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where material is stored. Part of the purpose of this system is to provide detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security barrier system - radiological control area barrier system controls access to areas where material is stored. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: Radiological Control Area Barrier System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

The security barrier system - radiological control area barrier system controls access to areas where material is stored. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

The security barrier system - radiological control area barrier system controls access to areas where material is stored. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

The security barrier system - radiological control area barrier system, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: Site Perimeter System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: Site Perimeter System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. Part of the purpose of this system is to provide detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Barrier System **Level 4:** N/A

Level 3: Site Perimeter System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The security barrier system - site perimeter system controls access to site and minimizes intrusion onto the site. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The security barrier system - site perimeter, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities **Level 4: N/A**

Level 3: Station 1 **Level 5: N/A**

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities **Level 4: N/A**

Level 3: Station 1 **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Part of the purpose of this system is to provide monitoring of and response to the detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities

Level 4: N/A

Level 3: Station 1

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

The security and safeguards system - security facilities are contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities **Level 4:** N/A

Level 3: Station 2 **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities **Level 4: N/A**

Level 3: Station 2 **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Part of the purpose of this system is to provide monitoring of and response to the detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities **Level 4: N/A**

Level 3: Station 2 **Level 5: N/A**

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ Yes? Rationale:

The security and safeguards system - security facilities are contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities

Level 4: N/A

Level 3: Station 3

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities **Level 4: N/A**

Level 3: Station 3 **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. Part of the purpose of this system is to provide monitoring of and response to the detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Facilities **Level 4:** N/A

Level 3: Station 3 **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The security facilities support the automated surveillance, badging, records, monitoring, and access control to all site area and facilities. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The security and safeguards system - security facilities are contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Automated Surveillance System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Automated Surveillance System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. Part of the purpose of this system is to provide detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Automated Surveillance System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The security surveillance system - automated surveillance system helps to detect the unauthorized access or movement of nuclear material, or contraband. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The security surveillance system - automated surveillance system, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Patrol System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ **Yes?** Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ **Yes?** Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ **Yes?** Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ **Yes?** Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Patrol System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. Part of the purpose of this system is to provide detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Patrol System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The security surveillance system - patrol system extends the defensive and surveillance capabilities to remote or inaccessible areas of the site which minimizes the likelihood of intrusion. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The security surveillance system - patrol system, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4: N/A**

Level 3: Survey Instrument System **Level 5: N/A**

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The security and safeguards system performs the surveillance and safeguards functions required to protect the repository from unauthorized intrusion, sabotage, theft, and the diversion of nuclear material. The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system performs no radiological safety function.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system is not required to function to prevent, mitigate, or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. Failure of this system will not result in a credible DBE which would otherwise result in a radioactive release above the federal limit.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system is not part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The security barrier system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. Failure of this system will not impact the natural or engineered barriers.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Survey Instrument System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system does not protect QA-1 or QA-2 SSCs from the effects of fire.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☑ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. Part of the purpose of this system is to provide detection and alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system is not involved in special nuclear material accountability.

Q-List Questions

SDD: SU48 - Security & Safeguards System

SSC: Security Surveillance Systems **Level 4:** N/A

Level 3: Survey Instrument System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The security surveillance system - survey instrument system provides instruments to detect the unauthorized movement of nuclear material or contraband. This system is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The security surveillance system - survey instrument system, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.15 Security Facilities, as QA-1.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Data Acquisition System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☐	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with the receipt, handling, storage, packaging, emplacement, or retrieval of high-level waste.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC may be required to function to mitigate or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. The direct failure of this SSC does not result in a credible DBE which would lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. The SSC does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. Direct failure of this SSC would not affect the waste isolation functions performed by the natural or engineered barriers.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Data Acquisition System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC does not perform a fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with the detection or alarming for unauthorized intrusion or the presence of explosive materials.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with special nuclear material accountability.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Data Acquisition System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not a permanently installed radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The Surface Environmental Monitoring System - Data Acquisition System, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.11 Monitoring and Operations Control Facilities, as QA-1.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Laboratory Facility System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☐	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with the receipt, handling, storage, packaging, emplacement, or retrieval of high-level waste.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC may be required to function to mitigate or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. The direct failure of this SSC does not result in a credible DBE which would lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. The SSC does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. Direct failure of this SSC would not affect the waste isolation functions performed by the natural or engineered barriers.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Laboratory Facility System

Level 4: N/A

Level 3: N/A

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous subetance releases into the environment. This SSC does not perform a fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous subetance releases into the environment. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with the detection or alarming for unauthorized intrusion or the presence of explosive materials.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous subetance releases into the environment. This SSC is not associated with special nuclear material accountability.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Laboratory Facility System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not a permanently installed radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The Surface Environmental Monitoring System - Laboratory Facility System, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.11 Monitoring and Operations Control Facilities, as QA-1.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Meteorological Monitoring System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☐	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This SSC is not associated with the receipt, handling, storage, packaging, emplacement, or retrieval of high-level waste.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This SSC may be required to function to mitigate or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. The direct failure of this SSC does not result in a credible DBE which would lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. The SSC does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. Direct failure of this SSC would not affect the waste isolation functions performed by the natural or engineered barriers.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Meteorological Monitoring System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This SSC is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This SSC does not perform a fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This SSC is not associated with the detection or alarming for unauthorized intrusion or the presence of explosive materials.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This SSC is not associated with special nuclear material accountability.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Meteorological Monitoring System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, monitors the area meteorology and alerts the site to adverse meteorological conditions. This SSC is not a permanently installed radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ Yes? Rationale:

The Surface Environmental Monitoring System - Meteorological Monitoring System, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.11 Monitoring and Operations Control Facilities, as QA-1.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Sample Collection System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☐	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with the receipt, handling, storage, packaging, emplacement, or retrieval of high-level waste.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC may be required to function to mitigate or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. The direct failure of this SSC does not result in a credible DBE which would lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground waters for radioactive and hazardous substance releases into the environment. The SSC does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. Direct failure of this SSC would not affect the waste isolation functions performed by the natural or engineered barriers.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Sample Collection System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC does not perform a fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with the detection or alarming for unauthorized intrusion or the presence of explosive materials.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not associated with special nuclear material accountability.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Sample Collection System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Sample Collection System, monitors the surface areas and ground water for radioactive and hazardous substance releases into the environment. This SSC is not a permanently installed radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The Surface Environmental Monitoring System - Sample Collection System, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.11 Monitoring and Operations Control Facilities, as QA-1.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Seismic Monitoring System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☐	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This SSC is not associated with the receipt, handling, storage, packaging, emplacement, or retrieval of high-level waste.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This SSC may be required to function to mitigate or monitor a credible DBE which would otherwise result in a radioactive release above the federal limit.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. The direct failure of this SSC does not result in a credible DBE which would lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. The SSC does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. Direct failure of this SSC would not affect the waste isolation functions performed by the natural or engineered barriers.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Seismic Monitoring System

Level 4: N/A

Level 3: N/A

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This SSC is not involved with the collection, containment, and/or monitoring of site-generated radioactive waste.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This SSC does not perform a fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. Failure of this system will not impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This SSC is not associated with the detection or alarming for unauthorized intrusion or the presence of explosive materials.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This SSC is not associated with special nuclear material accountability.

Q-List Questions

SDD: SU49 - Surface Environmental Monitoring System

SSC: Seismic Monitoring System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This system does not provide radiation shielding, reduction in dose rate, or have its own radioactive source term.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

The Surface Environmental Monitoring System - Seismic Monitoring System, monitors the area for seismic activity. This SSC is not a permanently installed radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

The Surface Environmental Monitoring System - Seismic Monitoring System, although not specifically addressed, is contained on the Q-List by direct inclusion for the Balance of Plant, SSA 3.2.3.11 Monitoring and Operations Control Facilities, as QA-1.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Administration Building **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Administration Building **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Administration Building **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Mock-Up Building **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Mock-Up Building **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** **Rationale:**

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** **Rationale:**

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** **Rationale:**

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** **Rationale:**

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** **Rationale:**

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Mock-Up Building **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Visitors Center **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4:** N/A

Level 3: Visitors Center **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration System Facilities **Level 4: N/A**

Level 3: Visitors Center **Level 5: N/A**

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** **Rationale:**

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Engineering System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ **Yes?** Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ **Yes?** Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ **Yes?** Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ **Yes?** Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Engineering System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** **Rationale:**

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** **Rationale:**

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** **Rationale:**

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthortzed explosive materials in the restricted area?

☐ **Yes?** **Rationale:**

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** **Rationale:**

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Engineering System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** **Rationale:**

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Office Services and Records System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Office Services and Records System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Office Services and Records System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ Yes? Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ Yes? Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ Yes? Rationale:

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Operations Management & Planning Computer System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Operations Management & Planning Computer System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Operations Management & Planning Computer System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems

Level 4: N/A

Level 3: Training System

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ **Yes?** Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ **Yes?** Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ **Yes?** Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ **Yes?** Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Training System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Training System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Transportation Dispatch Computer System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Administration System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Administration System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Administration System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Administration System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Administration System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Transportation Dispatch Computer System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Administration System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU50 - Administration System

SSC: Administration Systems **Level 4:** N/A

Level 3: Transportation Dispatch Computer System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** **Rationale:**

The Administration System is contained in Appendix B, Page B-2-3, of the Q-List, "Items excluded from the Q-List by exemption.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4:** N/A

Level 3: Central Shops **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4:** N/A

Level 3: Central Shops **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities

Level 4: N/A

Level 3: Central Shops

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4:** N/A

Level 3: Central Warehouse **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4:** N/A

Level 3: Central Warehouse **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4:** N/A

Level 3: Central Warehouse **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4:** N/A

Level 3: DC Receiving Shed **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4: N/A**

Level 3: DC Receiving Shed **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** **Rationale:**

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** **Rationale:**

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** **Rationale:**

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** **Rationale:**

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** **Rationale:**

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply System Facilities **Level 4:** N/A

Level 3: DC Receiving Shed **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Empty DC Supply System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Empty DC Supply System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Empty DC Supply System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Equipment Storage & Retrieval System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4: N/A**

Level 3: Equipment Storage & Retrieval System **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** **Rationale:**

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** **Rationale:**

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** **Rationale:**

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** **Rationale:**

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** **Rationale:**

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4: N/A**

Level 3: Equipment Storage & Retrieval System **Level 5: N/A**

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** **Rationale:**

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Inventory Planning & Management System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Inventory Planning & Management System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** **Rationale:**

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** **Rationale:**

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** **Rationale:**

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** **Rationale:**

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** **Rationale:**

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Inventory Planning & Management System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Maintenance Planning & Management System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Maintenance Planning & Management System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Maintenance Planning & Management System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Repair System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Maintenance & Supply System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Maintenance & Supply System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Maintenance & Supply System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Maintenance & Supply System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Maintenance & Supply System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4: N/A**

Level 3: Repair System **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Maintenance & Supply System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU51 - Maintenance & Supply System

SSC: Maintenance & Supply Systems **Level 4:** N/A

Level 3: Repair System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☐ **Yes?** Rationale:

The Maintenance & Supply System were determined to be a Non-Q item in Appendix B of the Q-list.

Q-List Questions

SDD: SU52 - Central Command & Control Operations System

SSC: Central Computer System

Level 4: N/A

Level 3: N/A

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☐	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ **Yes?** Rationale:

The Central Command & Control Operations System performs a monitoring function of the status of the repository operations and control the primary functions associated with critical and safety related equipment and may provide reasonable assurance of repository safe operations.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ **Yes?** Rationale:

There are no postulated design basis events that require the Central Command & Control Operations System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ **Yes?** Rationale:

There are no scenarios where direct failure of the Central Command & Control Operations System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ **Yes?** Rationale:

The Central Command & Control Operations System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

Direct failure of the Central Command & Control Operations System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU52 - Central Command & Control Operations System

SSC: Central Computer System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Central Command & Control Operations System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU52 - Central Command & Control Operations System

SSC: Central Computer System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The Central Command & Control Operations System is contained in the Q-List by Direct Inclusion for the balance of plant, SSA 3.2.3.11, Monitoring and Operations Control Facilities, as QA-1.

Q-List Questions

SDD: SU52 - Central Command & Control Operations System

SSC: Operator Station System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☐	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The Central Command & Control Operations System performs a monitoring function of the status of the repository operations and control the primary functions associated with critical and safety related equipment and may provide reasonable assurance of repository safe operations.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Central Command & Control Operations System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Central Command & Control Operations System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Central Command & Control Operations System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Central Command & Control Operations System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU52 - Central Command & Control Operations System

SSC: Operator Station System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Central Command & Control Operations System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU52 - Central Command & Control Operations System

SSC: Operator Station System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The Central Command & Control Operations System is contained in the Q-List by Direct Inclusion for the balance of plant, SSA 3.2.3.11, Monitoring and Operations Control Facilities, as QA-1.

Q-List Questions

SDD: SU53 - Offsite Utilities System

SSC: Offsite Utilities

Level 4: N/A

Level 3: N/A

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The Offsite Utilities System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the Offsite Utilities System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the Offsite Utilities System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The Offsite Utilities System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the Offsite Utilities System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU53 - Offsite Utilities System

SSC: Offsite Utilities **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Offsite Utilities System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU53 - Offsite Utilities System

SSC: Offsite Utilities

Level 4: N/A

Level 3: N/A

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ Yes? Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ Yes? Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ Yes? Rationale:

The Offsite Utilities System is contained in the Q-List by Direct Inclusion for the Surface Service and Utility Systems, SSA 3.1.2.2, as QA-1.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: Development Transportation System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The General Site Transportation System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the General Site Transportation System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the General Site Transportation System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The General Site Transportation System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the General Site Transportation System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: Development Transportation System **Level 4: N/A**

Level 3: N/A **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the General Site Transportation System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: Development Transportation System **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

The General Site Transportation System is contained in the Q-List by Direct Inclusion for the surface facilities, SSA 3.1.1.1, Transportation System, as QA-1.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4: N/A**

Level 3: Fuel Supply System **Level 5: N/A**

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☑	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The General Site Transportation System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the General Site Transportation System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the General Site Transportation System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The General Site Transportation System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the General Site Transportation System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4: N/A**

Level 3: Fuel Supply System **Level 5: N/A**

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☑ Yes? Rationale:

Failure of the Fuel Supply Transportation System as a result of a DBE (Fire) could impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: Fuel Supply System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

The General Site Transportation System is contained in the Q-List by Direct Inclusion for the surface facilities, SSA 3.1.1.1, Transportation System, as QA-1.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: General Rail System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☑	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The General Site Transportation System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the General Site Transportation System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the General Site Transportation System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The General Site Transportation System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the General Site Transportation System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: General Rail System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☑ Yes? Rationale:

Failure of the General Rail System as a result of a DBE could impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: General Rail System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The General Site Transportation System is contained in the Q-List by Direct Inclusion for the surface facilities, SSA 3.1.1.1, Transportation System, as QA-1.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4: N/A**

Level 3: General Road System **Level 5: N/A**

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The General Site Transportation System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the General Site Transportation System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the General Site Transportation System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The General Site Transportation System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the General Site Transportation System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: General Road System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the General Site Transportation System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: General Road System **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

The General Site Transportation System is contained in the Q-List by Direct Inclusion for the surface facilities, SSA 3.1.1.1, Transportation System, as QA-1.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: Vehicle Repair System **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The General Site Transportation System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the General Site Transportation System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the General Site Transportation System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The General Site Transportation System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the General Site Transportation System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4:** N/A

Level 3: Vehicle Repair System **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the Vehicle Repair System as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: General Site Transportation Systems **Level 4: N/A**

Level 3: Vehicle Repair System **Level 5: N/A**

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The General Site Transportation System is contained in the Q-List by Direct Inclusion for the surface facilities, SSA 3.1.1.1, Transportation System, as QA-1.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: Motor Pool & Facility Service Station **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☐	☐	☐	☐	☐	☐	☐	☑

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☐ Yes? Rationale:

The General Site Transportation System performs no radiological safety functions.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no postulated design basis events that require the General Site Transportation System to function.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☐ Yes? Rationale:

There are no scenarios where direct failure of the General Site Transportation System would result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☐ Yes? Rationale:

The General Site Transportation System does not perform a waste isolation function by forming part of the natural or engineered barriers.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

Direct failure of the General Site Transportation System will not affect the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: Motor Pool & Facility Service Station **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radiactive waste function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the Motor Pool & Facility Service Station as a result of a DBE will not impair QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform a special nuclear material accountability function.

Q-List Questions

SDD: SU54 - General Site Transportation System

SSC: Motor Pool & Facility Service Station **Level 4:** N/A

Level 3: N/A **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

This SSC has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

The General Site Transportation System is contained in the Q-List by Direct Inclusion for the surface facilities, SSA 3.1.1.1, Transportation System, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

Various important to radiological safety functions are performed by the basket within the waste package during packaging, storage, emplacement, and retrieval. These include; providing stability for the fuel assemblies inside the disposal container, transferring heat from the assemblies to the inner barrier, maintaining fuel geometry, and reducing criticality potential.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

This SSC will help mitigate several design basis events including those events which involve collision/crushing where the SNF assembly supports and basket will help maintain the integrity of the fuel assemblies. This SSC will also mitigate the effects of external events including seismic activity.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Direct failure of the basket could affect the fuel assembly geometry which could lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The Waste Package Basket is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The WP Basket performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The WP Basket performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package Basket has no function relating to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The Associated filler and criticality control materials maintain the waste package in a subcritical configuration during storage, emplacement and retrieval.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The associated filler and criticality control material mitigates design basis events which have the potential to reconfigure the SNF.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure of the associated filler and criticality control material could result in a release of radioactive material.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The associated filler and criticality control material is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials will be selected that will not impact the site characteristics.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The associated filler and criticality control material performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The associated filler and criticality control material performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the associated filler and criticality control material would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

The associated filler and criticality control material does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The associated filler and criticality control material does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel — **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) — **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The associated filler and criticality control material has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The associated filler and criticality control material is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ **Yes?** Rationale:

The inner barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ **Yes?** Rationale:

The inner barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ **Yes?** Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ **Yes?** Rationale:

The inner barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package inner barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Outer Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The outer barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The outer barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The outer barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Outer Barrier

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☑ Yes? Rationale:

This SSC provides identification of individual waste packages and their contents for a material accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container, with Absorber Plates, South Texas Fuel

Level 4: N/A

Level 3: Outer Barrier

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package outer barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The Associated filler and criticality control materials maintain the waste package in a subcritical configuration during storage, emplacement and retrieval.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The associated filler and criticality control material mitigates design basis events which have the potential to reconfigure the SNF.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure of the associated filler and criticality control material could result in a release of radioactive material.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The associated filler and criticality control material is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials will be selected that will not impact the site characteristics.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The associated filler and criticality control material performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The associated filler and criticality control material performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the associated filler and criticality control material would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

The associated filler and criticality control material does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The associated filler and criticality control material does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The associated filler and criticality control material has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The associated filler and criticality control material is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

Various important to radiological safety functions are performed by the basket within the waste package during packaging, storage, emplacement, and retrieval. These include; providing stability for the fuel assemblies inside the disposal container, transferring heat from the assemblies to the inner barrier, maintaining fuel geometry.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

This SSC will help mitigate several design basis events including those events which involve collision/crushing where the SNF assembly supports and basket will help maintain the integrity of the fuel assemblies. This SSC will also mitigate the effects of external events including seismic activity.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Direct failure of the basket could effect the fuel assembly geometry which could lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The Waste Package Basket is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The WP Basket performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The WP Basket performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package Basket has no function relating to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The inner barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The inner barrier are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The inner barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

The Waste package inner barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Outer Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The outer barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The outer barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The outer barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Outer Barrier **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☑ Yes? Rationale:

This SSC provides identification of individual waste packages and their contents for a material accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 12 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Outer Barrier **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

The Waste package outer barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The Associated filler and criticality control materials maintain the waste package in a subcritical configuration during storage, emplacement and retrieval.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The associated filler and criticality control material mitigates design basis events which have the potential to reconfigure the SNF.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure of the associated filler and criticality control material could result in a release of radioactive material.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The associated filler and criticality control material is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials will be selected that will not impact the site characteristics.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

The associated filler and criticality control material performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

The associated filler and criticality control material performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the associated filler and criticality control material would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

The associated filler and criticality control material does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

The associated filler and criticality control material does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The associated filler and criticality control material has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The associated filler and criticality control material is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

Various important to radiological safety functions are performed by the basket within the waste package during packaging, storage, emplacement, and retrieval. These include; providing stability for the fuel assemblies inside the disposal container, transferring heat from the assemblies to the inner barrier, maintaining fuel geometry, and reducing criticality potential.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

This SSC will help mitigate several design basis events including those events which involve collision/crushing where the SNF assembly supports and basket will help maintain the integrity of the fuel assemblies. This SSC will also mitigate the effects of external events including seismic activity.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Direct failure of the basket could effect the fuel assembly geometry which could lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The Waste Package Basket is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The WP Basket performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The WP Basket performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package Basket has no function relating to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ **Yes?** Rationale:

The inner barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ **Yes?** Rationale:

The inner barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ **Yes?** Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ **Yes?** Rationale:

The inner barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package inner barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Outer Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The outer barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The outer barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The outer barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates **Level 4:** N/A

Level 3: Outer Barrier **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☑ Yes? Rationale:

This SSC provides identification of individual waste packages and their contents for a material accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container, with Absorber Plates

Level 4: N/A

Level 3: Outer Barrier

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package outer barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The Associated filler and criticality control materials maintain the waste package in a subcritical configuration during storage, emplacement and retrieval.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The associated filler and criticality control material mitigates design basis events which have the potential to reconfigure the SNF.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure of the associated filler and criticality control material could result in a release of radioactive material.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The associated filler and criticality control material is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials will be selected that will not impact the site characteristics.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The associated filler and criticality control material performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The associated filler and criticality control material performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of the associated filler and criticality control material would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

The associated filler and criticality control material does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

The associated filler and criticality control material does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The associated filler and criticality control material has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The associated filler and criticality control material is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Basket **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ **Yes?** Rationale:

Various important to radiological safety functions are performed by the basket within the waste package during packaging, storage, emplacement, and retrieval. These include; providing stability for the fuel assemblies inside the disposal container, transferring heat from the assemblies to the inner barrier, maintaining fuel geometry.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ **Yes?** Rationale:

This SSC will help mitigate several design basis events including those events which involve collision/crushing where the SNF assembly supports and basket will help maintain the integrity of the fuel assemblies. This SSC will also mitigate the effects of external events including seismic activity.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ **Yes?** Rationale:

Direct failure of the basket could effect the fuel assembly geometry which could lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ **Yes?** Rationale:

The Waste Package Basket is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The WP Basket performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The WP Basket performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Basket **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package Basket has no function relating to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ **Yes?** Rationale:

The inner barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ **Yes?** Rationale:

The inner barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ **Yes?** Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ **Yes?** Rationale:

The inner barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Inner Barrier **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** **Rationale:**

The Waste package inner barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** **Rationale:**

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** **Rationale:**

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates

Level 4: N/A

Level 3: Outer Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☑	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The outer barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The outer barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The outer barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Outer Barrier **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☑ **Yes?** Rationale:

This SSC provides identification of individual waste packages and their contents for a material accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 21 Pressurized Water Reactor Disposal Container. No Absorber Plates **Level 4:** N/A

Level 3: Outer Barrier **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ Yes? Rationale:

The Waste package outer barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ Yes? Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ Yes? Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The Associated filler and criticality control materials maintain the waste package in a subcritical configuration during storage, emplacement and retrieval.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The associated filler and criticality control material mitigates design basis events which have the potential to reconfigure the SNF.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure of the associated filler and criticality control material could result in a release of radioactive material.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The associated filler and criticality control material is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials will be selected that will not impact the site characteristics.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ **Yes?** Rationale:

The associated filler and criticality control material performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ **Yes?** Rationale:

The associated filler and criticality control material performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ **Yes?** Rationale:

Failure of the associated filler and criticality control material would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety or waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ **Yes?** Rationale:

The associated filler and criticality control material does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ **Yes?** Rationale:

The associated filler and criticality control material does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates **Level 4:** N/A

Level 3: Associated Filler and Criticality Control Materials (if needed) **Level 5:** N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The associated filler and criticality control material has no functions related to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

The associated filler and criticality control material is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ **Yes?** Rationale:

Various important to radiological safety functions are performed by the basket within the waste package during packaging, storage, emplacement, and retrieval. These include; providing stability for the fuel assemblies inside the disposal container, transferring heat from the assemblies to the inner barrier, maintaining fuel geometry, and reducing criticality potential.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ **Yes?** Rationale:

This SSC will help mitigate several design basis events including those events which involve collision/crushing where the SNF assembly supports and basket will help maintain the integrity of the fuel assemblies. This SSC will also mitigate the effects of external events including seismic activity.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ **Yes?** Rationale:

Direct failure of the basket could effect the fuel assembly geometry which could lead to a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ **Yes?** Rationale:

The Waste Package Basket is part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ **Yes?** Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

The WP Basket performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

The WP Basket performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates

Level 4: N/A

Level 3: Basket

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package Basket has no function relating to minimizing radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-1	QA-2	QA-3	QA-4	QA-5	QA-6	QA-7	Non-Q
☑	☑	☐	☐	☐	☐	☐	☐

QA-1 - Important to Radiological Safety:

1.1 Is the SSC required to provide reasonable assurance that high-level waste can be received, handled, packaged, stored, emplaced, and retrieved without exceeding the federal limits?

☑ Yes? Rationale:

The inner barrier provides for containment of radioactive material contained in, or external to, the uncanistered SNF.

1.2 Is the SSC required to function to prevent, mitigate, or monitor a credible Design Basis Event which would otherwise result in a radioactive release above the federal limits?

☑ Yes? Rationale:

The inner barriers are required to mitigate design basis events since it must continue to perform its containment function at all times.

1.3 Will the direct failure of the SSC result in a credible Design Basis Event which would lead to a radioactive release above the federal limits?

☑ Yes? Rationale:

Failure to perform its containment function could result in a radioactive release above the federal limits.

QA-2 - Important to Waste Isolation:

2.1 Does the SSC perform a waste isolation function by forming part of the natural or engineered barriers?

☑ Yes? Rationale:

The inner barrier forms part of the engineered barrier.

2.2 Can direct failure of the SSC significantly affect the hydrological, geochemical, or geomechanical characteristics of the natural or engineered barriers which may prevent them from performing their waste isolation function?

☐ Yes? Rationale:

It is expected that materials used in this SSC will be selected such that they will not impact the characteristics of the natural or engineered barriers.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-3 - Important to Radioactive Waste Control:

3.1 Is the function of the SSC designed for collection, containment, and/or monitoring of site-generated radioactive waste?

☐ Yes? Rationale:

This SSC performs no site-generated radioactive waste control function.

QA-4 - Important to Fire Protection:

4.1 Does the SSC protect QA-1 or QA-2 SSCs from the effects of fire?

☐ Yes? Rationale:

This SSC performs no fire protection function.

QA-5 - Important to Potential Interaction:

5.1 As a result of a Design Basis Event, could failure of the SSC impair the capability of QA-1 or QA-2 SSCs from performing their radiological safety or waste isolation function?

☐ Yes? Rationale:

Failure of this SSC would not impact or impair a QA-1 or QA-2 SSC from performing its radiological safety of waste isolation function.

QA-6 - Important to Physical Protection of Facility and Materials:

6.1 Does the SSC's function provide detection or alarm of unauthorized intrusion or unauthorized explosive materials in the restricted area?

☐ Yes? Rationale:

This SSC does not perform a physical protection function.

6.2 Is the SSCs function required for special nuclear material accountability?

☐ Yes? Rationale:

This SSC does not perform an accountability function.

Q-List Questions

SDD: WP01 - Uncanistered SNF Disposal Containers

SSC: 24 Boiling Water Reactor Disposal Container, with Thick Absorber Plates

Level 4: N/A

Level 3: Inner Barrier

Level 5: N/A

QA-7 - Important to Occupational Radiological Exposure:

7.1 Does the SSC provide personnel radiation shielding, reduce dose rates in radioactive areas, or require personnel access into radiation areas by its own radioactive source term?

☐ **Yes?** Rationale:

The Waste package inner barrier has no function relating to minimizing personnel radiological exposure.

7.2 Is the SSC a permanently installed radiation monitor which monitors areas for personnel radiation protection?

☐ **Yes?** Rationale:

This SSC is not a radiation monitor.

Previous QA Classification:

This question is for historical and traceability purposes only. A "yes" answer to this question does not provide inclusion to the Q-List

8.0 Are there other factors, such as previous analyses, a body of consensus, or by direct inclusion, that led to the previous conclusion that this SSC is important to radiological safety (QA-1) or waste isolation (QA-2)?

☑ **Yes?** Rationale:

This SSC is contained on the Q-List by direct inclusion for the Waste Package, SSA 2.1 Spent Fuel, as QA-1.