



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

**REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931**

August 8, 2000

Duke Energy Corporation
ATTN: Mr. W. R. McCollum
Vice President
Oconee Nuclear Station
7800 Rochester Highway
Seneca, SC 29672

**SUBJECT: SAFETY SYSTEM DESIGN AND PERFORMANCE CAPABILITY INSPECTION
NRC INSPECTION REPORT NOS. 50-269/00-12, 50-270/00-12, 50-287/00-12**

Dear Mr. McCollum:

The purpose of this letter is to request information in preparation for the subject inspection, which is scheduled to be performed at your Oconee facility on October 16 - 20 and October 30 - November 3, 2000. The information requested is listed in the enclosure to this letter. It will be needed by October 2, 2000, to support our inspection preparation effort.

The primary focus of the inspection will be the capability of the standby shutdown facility and the station auxiliary service water pump to perform their design functions for event mitigation. The inspection team will include five inspectors lead by Mr. Robert Schin. He has discussed the inspection plans and information being requested with Ms. Judy Smith of your staff. Mr. Schin will visit the Oconee facility on October 2 - 3 to review and collect the requested information.

In an effort to minimize your administrative burden, the team plans to use information already available in the NRC Region II offices and plans to verify, during the October 2 - 3 site visit, that the information is up to date. Also, in an effort to minimize your administrative burden, the information requested includes several lists from which the team plans to select individual items for review onsite.

During the information gathering visit, the lead inspector will also discuss the following inspection support administrative details: office space size and location; specific documents requested to be made available to the team in or near their office spaces; arrangements for inspector site access badging as needed; scheduling the inspection entrance and exit meetings; and availability of knowledgeable plant licensing, engineering, operations, maintenance, and risk analysis personnel to serve as points of contact during the inspection.

DEC

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Thank you for your cooperation in this matter. If you have any questions regarding this inspection, please contact Mr. Schin at (404) 562-4629 or me at (404) 562-4605.

Sincerely,

/RA/

Kerry D. Landis, Chief
Engineering Branch
Division of Reactor Safety

Docket Nos. 50-269, 50-270, 50-287
License Nos. DPR-38, DPR-47, DPR-55

Enclosure: Information Requested for Safety System Design and Performance Capability
Inspection

cc w/encl:
Compliance Manager (ONS)
Duke Energy Corporation
Electronic Mail Distribution

Lisa Vaughn
Legal Department (PB05E)
Duke Energy Corporation
422 South Church Street
Charlotte, NC 28242

Rick N. Edwards
Framatome Technologies
Electronic Mail Distribution

Anne Cottingham
Winston and Strawn
Electronic Mail Distribution

Mel Fry, Director
Division of Radiation Protection
N. C. Department of Environmental
Health & Natural Resources
Electronic Mail Distribution

(cc w/encl cont'd - See page 3)

(cc w/encl cont'd)
 Henry J. Porter, Assistant Director
 Div. of Radioactive Waste Mgmt.
 S. C. Department of Health and
 Environmental Control
 Electronic Mail Distribution

R. Mike Gandy
 Division of Radioactive Waste Mgmt.
 S. C. Department of Health and
 Environmental Control
 Electronic Mail Distribution

County Supervisor of
 Oconee County
 415 S. Pine Street
 Walhalla, SC 29691-2145

Lyle Graber, LIS
 NUS Corporation
 Electronic Mail Distribution

L. A. Keller, Manager
 Nuclear Regulatory Licensing
 Duke Energy Corporation
 526 S. Church Street
 Charlotte, NC 28201-0006

Peggy Force
 Assistant Attorney General
 N. C. Department of Justice
 Electronic Mail Distribution

Distribution w/encl:
 D. LaBarge, NRR
 PUBLIC

OFFICE	RII:DRS	RII:DRP					
SIGNATURE	RSCHIN 8/8/00	COGLE 8/8/00					
NAME	RSCHIN	COGLE					
DATE	8/ /2000	8/ /2000	8/ /2000	8/ /2000	8/ /2000	8/ /2000	8/ /2000
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

INFORMATION REQUESTED
FOR SAFETY SYSTEM DESIGN AND PERFORMANCE CAPABILITY INSPECTION

This inspection will focus on the standby shutdown facility (SSF), station auxiliary service water (ASW) pump, and related support equipment. This equipment includes, but may not be limited to: the SSF diesel generator, batteries, electrical switchgear and cables, control circuits, starting air, fuel oil, lubricating oil, cooling water, and ventilation systems; the SSF reactor coolant makeup pumps, associated piping, and water sources; the SSF ASW pump, associated piping, and water sources; the SSF portable pumping system; the SSF building; the station ASW pump, associated piping, water sources, and power supplies; and the main steam atmospheric dump valves. For these systems and equipment, the following information is requested:

- Design criteria (i.e., design basis documents). Also, provide a description of any current or planned safety functions that are not included in the design basis documents. In addition, provide a list of any operator actions relied on within the first 30 minutes of an event that are not included in the design basis documents.
- Any approved changes to the Updated Final Safety Analysis Report (UFSAR) which have not yet been docketed.
- Any approved changes to the Selected Licensee Commitments (SLC) which have not yet been docketed.
- Piping and instrumentation drawings, one-line diagrams, and logic diagrams. Also, electrical elementary drawings of control circuits (not wiring installation drawings) for the following equipment: SSF EDG; SSF EDG output breaker; SSF RC makeup pump; SSF ASW pump; station ASW pump; and motor operated valves 1SF-82, 1SF-97, 1HP-398, 1CCW-268, and 1CCW-269.
- Procedures for normal operation, annunciator response, abnormal operation, and emergency operation.
- Self-assessments or audits performed in the last 24 months.
- Operator training lesson plans, system descriptions, and Job Performance Measures (JPMs).
- System Health Reports and Performance Trends.
- Maintenance Rule Performance Criteria.
- A list of calculations and tests that support the design basis capabilities.
- A list of periodic surveillance/test/inspection/maintenance activities and frequencies (exclusive of those listed above) used to monitor, demonstrate, or maintain the continuing capabilities of the systems and components. (Please identify any applicable TS or SLC number beside each activity.)

ENCLOSURE

- A list of plant modifications implemented since original installation of the equipment.
- A list of currently open temporary modifications and operator work arounds.
- A list of Problem Identification Process (PIP) reports and any other deficiency reports initiated since 1995.
- A list of corrective maintenance activities and maintenance preventable functional failures since 1995.
- A list of item equivalency evaluations and commercial grade dedications since 1995.
- A list of operating experience feedback reports since 1995 (e.g., P21, IN, BL, SOER).
- A list of applicable drawings.