



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
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ARLINGTON, TEXAS 76011-8064

July 25, 2000

S. K. Gambhir, Division Manager  
Nuclear Operations  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
P.O. Box 399  
Hwy. 75 - North of Fort Calhoun  
Fort Calhoun, Nebraska 68023-0399

SUBJECT: PUBLIC MEETING TO DISCUSS THE RESULTS OF THE END-OF-CYCLE  
PLANT PERFORMANCE ASSESSMENT OF FORT CALHOUN STATION

Dear Mr. Gambhir:

This refers to the meeting conducted at the Fort Calhoun Station on July 17, 2000, between your staff and the NRC. This meeting was held to discuss the results of the end-of-cycle plant performance assessment of Fort Calhoun Station conducted on May 3, 2000.

A list of attendees and a copy of presentation slides are enclosed.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Should you have any questions concerning this matter, we will be pleased to discuss them with you.

Sincerely,

*/RA/*

Kriss M. Kennedy, Chief  
Project Branch C  
Division of Reactor Projects

Docket No.: 50-285  
License No.: DPR-40

Enclosures:

1. Attendance List
2. NRC Presentation

cc w/enclosures:

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Omaha Public Power District

-3-

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Harrison County  
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- S. Richards, NRR Project Director **(SAR)**
- S. Dembek, Chief, Section 2, NRR/DLPM **(SXD)**
- R. Wharton, NRR Project Manager **(LRW)**

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RIV:PE:DRP/C	C:DRP/C			
WCSifre;dlf	KMKennedy			
<b>KMK for</b>	<b>/RA/</b>			
7/25/00	7/25/00			

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T=Telephone

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## ENCLOSURE 1

OMAHA PUBLIC POWER DISTRICT  
FORT CALHOUN STATION  
**JULY 17, 2000, MEETING ATTENDANCE LIST**

SUBJECT: PUBLIC MEETING TO DISCUSS THE RESULTS OF THE END-OF-CYCLE  
PLANT PERFORMANCE ASSESSMENT

NAME	TITLE/ORGANIZATION
Jim Chase	DM/OPPD
Dennis W. Dole	Mg. Org. Dev.
Delvin R. Trausch	Manager, Quality Assurance
Mary A. Tesar	Division Manager, NSS
Bill Ponec	Manager, Business Processes
Ralph Phelps	Division Manager, Nuclear Engineering
Steve Gebers	Corporate Health Physicist
Mike Jones	OPPD Corporate Communication
H. John Sefick	Manager, Security & ED
Dave Bannish	Manager, Operations/NOD
Ron Short	Assistant Plant Manager, FCS, OPPD
Jean Chamberlain	Manager, NPCCS, OPPD
Gary Cavanaugh	Licensing
Rich Clemens	Plant Manager
Jeff Spilker	Manager, CAG
Bill Pook	Region 5/6 Engineering Manager
Mark Puckett	Manager, Radiation Protection
Ruben Hamilton	Manager,
Sudesh K. Gambhir	Division Manager, NOD
R. L. Jaworski	Special Assignment, NOD
E. E. Morris	E&P Coordinator
Rick Westest	Manager, Training
John B. Herman	Manager, P&S
James W. Tills	Manager, Maintenance
Merl Core	Manager, System Engineering

ENCLOSURE 2

# **PLANT PERFORMANCE ASSESSMENT**

**FOR**

**FORT CALHOUN  
STATION**

**MAY 30, 1999, TO  
APRIL 1, 2000**

**REGION IV OFFICE**  
**ARLINGTON, TEXAS**

**KRISS KENNEDY**

**BRANCH CHIEF**

**ON SITE**  
**INSPECTION STAFF**

**WAYNE WALKER**

**SENIOR RESIDENT INSPECTOR**

-----

**CLYDE OSTERHOLTZ**

**RESIDENT INSPECTOR**

# PURPOSE

- **REVIEW OF SAFETY PERFORMANCE BY LICENSEE**
- **PLANNING AND ALLOCATION OF NRC INSPECTION RESOURCES**

# REACTOR OVERSIGHT PROGRAM

- **DIVIDED INTO PERFORMANCE AREAS KNOWN AS STRATEGIC PERFORMANCE ARENAS**
  - **REACTOR SAFETY** - avoiding accidents and reducing the consequences of accidents if they occur
  - **RADIATION SAFETY** - protecting plant employees and the public during routine operations
  - **SAFEGUARDS** - protecting the plant against sabotage or other security threats

# **FORT CALHOUN STATION ASSESSMENT**

- **ASSESSMENT PERIOD**
  - **MAY 30, 1999 TO APRIL 1, 2000**
  
- **ASSESSMENT BASED ON**
  - **PERFORMANCE INDICATORS  
SUBMITTED BY FORT CALHOUN  
STATION**
  
  - **INSPECTION FINDINGS**

# RESULTS

- **FINDINGS RESULTING FROM NRC INSPECTIONS WERE GREEN (VERY LOW SAFETY SIGNIFICANCE)**
- **PERFORMANCE INDICATORS FOR THE SEVEN CORNERSTONES OF SAFETY WERE GREEN, WITH THE FOLLOWING EXCEPTIONS:**

# REACTOR SAFETY

## EMERGENCY PREPAREDNESS

- **WHITE PERFORMANCE INDICATOR FOR DRILL/EXERCISE PERFORMANCE ON TWO OCCASIONS**
- **LESS THAN 90 % OF DRILLS AND EXERCISES PERFORMED IN TIMELY AND ACCURATE MANNER**
- **SUPPLEMENTAL INSPECTION CONDUCTED. CONCLUDED PROPOSED CORRECTIVE ACTIONS WERE SATISFACTORY**
- **CALCULATION OF PIs ON QUARTERLY BASIS WOULD HAVE RESULTED IN GREEN PI**

# RADIATION SAFETY

## OCCUPATIONAL RADIATION SAFETY

- **WHITE PERFORMANCE INDICATOR FOR OCCUPATIONAL EXPOSURE CONTROL EFFECTIVENESS**
- **THREE EVENTS ASSOCIATED WITH INADEQUATE CONTROL OF TS RESTRICTED HIGH RADIATION AREAS (>1000mrem/hr)**

- **SUPPLEMENTAL INSPECTION  
CONDUCTED DURING CURRENT  
ASSESSMENT CYCLE CONCLUDED**
  - **LICENSEE CONDUCTED  
COMPREHENSIVE EVALUATION  
OF EVENTS**
  - **LICENSEE CORRECTIVE  
ACTIONS WERE EXTENSIVE  
AND THOROUGH**

# **SAFEGUARDS**

**NO CONCERNS  
IDENTIFIED**

# **OVERALL** **ASSESSMENT**

- **FORT CALHOUN STATION IS BEING OPERATED IN A SAFE MANNER**
  
- **PLANNED INSPECTION ACTIVITY**
  - **BASELINE INSPECTIONS**
  
  - **SUPPLEMENTAL INSPECTION IN AREA OF OCCUPATIONAL EXPOSURE CONTROL EFFECTIVENESS (COMPLETED)**

## NRC'S REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

<b>Reactor Safety</b>	<b>Radiation Safety</b>	<b>Safeguards</b>
<ul style="list-style-type: none"><li>•Initiating Events</li><li>•Mitigating Systems</li><li>•Barrier Integrity</li><li>•Emergency Preparedness</li></ul>	<ul style="list-style-type: none"><li>•Occupational</li><li>•Public</li></ul>	<ul style="list-style-type: none"><li>•Physical Protection</li></ul>

To monitor these seven cornerstones of safety, the NRC used two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW, or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, or RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspections so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: <http://www.nrc.gov/NRR/OVERSIGHT/index.html>