

James A. FitzPatrick
Nuclear Power Plant
268 Lake Road
P.O. Box 41
Lycoming, New York 13093
315-342-3840



Michael J. Colomb
Site Executive Officer

July 21, 2000
JAAP-00-0164

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

SUBJECT: **James A. FitzPatrick Nuclear Power Plant**
Docket No. 50-333
Reporting of Changes and Errors in ECCS Evaluation Models

ATTACHMENT: Letter from Ala F. Alzaben (Global Nuclear Fuel) to Paul Lemberg (NYPA), AFA-00-N031 dated July 05, 2000, Summary of Changes and Errors in ECCS Evaluation Models.

Dear Sir:

Attachment 1 of this letter reports changes and errors in Emergency Core Cooling System (ECCS) evaluation models in accordance with 10 CFR 50.46(a)(3)(iii) for the period from July 01, 1999 to June 30, 2000. There have been no changes or errors applicable to the James A. FitzPatrick Nuclear Power Plant. It is the Authority's understanding that this report has been previously provided to the NRC directly by Global Nuclear Fuel.

There are no commitments contained in this submittal.

Questions concerning this report may be addressed to Mr. George Tasick at (315) 349-6572.

Very truly yours,



Michael J. Colomb

MJC:GB:las
Enclosure

A001

cc:

Regional Administrator
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Office of the Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 136
Lycoming, NY 13093

United States Nuclear Regulatory Commission
Mr. Guy Vissing
Project Directorate 1
Mail Stop OWFN 8C2
Washington, DC 20555



Global Nuclear Fuel

Ala F. Alzaben
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AFA-00-N031
July 5, 2000

Mr. Paul Lemberg
Manager of Nuclear Fuel Supply
New York Power Authority
123 Main Street, 11-F
White Plains, NY 10601

cc: G. S. Grochowski
M. Karasulu
G. L. Rorke
R. D. Nourse

Subject: **Summary of Changes and Errors in ECCS Evaluation Models**

Dear Mr. Lemberg:

Enclosed is a copy of the annual letter recently sent to the NRC summarizing the impact of changes and errors in the methodology used by GE/GNF-A to demonstrate compliance with the Emergency Core Cooling System (ECCS) requirements of 10CFR50.46.

This letter is for information only.

Plant specific errors (e.g., input errors) are not reported to the NRC in the annual letter. These errors are separately communicated to affected utilities and are now incorporated into the cycle specific Supplemental Reload Licensing Report.

The enclosed notification letter to the NRC does not satisfy the licensee reporting requirements per 10 CFR 50.46(a)(3)(ii), however, per Reference 1, licensees may reference this notification in their annual report.

If you have any question please give me a call.

Sincerely,

Ala F. Alzaben
Fuel Project Manager
(910) 675-6232
Enclosure



Global Nuclear Fuel

Glen A. Watford
Manager, Nuclear Fuel Engineering

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June 30, 2000

FLN-2000-06

Document Control Desk
US Nuclear Regulatory Commission
Washington, DC 20555-0001

Attention: J. L. Wermiel

Subject: **Summary of Changes and Errors in ECCS Evaluation Models**

Reference: Letter, G. A. Watford to the Document Control Desk (J. L. Wermiel), *Reporting of Changes and Errors in ECCS Evaluation Models*, dated June 30, 1999 (MFN-004-99).

The purpose of this letter is to summarize the impact of changes and errors in the methodology used by GE/GNF-A to demonstrate compliance with the Emergency Core Cooling System (ECCS) requirements of 10 CFR 50.46. This report covers the period from the last report (Reference) to the present. It is noted that Peak Cladding Temperature (PCT) variations resulting from system or fuel changes are not addressed in this letter. These should be treated, as appropriate, on a plant specific basis in accordance with other sections of 10CFR50.

A summary of the changes and errors is provided in the attached table. The table describes the approved methodology affected, the range of applicability of the change/error, a brief description of the change/error and the estimated impact

All utilities using these evaluation models have been notified of these changes/errors to assist them in reporting, in accordance with 10CFR50.46 (a) (3) (ii).

If you have any questions, please call me at (910) 675-5446.

Sincerely,

Glen A. Watford, Manager
Nuclear Fuel Engineering

**Summary of Changes and Errors in ECCS Evaluation Models
July 1999 through June 2000**

Error/ Change	Approved Methodology	Applicability	Description	Impact
Error	NEDC-32950P, Compilation of Improvements to GENE's SAFER ECCS-LOCA Evaluation Model," January 2000.	BWR/6 plants	The reactor pressure vessel thermal response is simulated in the SAFER code as several heat slabs for which the one-dimensional radial conduction equation is solved (Reference). A logic error was discovered in an automated SAFER/GESTR basedeck generation procedure that calculated the heat transfer areas for the vessel heat slabs. As a result of this logic error, the heat transfer areas for the vessel heat slabs in the downcomer region were incorrectly specified for BWR/6 plants. This error affects the steam generation in the vessel during the reflooding stage of the event once the lower plenum fills and water spilling over from the jet pumps comes into contact with the vessel wall in the downcomer region.	0 to -45°F