



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

LRN-00-0266

**JUL 14 2000**

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT  
SALEM UNIT NO. 1  
DOCKET NO. 50-272**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for June 2000 is attached.

Sincerely,

A handwritten signature in black ink, appearing to read "M. B. Bezilla" with a stylized flourish at the end.

M. B. Bezilla  
Vice President - Operations

/rbk  
Enclosures

C Mr. H. J. Miller  
Regional Administrator USNRC, Region 1  
475 Allendale Road  
King of Prussia, PA 19046

Handwritten initials "JE24" in black ink, located in the bottom right corner of the page.

The power is in your hands.

DOCKET NO.: 50-272  
 UNIT: Salem 1  
 DATE: 7/17/00  
 COMPLETED BY: R. Knieriem  
 TELEPHONE: (856) 339-1782

Reporting Period: June 2000

**OPERATING DATA REPORT**

Design Electrical Rating (MWe-Net)  
 Maximum Dependable Capacity (MWe-Net)  
 No. of hours reactor was critical  
 No. of hours generator was on line (service hours)  
 Unit reserve shutdown hours  
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
720	4118	122663
710	4044	118296
0	0	0
742989	4348663	118969998

**UNIT SHUTDOWNS**

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT
3	6/3/00	F	10.0	A	5	EHC system leak on 12W intercept valve. Reactor not shut down.

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F Administrative
- G- Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

**Summary:**

Salem Unit 1 began the month of June 2000 operating at full power. On June 3, power was reduced to 20% to repair a Main Turbine Electro-hydraulic Control System leak. Unit 1 returned to full power on June 4. However, another power reduction to 89% power was necessary on June 4, to correct a leaking Moisture Separator Reheater relief valve. Full power operation was resumed late that day, and continued until June 12, when power was reduced to 74% in response to the loss of an off-site transmission line. Salem Unit 1 returned to full power on the following day and continued full power operation until June 22, when power was reduced to 90% to repair the 13C

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Feedwater Heater. Salem Unit 1 returned to full power on June 27, and operated at full power for the remainder of the month.

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**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS  
FOR THE SALEM UNIT 1 GENERATING STATION**

**MONTH: June 2000**

The following items completed during **June 2000** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

**Design Changes - Summary of Safety Evaluations**

**1EC-3651, Package 3, Modification to Station Air Compressor Controls**

This design change implemented vendor recommended modifications to the Station Air Compressors to eliminate compressor cycling during low load operation. The modification changed the compressor control from a total enclosure control scheme to a constant pressure control scheme and involved the installation of control enclosure, the installation of an orifice in the 3<sup>rd</sup> stage discharge piping, and the installation of a differential pressure transmitter across the newly installed orifice.

Review of this design change under 10CFR50.59 was required because the modification constitutes a change to the facility as described in the SAR. The Station Air Compressors are not safety-related and perform no safety function. Therefore, this design change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change would not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the

Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

**Temporary Modifications - Summary of Safety Evaluations**

There were no changes in this category implemented during June 2000.

**Procedures - Summary of Safety Evaluations**

There were no changes in this category implemented during June 2000.

**UFSAR Change Notices - Summary of Safety Evaluations**

There were no changes in this category implemented during June 2000.

**Procedures - Summary of Safety Evaluations**

There were no changes in this category implemented during June 2000.

**Deficiency Reports - Summary of Safety Evaluations**

There were no changes in this category implemented during June 2000.

**Other - Summary of Safety Evaluations**

There were no changes in this category implemented during June 2000.