

July 27, 2000

Note to: File Center

From: K. Cotton */RA/*

Revised technical specification pages for MA7366 for Virgil C. Summer
Nuclear Power Plant.

From: "PROPER, JAMES R" <JPROPER@scana.com>
To: "krc@nrc.gov" <krc@nrc.gov>
Date: Tue, Jul 25, 2000 3:05 PM
Subject: FW: New Pages for IWE/IWL TS Change

Mel,

I agreed to add a paragraph to the bases section which is the same as the words we agreed to about reportability (in the action section). The NRC reviewer asked that we do this for consistency sake.

> -----Original Message-----

> **From:** RAILEY, DONNA W
> **Sent:** Tuesday, July 25, 2000 2:29 PM
> **To:** PROPER, JAMES R
> **Subject:** New Pages for IWE/IWL TS Change

>

> <<Insert B.doc>> <<B3 6-2a.doc>>

CC: "BROWNE, MELVIN N" <MBROWNE@scana.com>, "JONES, DO..."

Insert B (B 3/4.6.1.6)

The reactor building structural integrity limitations as described in the Containment Inservice Inspection Program (CISIP) ensure that the structural integrity of the containment will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the containment will withstand the maximum pressure of 53.5 psig in the event of a steam line break accident. The measurement of containment tendon lift off force, the tensile tests of the tendon wires, the visual examination of tendons, anchorages and exposed interior and exterior surfaces of the containment, and the Type A leakage test are sufficient to demonstrate this capability. Visual and other required examinations of tendons, anchorages, and surfaces are performed periodically in accordance with plant procedures. These procedures embody applicable requirements of the 1992 Addenda of ASME Code, Section XI, Subsection IWL as set forth in 10CFR50.55a. Any degradations exceeding the CISIP acceptance criteria will be reviewed under an engineering evaluation within 60 days of the completion of the inspection to determine what impact the degradation has on overall containment operability, if any.

In addition, any significant degradation which seriously challenges containment operability found during the inspection shall be reported to the NRC in accordance with Technical Specification 6.9.2 within 30 days. The report shall include the description of degradation, operability determination, root cause determination, and corrective actions taken.

The tendon lift-off forces are evaluated to ensure that 1) the rate of tendon force loss is within predicted limits, and 2) a minimum required tendon force level exists in the containment. In order to assess the rate of force loss, the average lift off force for a tendon is compared with 95% of the predicted force. The predicted force is calculated by subtracting the initial, time-dependent, and other losses where applicable from the original stressing force, consistent with the recommendations of Regulatory Guide 1.35.1, Revision 3 dated July 1990.

CONTAINMENT SYSTEMS

BASES

CONTAINMENT STRUCTURAL INTEGRITY (Continued)

In addition, any significant degradation which seriously challenges containment operability found during the inspection shall be reported to the NRC in accordance with Technical Specification 6.9.2 within 30 days. The report shall include the description of degradation, operability determination, root cause determination, and corrective actions taken.

The tendon lift-off forces are evaluated to ensure that 1) the rate of tendon force loss is within predicted limits, and 2) a minimum required tendon force level exists in the containment. In order to assess the rate of force loss, the average lift off force for a tendon is compared with 95% of the predicted force. The predicted force is calculated by subtracting the initial, time-dependent, and other losses where applicable from the original stressing force, consistent with the recommendations of Regulatory Guide 1.35.1, Revision 3 dated July 1990.

From: "BROWNE, MELVIN N" <MBROWNE@scana.com>
To: "Karen Cotton (E-mail)" <krc@nrc.gov>
Date: Thu, Jul 20, 2000 5:17 PM
Subject: FW: Latest Revision to Page 6-12c

Karen,
Please find the revised page for containment tendons attached.
Mel

-----Original Message-----

From: RAILEY, DONNA W
Sent: Thursday, July 20, 2000 3:05 PM
To: BROWNE, MELVIN N
Subject: Latest Revision to Page 6-12c

<<6-12c.doc>>

g. Containment Leakage Rate Testing Program (Continued)

2) Air lock testing acceptance criteria are:

- a. Overall air lock leakage rate is $\leq 0.10 L_a$ when tested at $\geq P_a$.
- b. For each door, leakage rate is $\leq 0.01 L_a$ when pressurized to ≥ 8.0 psig for at least 3 minutes.

The provisions of Specification 4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of Specification 4.0.3 are applicable to the Containment Leakage Rate Testing Program.

h. Containment Inservice Inspection Program

This program provides controls for monitoring containment vessel structural integrity including routine inspections and tests to identify degradation and corrective actions if degradation is found. The Containment Inservice Inspection Program, inspection frequencies and acceptance criteria shall be in accordance with 10CFR50.55a as modified by approved exemptions. Predicted lift-off forces shall be determined consistent with the recommendations of Regulatory Guide 1.35.1, Revision 3 dated July 1990.

Any degradation exceeding the acceptance criteria of the containment structure detected during the tests required by the Containment Inservice Inspection Program shall undergo an engineering evaluation within 60 days of the completion of the inspection surveillance. The results of the engineering evaluation shall be reported to the NRC within an additional 30 days of the time the evaluation is completed. The report shall include the cause of the condition that does not meet the acceptance criteria, the acceptability of the concrete containment without repair of the item, whether or not repair or replacement is required and, if required, the extent, method, and completion of necessary repairs, and the extent, nature, and frequency of additional examinations.

In addition, any significant degradation which seriously challenges containment operability found during the inspection shall be reported to the NRC in accordance with Technical Specification 6.9.2 within 30 days. The report shall include the description of degradation, operability determination, root cause determination, and corrective actions taken.