



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

July 20, 2000

Mr. John K. Wood  
Vice President - Nuclear, Perry  
FirstEnergy Nuclear Operating Company  
P.O. Box 97, A200  
Perry, OH 44081

SUBJECT: PERRY NUCLEAR POWER PLANT, UNIT 1 - CORRECTIONS TO  
AMENDMENT NOS. 111 AND 112 (TAC NOS. MA6462 & MA6459)

Dear Mr. Wood:

Our letters of March 30 and June 1, 2000, transmitted Amendment Nos. 111 and 112, respectively, to Facility Operating License No. NPF-58 for the Perry Nuclear Power Plant, Unit 1. Amendment No. 111 approved nine minor, unrelated revisions to the technical specifications (TSs) which included clarifications and editorial changes. Amendment No. 112 provided staff approval for a five percent power uprate at the Perry facility.

Your staff recently informed us that changes were either needed or desired on several of the reissued TS pages. Specifically, your staff identified the following:

- TS page 3.8-21, from Amendment No. 111, inadvertently deleted the word "Air" from the title, "Diesel Fuel Oil, Lube Oil, and Starting Air."
- TS page 5.0-15a, from Amendment No. 112, included undesirable font and wrap-around problems.
- TS page 5.0-16, from Amendment No. 111, did not include revision bars for all changes.

Enclosed please find the revised pages of the TSs. We apologize for any inconvenience that may have resulted.

Sincerely,

Douglas V. Pickett, Senior Project Manager, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-440

Enclosure: As stated

cc w/encl: See next page

July 20, 2000

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Vice President - Nuclear, Perry  
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Perry, OH 44081

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FirstEnergy Nuclear Operating Company

Perry Nuclear Power Plant, Units 1 and 2

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3.8 ELECTRICAL POWER SYSTEMS

3.8.3 Diesel Fuel Oil, Lube Oil, and Starting Air

LC0 3.8.3 The stored diesel fuel oil, lube oil, and starting air subsystem shall be within limits for each required diesel generator (DG).

APPLICABILITY: When associated DG is required to be OPERABLE.

ACTIONS

-----NOTE-----  
Separate Condition entry is allowed for each DG.  
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| CONDITION   | REQUIRED ACTION   | COMPLETION TIME |
|---|---|-----------------|
| <p>A. One or more DGs with fuel oil level:</p> <ol style="list-style-type: none"> <li>1. For Div 1 and Div 2,<br/>&lt; 73,700 gal and ≥ 65,100 gal; and</li> <li>2. For Div 3,<br/>&lt; 36,700 gal and ≥ 32,000 gal.</li> </ol> | <p>A.1 Restore fuel oil level to within limits.</p>     | <p>48 hours</p> |
| <p>B. One or more DGs with lube oil inventory:</p> <ol style="list-style-type: none"> <li>1. For Div 1 and Div 2,<br/>&lt; 374 gal and ≥ 350 gal; and</li> <li>2. For Div 3,<br/>&lt; 260 gal and ≥ 236 gal.</li> </ol>         | <p>B.1 Restore lube oil inventory to within limits.</p> | <p>48 hours</p> |

(continued)

## 5.5 Programs and Manuals

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### 5.5.12 Primary Containment Leakage Rate Testing Program (continued)

- BN-TOP-1 methodology may be used for Type A tests.
- The corrections to NEI 94-01 which are identified on the Errata Sheet attached to the NEI letter, "Appendix J Workshop Questions and Answers," dated March 19, 1996 are considered an integral part of NEI 94-01.
- The containment isolation check valves in the Feedwater penetrations are tested per the Inservice Testing Program (Technical Specification 5.5.6).

The peak calculated primary containment internal pressure for the design basis loss of coolant accident is 6.40 psig. For conservatism  $P_a$  is defined as 7.80 psig.

The maximum allowable primary containment leakage rate,  $L_a$ , shall be 0.20% of primary containment air weight per day at the peak containment pressure ( $P_a$ ).

Leakage rate acceptance criteria are:

- a. Primary containment leakage rate acceptance criterion is  $\leq 1.0 L_a$ . However, during the first unit startup following testing performed in accordance with this Program, the leakage rate acceptance criteria are  $< 0.6 L_a$  for the Type B and Type C tests, and  $\leq 0.75 L_a$  for the Type A tests;
- b. Air lock testing acceptance criteria are:
  - 1) Overall air lock leakage rate is  $\leq 2.5$  scfh when tested at  $\geq P_a$ .
  - 2) For each door, leakage rate is  $\leq 2.5$  scfh when the gap between the door seals is pressurized to  $\geq P_a$ .

The provisions of SR 3.0.2 do not apply to the test frequencies specified in the Primary Containment Leakage Rate Testing Program.

The provisions of SR 3.0.3 are applicable to the Primary Containment Leakage Rate Testing Program.

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(continued)

## 5.0 ADMINISTRATIVE CONTROLS

### 5.6 Reporting Requirements

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The following reports shall be submitted in accordance with 10 CFR 50.4.

#### 5.6.1 Occupational Radiation Exposure Report

A tabulation on an annual basis of the number of station, utility, and other personnel (including contractors), for whom monitoring was required, receiving exposures > 100 mrem/yr and their associated man rem exposure according to work and job functions, (e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling). This tabulation supplements the requirements of 10 CFR 20.2206. The dose assignments to various duty functions may be estimated based on pocket dosimeter, thermoluminescent dosimeter (TLD), or film badge measurements. Small exposures totalling < 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources should be assigned to specific major work functions.

The Occupational Radiation Exposure Report covering the activities of the unit for the previous year shall be submitted by April 30 of each year.

#### 5.6.2 Annual Radiological Environmental Operating Report

The Annual Radiological Environmental Operating Report covering the operation of the unit during the previous year shall be submitted by May 1 of each year. The report shall include summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in the Offsite Dose Calculation Manual (ODCM), and in 10 CFR 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C.

The Annual Radiological Environmental Operating Report shall include the results of analyses of all radiological environmental samples and of all environmental radiation measurements taken during the period pursuant to the locations specified in the table and figures in the ODCM, as well as summarized and tabulated

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