

July 20, 2000

Mr. Craig G. Anderson
Vice President, Operations ANO
Entergy Operations, Inc.
1448 S. R. 333
Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT 2 - PUBLIC NOTICE OF APPLICATION FOR
AMENDMENT TO FACILITY OPERATING LICENSE (TAC NO. MA9493)

Dear Mr. Anderson:

The enclosed announcement was forwarded to The Courier (in Russellville) and the Arkansas Democrat-Gazette (in Little Rock) for publication. This announcement relates to your application dated July 13, 2000, for amendment to Facility Operating License No. NPF-6. The proposed amendment would revise Technical Specification 4.4.5.0 with a Note to allow the performance of a special steam generator tubing inspection during the 2P00 mid-cycle outage utilizing different scope and expansion criteria than currently specified. The Note would also state that the scope and expansion criteria for the inspection are specified in correspondence to the NRC submitted under separate cover, and that the scope and expansion criteria shall be approved by the NRC prior to entering Mode 4.

A separate notice will be published later in the *Federal Register* concerning the revision to the Steam Generators Surveillance Requirements Section 4.4.5.0 of the Technical Specifications.

Sincerely,

/RA/

Thomas W. Alexion, Project Manager, Section 1
Project Directorate IV and Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-368

Enclosure: Public Notice

cc w/encl: See next page

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/RA/

Thomas W. Alexion, Project Manager, Section 1
Project Directorate IV and Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosure: Public Notice

cc w/encl: See next page

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* See previous concurrence

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|--------|------------|------------|---------------|-----------|
| OFFICE | PDIV-1/PM* | PDIV-D/LA* | OGC* | PDIV-1/SC |
| NAME | TAlexion | DJohnson | RWeissman | RGramm |
| DATE | 07/17/00 | 07/17/00 | July 17, 2000 | 7/18/00 |

OFFICIAL RECORD COPY

Arkansas Nuclear One

cc:

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& Chief Operating Officer
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PUBLIC NOTICE

NRC STAFF PROPOSES TO AMEND FACILITY OPERATING LICENSE AT ARKANSAS NUCLEAR ONE, UNIT 2

The U. S. Nuclear Regulatory Commission (NRC) staff has received an application dated July 13, 2000, from Entergy Operations, Inc. (Entergy Operations or the licensee), for an exigent amendment to the operating license for Arkansas Nuclear One, Unit 2 (ANO-2). The facility is located in Pope County, approximately 57 miles northwest of Little Rock, Arkansas.

The proposed amendment would revise Technical Specification (TS) 4.4.5.0 with a Note to allow the performance of a special steam generator tubing inspection during the 2P00 mid-cycle outage utilizing different scope and expansion criteria than currently specified. The proposed Note would also state that the scope and expansion criteria for the inspection are specified in correspondence to the NRC submitted under separate cover, and that the scope and expansion criteria shall be approved by the NRC prior to entering Mode 4 (upon completion of the outage).

By way of background, on February 11, 2000, the licensee submitted to the NRC a deterministic operational assessment of steam generator tubing to justify continued operation for the remainder of Cycle 14. The emphasis of the operational assessment was to evaluate approximately one half-cycle of operation for eggcrate axial flaws on the hot leg portion of the steam generators. Cycle 14 is scheduled to end in September 2000, at which time ANO-2 will enter its refueling outage and the steam generators will be replaced. On March 9, 2000, the licensee also submitted a license amendment request to allow the use of a risk-informed approach to operate ANO-2 for the remainder of Cycle 14. By letter dated July 14, 2000, the NRC made the licensee aware of concerns with both of these approaches. As stated in the letter, these concerns had been discussed in a telephone conversation between the NRC staff and the licensee on July 13, 2000. Due to these concerns, the licensee proposes to perform a

special mid-cycle inspection of the steam generators during an outage that is scheduled to begin on July 21, 2000. Due to the short time interval between the submission of this request and the actual performance of the proposed special steam generator inspection, insufficient time remains for normal NRC processing and notification. Therefore, the licensee requested that this proposed TS change be considered as submitted under exigent circumstances as described in 10 CFR 50.91(a)(6).

Pursuant to 10 CFR 50.91(a)(6), for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration, as defined in 10 CFR 50.92.

The licensee and the NRC staff have evaluated this proposed change with regard to the determination of whether or not a significant hazards consideration is involved.

Operation of ANO-2 in accordance with the proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated. As stated in the application:

This change has no actual impact on any previously analyzed accident in the final safety analysis report (FSAR). A double-ended break of one steam generator tube is postulated as part of the ANO-2 design basis accident evaluation. The change permits Energy Operations to determine the appropriate scope and expansion criteria for a special steam generator tube inspection that is being performed at a frequency more conservative than that of the augmented inservice inspection program included in the TSs. The special inspection will find and repair certain steam generator tubing flaws that would otherwise remain in service until the next scheduled refueling outage. The increased inspection frequency reduces the probability that a flaw in a steam generator tube could grow to a size that would affect the leakage or structural integrity of the tube. The augmented inservice inspection program contained in the TSs is not being modified.

This change does not modify any parameter that will increase radioactivity in the primary system or increase the amount of radioactive steam released from the secondary safety valves or atmospheric dump valves in the event of a tube rupture.

Therefore, this change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated. As stated in the application:

The scope of this change does not establish a potential new accident precursor. The design basis accident analyses for ANO-2 include the consequences of a double-ended break of one steam generator tube which bounds other postulated failure mechanisms. The proposed change would permit determination of alternate inspection criteria for a special inspection which is in addition to the periodic inservice inspections required by the TSs. The equipment used in the special inspection would not affect any plant components differently than those used for current TS required inspections.

Therefore, this change does not create the possibility of a new or different kind of accident from any previously evaluated.

The proposed amendment will not involve a significant reduction in a margin of safety.

As stated in the application:

As previously stated, a double-ended rupture of one steam generator tube is accounted for in the ANO-2 design basis accident analysis. Considering that the 2P00 special inspection is in addition to the inservice inspection program defined in the ANO-2 TSs and that leakage detection capability is not being modified, performance of a special inspection of any scope will increase the margin of safety over the current TS requirements.

Therefore, this change does not involve a significant reduction in the margin of safety.

Following an initial review of this application, the requested amendment has been evaluated against the standards in 10 CFR 50.92. Based on the analysis in the application, as set forth above, it appears that the changes do not significantly increase the probability or consequences of any accident previously considered, do not create the possibility of an accident of a different kind, and do not significantly decrease any margin of safety.

Accordingly, the NRC staff proposes to determine that the requested amendment involves no significant hazards consideration.

If the proposed determination that the requested license amendment involves no significant hazards consideration becomes final, the staff will issue the amendment without first

offering an opportunity for a prior public hearing. An opportunity for a hearing will be published in the *Federal Register*.

If the staff decides in its final determination that the amendment does involve a significant hazards consideration, a notice of opportunity for a prior hearing will be published in the *Federal Register* and, if a hearing is granted, it will be held before the amendment is issued.

Comments on the proposed determination of no significant hazards consideration may be (1) telephoned to Robert A. Gramm, Chief, Section IV-1, Project Directorate IV and Decommissioning, Office of Nuclear Reactor Regulation, by collect call to 301-415-1010, or by facsimile to 301-415-7025; (2) e-mailed to rag@nrc.gov; or (3) submitted in writing to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555. All comments received by close of business on July 24, 2000, from 7:30 a.m. to 4:15 p.m., on Federal workdays, will be considered in reaching a final determination. A copy of the application may be examined electronically through the ADAMS Public Electronic Reading Room link at the NRC Web site (<http://www.nrc.gov>) and at the Commission's Public Document Room, 2120 L Street, Gelman Building, NW., Washington, DC 20555.