

September 5, 2000

Mr. L. W. Myers
Senior Vice President
Beaver Valley Power Station
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY 1 AND 2 - REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING REVISIONS TO UPDATED FINAL SAFETY ANALYSIS REPORTS (UFSARs) DESIGN-BASIS DOSE ANALYSES (TAC NOS. MA9059 AND MA9060)

Dear Mr. Myers:

By letter dated May 12, 2000, FirstEnergy Nuclear Operating Company submitted License Amendment Request Nos. 280 and 151 seeking approval of proposed changes to the Beaver Valley Power Station, Units 1 and 2 (BVPS-1 & 2) UFSARs.

The Nuclear Regulatory Commission (NRC) staff has been reviewing your submittal. However, we have determined that additional information is required to complete our review. The required information was discussed with Messrs. J. Maracek, J. Lebda, et al. of your staff during a July 28, 2000, conference call. The NRC requests that you provide your response to the enclosed RAI within 60 days of receipt of this letter. This was discussed with Mr. Maracek and was established as a mutually agreeable timetable for your response. If circumstances result in the need to revise the target date, please call me at the earliest opportunity.

Should you have any questions regarding this request, please contact me at (301) 415-1427.

Sincerely,

/RA/

Daniel S. Collins, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosure: RAI

cc w/encl: See next page

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*See previous concurrence

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING REVISIONS TO UPDATED FINAL SAFETY ANALYSIS REPORT (UFSAR)

DESIGN-BASIS DOSE ANALYSES

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-334 AND 50-412

The Nuclear Regulatory Commission staff has been reviewing FirstEnergy Nuclear Operating Company's submittal, License Amendment Request Nos. 280 and 151, dated May 12, 2000, requesting approval of proposed changes to the Beaver Valley Power Station (BVPS), Units 1 and 2, UFSARs. The proposed revisions to the UFSARs modify information on design-basis accident radiological doses as a result of the recent complete re-evaluation of all BVPS dose calculations. Based on our review, the following information is required to complete our review.

Unit 1:

- 1) How is the 310 cubic-feet-per-minute (cfm) unfiltered inleakage to the control room during the 20-minute fan start delay determined and verified?
- 2) What is the basis for the control room pressurization fan flow rate being higher in the main steam line break (MSLB) than that assumed for the loss-of-coolant accident (LOCA) and reactor coolant pump (RCP) locked rotor accident?
- 3) For the LOCA Analysis
 - a) What is the basis for the assumed sump water volume increase?
 - b) What is the basis for the reactor water storage tank (RWST) leakage starting later and having a longer duration leakage than previously assumed and documented in the UFSAR?
 - c) Is the RWST control room X/Q the same as assumed previously, and has the staff previously reviewed and approved a submittal that uses this X/Q value?
 - d) The locked rotor accident and MSLB assume 310 cfm unfiltered inleakage to the control room during the 20-minute fan start delay. Why is it not assumed for the LOCA analysis also?
 - e) In Table 2.2-12, there are added control room X/Qs, "Containment edge to service building," and "Containment top to service building," which the note says were effective August 1999. Did the staff review and approve a submittal using these new values?

Unit 2:

- 1) Why does Table 15.0-10a show an increase in letdown flow rate?
- 2) What is the value assumed for reactor coolant system fluid mass for each of the following accident analyses: MSLB, RCP locked rotor, and rod cluster control assembly (RCCA) ejection?
- 3) For the Control Room Habitability Analysis, please provide the ventilation parameters used for each of the following accident analyses: MSLB, RCP locked rotor, RCCA Ejection, steam generator tube rupture, and LOCA.
- 4) For the LOCA analysis, please provide the basis for the following:
 - a) The reduction in the assumed containment free volume.
 - b) The change in the sprayed fraction of containment volume.
 - c) The increase in the particulate iodine removal coefficient.
 - d) The change in emergency core cooling system leakage initiation time.
 - e) The increase in the assumed sump water volume.
 - f) The change in RWST release initiation time.

Beaver Valley Power Station, Units 1 and 2

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