



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 11, 2000

Mr. Michael D. Wadley, President
NSP Nuclear Generation
Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

**SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: RELOCATION OF SHUTDOWN MARGIN
REQUIREMENTS TO THE CORE OPERATING LIMITS REPORT
(TAC NOS. MA5294 AND MA5297)**

Dear Mr. Wadley:

The Commission has issued the enclosed Amendment No. 151 to Facility Operating License No. DPR-42 and Amendment No. 142 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated April 12, 1999, as supplemented July 7, 2000.

The amendments revise several TS sections in order to relocate shutdown margin requirements to the Core Operating Limits Report and to ensure that the TS requirements are consistent with the dilution analysis in the Updated Safety Analysis Report.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Tae Kim, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures: 1. Amendment No. 151 to DPR-42
2. Amendment No. 142 to DPR-60
3. Safety Evaluation

cc w/encl: See next page

July 11, 2000

Mr. Michael D. Wadley, President
NSP Nuclear Generation
Northern States Power Company
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Minneapolis, MN 55401

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/RA/

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Project Directorate III
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Docket Nos. 50-282 and 50-306

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1. Amendment No. 151 to DPR-42
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cc w/encl: See next page

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NAME	TKim <i>AK</i>	RBouling <i>AK</i>	FAkstulewicz	WBeckner	LCrocker	CCraig <i>AK</i>
DATE	7/11/00	7/11/00	06/05/00	06/06/00	06/26/00	7/11/00

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Prairie Island Nuclear Generating Plant,
Units 1 and 2

cc:

J. E. Silberg, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington, DC 20037

Plant Manager
Prairie Island Nuclear Generating Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, MN 55089

Adonis A. Neblett
Assistant Attorney General
Office of the Attorney General
455 Minnesota Street
Suite 900
St. Paul, MN 55101-2127

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
1719 Wakonade Drive East
Welch, MN 55089-9642

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, IL 60532-4351

Mr. Stephen Bloom, Administrator
Goodhue County Courthouse
Box 408
Red Wing, MN 55066-0408

Commissioner
Minnesota Department of Commerce
121 Seventh Place East
Suite 200
St. Paul, MN 55101-2145

Site Licensing
Prairie Island Nuclear Generating Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, MN 55089

Tribal Council
Prairie Island Indian Community
ATTN: Environmental Department
5636 Sturgeon Lake Road
Welch, MN 55089

Site General Manager
Prairie Island Nuclear Generating Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, MN 55089

January 2000



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 151
License No. DPR-42

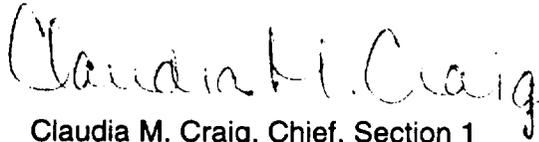
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated April 12, 1999, as supplemented July 7, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 151, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days. In addition, the licensee shall include the relocated information in the Core Operating Limits Report submitted to the NRC, as was described in the licensee's application dated April 12, 1999, as supplemented July 7, 2000, and evaluated in the NRC staff's safety evaluation dated July 11, 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 11, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 151

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

Table 1-1
3.10-1
3.10-5
Figure TS 3.10-1
TS 6.0-13

INSERT

Table 1-1
TS 3.10-1
TS 3.10-5
-
TS 6.0-13

TABLE TS.1-1
OPERATIONAL MODES

<u>MODE</u>	<u>TITLE</u>	<u>REACTIVITY CONDITION</u>	<u>%RATED THERMAL POWER</u>	<u>AVERAGE COOLANT TEMPERATURE</u>	<u>REACTOR VESSEL HEAD CLOSURE BOLTS FULLY TENSIONED</u>
1	POWER OPERATION	Critical	> 2%	NA	YES
2	HOT STANDBY**	Critical	≤ 2%	NA	YES
3	HOT SHUTDOWN**	Subcritical	NA	≥ 350°F	YES
4	INTERMEDIATE SHUTDOWN**	Subcritical	NA	< 350°F ≥ 200°F	YES
5	COLD SHUTDOWN	Subcritical	NA	< 200°F	YES
6	REFUELING	NA*	NA	NA	NO

* Boron concentration of the reactor coolant system and the refueling cavity sufficient to ensure that the more restrictive of the following conditions is met:

- a. $K_{eff} \leq 0.95$,
- b. Boron concentration ≥ 2000 ppm, or
- c. Shutdown Margin as specified in the Core Operating Limits Report.

** Prairie Island specific MODE title, not consistent with Standard Technical Specification MODE titles. MODE numbers are consistent with Standard Technical Specification MODE numbers.

PRAIRIE ISLAND UNIT 1
 PRAIRIE ISLAND UNIT 2

Amendment No. 111, 151
 Amendment No. 104, 142

3.10 CONTROL ROD AND POWER DISTRIBUTION LIMITS

Applicability

Applies to the limits on core fission power distribution and to the limits on control rod operations.

Objective

To assure 1) core subcriticality after reactor trip, 2) acceptable core power distributions during POWER OPERATION, and 3) limited potential reactivity insertions caused by hypothetical control rod ejection.

SpecificationA. Shutdown Margin

1. The SHUTDOWN MARGIN shall be maintained within the limits specified in the Core Operating Limits Report when in HOT SHUTDOWN, INTERMEDIATE SHUTDOWN and COLD SHUTDOWN.
2. With the SHUTDOWN MARGIN less than the applicable limit specified in 3.10.A.1 above, within 15 minutes initiate boration to restore SHUTDOWN MARGIN to within the applicable limit.

B. Power Distribution Limits

1. At all times, except during low power PHYSICS TESTING, measured hot channel factors, F_Q^N and $F_{\Delta H}^N$, as defined below and in the bases, shall meet the following limits:

$$F_Q^N \times 1.03 \times 1.05^* \leq (F_Q^{RTP} / P) \times K(Z)$$

$$F_{\Delta H}^N \times 1.04^{**} \leq F_{\Delta H}^{RTP} \times [1 + PFDH(1-P)]$$

where the following definitions apply:

- F_Q^{RTP} is the F_Q limit at RATED THERMAL POWER specified in the CORE OPERATING LIMITS REPORT.
- $F_{\Delta H}^{RTP}$ is the $F_{\Delta H}$ limit at RATED THERMAL POWER specified in the CORE OPERATING LIMITS REPORT.
- PFDH is the Power Factor Multiplier for $F_{\Delta H}^N$ specified in the CORE OPERATING LIMITS REPORT.
- $K(Z)$ is a normalized function that limits $F_Q(z)$ axially as specified in the CORE OPERATING LIMITS REPORT.

*For Unit 1, Cycle 19, when the number of available moveable detector thimbles is greater than or equal to 50% and less than 75% of the total the 5% measurement uncertainty shall be increased to $[5\% + (3-T/9)(3\%)]$ where T is the number of available thimbles.

**For Unit 1, Cycle 19, when the number of available moveable detector thimbles is greater than or equal to 50% and less than 75% of the total, the 4% measurement uncertainty shall be increased to $[4\% + (3-T/9)(2\%)]$ where T is the number of available thimbles.

PRAIRIE ISLAND UNIT 1
PRAIRIE ISLAND UNIT 2

Amendment No. 111, 124, 130, 151
Amendment No. 83, 104, 142

- 3.10.C.2. If the QUADRANT POWER TILT RATIO exceeds 1.02 but is less than 1.07 for a sustained period of more than 24 hours, or if such a tilt recurs intermittently, the reactor shall be brought to the HOT SHUTDOWN condition. Subsequent operation below 50% of rating, for testing, shall be permitted.
3. Except for PHYSICS TESTS if the QUADRANT POWER TILT RATIO exceeds 1.07, the reactor shall be brought to the HOT SHUTDOWN condition. Subsequent operation below 50% of rating, for testing, shall be permitted.
4. If the core is operating above 85% power with one excore nuclear channel inoperable, then the core quadrant power balance shall be determined daily and after a 10% power change using either 2 movable detectors or 4 core thermocouples per quadrant, per Specification 3.11.

D. Rod Insertion Limits

1. The shutdown rods shall be limited in physical insertion as specified in the CORE OPERATING LIMITS REPORT when the reactor is critical or approaching criticality.
2. When the reactor is critical or approaching criticality, the control banks shall be limited in physical insertion as specified in the CORE OPERATING LIMITS REPORT.
3. Insertion limits do not apply during PHYSICS TESTS or during periodic exercise of individual rods. The shutdown margin specified in the Core Operating Limits Report must be maintained except for low power PHYSICS TESTING. For this test the reactor may be critical with all but one high worth full-length control rod inserted for a period not to exceed 2 hours per year provided a rod drop test is run on the high worth full-length rod prior to this particular low power PHYSICS TEST.

C. Radioactive Effluent Report

The Radioactive Effluent Report covering the operation of the plant during the previous calendar year shall be submitted by May 15 of each year. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the plant. The material provided shall be consistent with the objectives outlined in the ODCM and in conformance with 10CFR50.36a and 10CFR50, Appendix I, Section IV.B.1.

D. Monthly Operating Reports

Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the pressurizer power operated relief valves or pressurizer safety valves, shall be submitted on a monthly basis no later than the 15th of each month following the calendar month covered by the report.

E. Core Operating Limits Report (COLR)

1. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:

- a. Heat Flux Hot Channel Factor Limit (F_0^{RTP}), Nuclear Enthalpy

Rise Hot Channel Factor Limit (FA_m^{RTP}), PFDH, K(Z) and V(Z)
(Specifications 3.10.B.1, 3.10.B.2 and 3.10.B.3)

- b. Axial Flux Difference Limits and Target Band
(Specifications 3.10.B.4 through 3.10.B.9)

- c. Shutdown and Control Bank Insertion Limits
(Specification 3.10.D)

- d. Reactor Coolant System Flow Limit (Specification 3.10.J)

- e. Shutdown Margin
(Table TS.1-1 and Specifications 3.10.A.1 and 3.10.D.3)

2. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

NSPNAD-8101-A, "Qualification of Reactor Physics Methods for Application to PI Units" (latest approved version)

NSPNAD-8102-A, "Prairie Island Nuclear Power Plant Reload Safety Evaluation Methods for Application to PI Units" (latest approved version)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-60

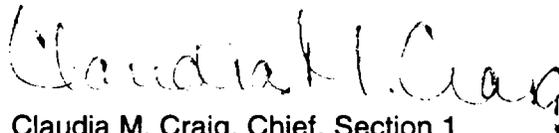
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated April 12, 1999, as supplemented July 7, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 142, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days. In addition, the licensee shall include the relocated information in the Core Operating Limits Report submitted to the NRC, as was described in the licensee's application dated April 12, 1999, as supplemented July 7, 2000, and evaluated in the NRC staff's safety evaluation dated July 11, 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 11, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

Table TS.1-1
TS.3.10-1
TS.3.10-5
Figure TS.3.10-1
TS.6.0-13

INSERT

Table TS.1-1
TS.3.10-1
TS.3.10-5
-
TS.6.0-13

TABLE TS.1-1
OPERATIONAL MODES

<u>MODE</u>	<u>TITLE</u>	<u>REACTIVITY CONDITION</u>	<u>%RATED THERMAL POWER</u>	<u>AVERAGE COOLANT TEMPERATURE</u>	<u>REACTOR VESSEL HEAD CLOSURE BOLTS FULLY TENSIONED</u>
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5	COLD SHUTDOWN	Subcritical	NA	< 200°F	YES
6	REFUELING	NA*	NA	NA	NO

* Boron concentration of the reactor coolant system and the refueling cavity sufficient to ensure that the more restrictive of the following conditions is met:

- a. $K_{eff} \leq 0.95$,
- b. Boron concentration ≥ 2000 ppm, or
- c. Shutdown Margin as specified in the Core Operating Limits Report.

** Prairie Island specific MODE title, not consistent with Standard Technical Specification MODE titles. MODE numbers are consistent with Standard Technical Specification MODE numbers.

PRAIRIE ISLAND UNIT 1
 PRAIRIE ISLAND UNIT 2

Amendment No. 111, 151
 Amendment No. 104, 142

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Objective

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1. The SHUTDOWN MARGIN shall be maintained within the limits specified in the Core Operating Limits Report when in HOT SHUTDOWN, INTERMEDIATE SHUTDOWN and COLD SHUTDOWN.
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where the following definitions apply:

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**For Unit 1, Cycle 19, when the number of available moveable detector thimbles is greater than or equal to 50% and less than 75% of the total, the 4% measurement uncertainty shall be increased to $[4\% + (3-T/9)(2\%)]$ where T is the number of available thimbles.

PRAIRIE ISLAND UNIT 1
PRAIRIE ISLAND UNIT 2

Amendment No. 111, 124, 136, 151
Amendment No. 83, 104, 142

- 3.10.C.2. If the QUADRANT POWER TILT RATIO exceeds 1.02 but is less than 1.07 for a sustained period of more than 24 hours, or if such a tilt recurs intermittently, the reactor shall be brought to the HOT SHUTDOWN condition. Subsequent operation below 50% of rating, for testing, shall be permitted.
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Rise Hot Channel Factor Limit (F_{Δ}^{HTP}), PFDR, K(Z) and V(Z)
(Specifications 3.10.B.1, 3.10.B.2 and 3.10.B.3)

b. Axial Flux Difference Limits and Target Band
(Specifications 3.10.B.4 through 3.10.B.9)

c. Shutdown and Control Bank Insertion Limits
(Specification 3.10.D)

d. Reactor Coolant System Flow Limit (Specification 3.10.J)

e. Shutdown Margin
(Table TS.1-1 and Specifications 3.10.A.1 and 3.10.D.3)

2. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

NSFNAD-8101-A, "Qualification of Reactor Physics Methods for Application to PI Units" (latest approved version)

NSFNAD-8102-A, "Prairie Island Nuclear Power Plant Reload Safety Evaluation Methods for Application to PI Units" (latest approved version)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-42
AND AMENDMENT NO. 142 TO FACILITY OPERATION LICENSE NO. DPR-60
NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2
DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By application dated April 12, 1999, as supplemented July 7, 2000, Northern States Power Company (NSP or the licensee) requested changes to the Technical Specifications (TSs) for the Prairie Island Nuclear Generating Plant, Units 1 and 2. The proposed changes would revise various TS sections in order to relocate shutdown margin requirements to the Core Operating Limits Report (COLR). NRC Generic Letter (GL) 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications," issued on October 3, 1988, provided guidance to licensees for the relocation of certain cycle-dependent core operating limits from the TSs to the COLR. This allows changes to the values of these cycle-dependent core operating limits without prior NRC approval, provided that the NRC-approved methodology for calculating those values is followed. Shutdown margin was not one of the parameters included in GL 88-16 that could be relocated to the COLR. However, Technical Specification Task Force (TSTF) Standard Technical Specification Change Traveler TSTF-9, Revision 1, proposed the relocation of shutdown margin parameters from the TSs to the COLR. TSTF-9, Revision 1, was endorsed by the NRC staff on September 18, 1996. The relocated information in the COLR will be subject to the provisions of 10 CFR 50.59. The July 7, 2000, supplemental letter provided clarifying information that was within the scope of the original application and did not change the staff's original proposed no significant hazards consideration determination.

2.0 EVALUATION

These amendments relocate the shutdown margin limits from the TSs to the COLR. The shutdown margin limits being incorporated into the COLR will be developed utilizing methodology described in NSPNAD-8102-P, Revision 7, "Prairie Island Nuclear Power Plant Reload Safety Evaluation Methods for Application to PI Units," dated January 1999. NSPNAD-8102-P, Revision 7, was approved by the staff in a safety evaluation dated July 15, 1999. That methodology will establish the minimum required shutdown margin for a postulated dilution accident during operating Modes 3, 4, 5, and 6, and will ensure that a complete loss of shutdown margin will not occur for at least 24 minutes from initiation of the dilution, as specified in the Prairie Island Updated Safety Analysis Report. The relocation of the shutdown limits to the COLR is an administrative change which has no adverse impact upon plant operation or safety.

These amendments include the following changes to the TSs in order to relocate the shutdown margin limits to the COLR:

2.1 Changes to TABLE TS.1-1, Operational Modes

Table TS.1-1 specifies each operating mode. For Mode 6 (refueling), the current requirement for reactivity condition requires that the more restrictive of the following conditions are met: 1) $K_{eff} \leq 0.95$, or 2) Boron Concentration ≥ 2000 ppm. By these amendments, a third condition, "Shutdown Margin as specified in the Core Operating Limits Report," is added. A reference to the COLR is being incorporated to specify additional shutdown margin requirements for Mode 6 (refueling) operation. This change will ensure that the most restrictive shutdown margin requirement will be invoked during Mode 6 operation.

2.2 Changes to TS.3.10.A, Shutdown Margin

TS.3.10.A specifies the shutdown margin requirements for operating Modes 3 (hot shutdown), 4 (intermediate shutdown), and 5 (cold shutdown). For operating Modes 3 and 4, the shutdown margin is required to be greater than or equal to the applicable value shown in Figure TS.3.10-1. For Mode 5, the shutdown margin is required to be greater than or equal to $1\% \Delta k/k$. By these amendments, TS.3.10.A would state: "1. The SHUTDOWN MARGIN shall be maintained within the limits specified in the Core Operating Limits Report when in HOT SHUTDOWN, INTERMEDIATE SHUTDOWN, and COLD SHUT DOWN. 2. With the SHUTDOWN MARGIN less than the applicable limit specified in 3.10.A.1 above, within 15 minutes initiate boration to restore SHUTDOWN MARGIN to within the applicable limit."

2.3 Changes to TS.3.10.D, Rod Insertion Limits

Current TS.3.10.D.3 states, in part, that "...The shutdown margin shown in Figure TS.3.10-1 must be maintained except for low power PHYSICS TESTING..." These amendments delete references to Figure TS.3.10-1, and instead refers to the Core Operating Limits Report.

2.4 Deletion of Figure TS.3.10-1

These amendments delete Figure TS.3.10-1 since Figure TS.3.10-1 is no longer referenced in other parts of the TSs. As stated above, TS.3.10.A and TS.3.10.D.3 are being revised to reference the COLR instead of Figure TS.3.10-1.

2.5 Changes to TS.6.7.A.6, Core Operating Limits Report

TS.6.7.A.6 specifies a set of core operating limits that must be established and documented in the COLR before each reload cycle or any remaining part of a reload cycle. These amendments add the requirement for the shutdown margin limit in the COLR.

The above changes are administrative in nature since the changes involve relocating the shutdown margin limits from the TSs to the COLR. The requirement to establish the shutdown margin limits in accordance with the specified methodology remains in TS.6.7.A.6.

TS.6.7.A.6.b specifies that "The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:... NSPNAD-8102-A, 'Prairie Island Nuclear Power Plant Reload

Safety Evaluation Methods for Application to PI Units' (latest approved version)..." As stated previously, the shutdown margin limits being incorporated into the COLR will be developed utilizing methodology described in NSPNAD-8102, Revision 7, which is the latest version approved by the staff on July 15, 1999. That methodology will establish the minimum required shutdown margin for a postulated dilution accident during operating Modes 3, 4, 5, and 6, and will ensure that a complete loss of shutdown margin will not occur for at least 24 minutes from initiation of the dilution, as specified in the Prairie Island Updated Safety Analysis Report.

The staff has concluded that these amendments are acceptable since the relocation of the shutdown limits to the COLR is an administrative change which has no adverse impact upon plant operation or safety.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (65 FR 24999). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Kim

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