

Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee 37379

July 7, 2000

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Gentlemen:

In the Matter of)Docket Nos. 50-327Tennessee Valley Authority)50-328

SEQUOYAH NUCLEAR PLANT (SQN) - UNITS 1 AND 2 - EMERGENCY PLAN IMPLEMENTING PROCEDURE (EPIP) REVISION

In accordance with the requirements of 10 CFR 50, Appendix E, Section V, the enclosure provides the following EPIP:

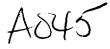
EPIP	Revision	Title
EPIP-1	30	Emergency Plan Classification Matrix

If you have any questions concerning this matter, please telephone me at (423) 843-7170 or J. D. Smith at (423) 843-6672.

Sincer

Redro Salas Licensing and Industry Affairs Manager

Enclosure cc: See page 2



U.S. Nuclear Regulatory Commission Page 2 July 7, 2000

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ENCLOSURE

EMERGENCY PLAN IMPLEMENTING PROCEDURE REVISION

TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT

EMERGENCY PLAN IMPLEMENTING PROCEDURE

EPIP-1

EMERGENCY PLAN CLASSIFICATION MATRIX

Revision 30

QUALITY RELATED

PREPARED BY: W. P. BROOKS

RESPONSIBLE ORGANIZATION: <u>Emergency Preparedness</u>

APPROVED BY: D. L. Koehl

EFFECTIVE DATE:06/30/2000

LEVEL OF USE: REFERENCE

REVISION DESCRIPTION: INTENT REVISION. Revised to incorporate changes from NP-REP, Appendix B, Revisions 54 and 56.

1.0 PURPOSE

This procedure provides criteria to the Shift Manager (SM) or Site Emergency Director (SED) to be used in classifying and declaring an emergency based on plant conditions. The responsibility for declaring an emergency, based on the criteria in this procedure, belongs to the SM or SED, the designated Unit Supervisor when acting as the SM, or the TSC SED. This responsibility **cannot** be delegated.

2.0 REFERENCES

2.1 Developmental Documents

- A. 10 CFR 50, Domestic Licensing Of Production And Utilization Facilities
- B. Reg Guide-1.101, Emergency Planning And Preparedness For Nuclear Power Reactors endorsing Numarc Nesp-007, Revision 2, 1/1992 Methodology For Development Of Emergency Action Levels.
- C. Sequoyah Technical Specifications (Tech Specs), Abnormal Operating Procedures (AOPS), Emergency Operating Procedures (EOPS), Functional Restoration Guidelines (FRG), Technical Instructions (TI), Surveillance Instructions (SI), and the Updated Final Safety Analysis Report (UFSAR) are also referenced in Appendix B of the Radiological Emergency Plan.

3.0 INSTRUCTIONS

3.1 The Nuclear Power (NP) Radiological Emergency Plan (REP) will be activated when any one of the conditions listed in this matrix is detected. This procedure will be used in conjunction with the Nuclear Power Radiological Emergency Plan, Appendix B.

If the event is determined to be one of the four emergency classifications then implement one of the following procedures as applicable:

- EPIP-2 Notification of Unusual Event
- EPIP-3 Alert
- EPIP-4 Site Area Emergency
- EPIP-5 General Emergency

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3.0 INSTRUCTIONS (Continued)

- **3.2** The criteria in SQN EPIP-1 are given for reference: knowledge of actual plant conditions or the extent of the emergency may require that additional steps be taken. In all cases, this logic procedure should be combined with the sound judgment of the SM or SED and/or the TSC SED to arrive at an appropriate classification for a particular set of circumstances. These criteria apply to both Unit 1 and Unit 2. The SED must be aware of the affects of simultaneous events on both units.
- **3.3** If there is a reason to doubt if a given initiating condition has actually occurred, the SM or SED shall follow indications provided. Unless a suspected spurious or otherwise false alarm can be substantiated within an acceptable time frame (based on potential severity of the event), the SM or SED is to proceed with actions as required by this procedure until such time as the alarm is verified to be false.

3.4 Classification Determination

- **3.4.1** To determine the classification of the emergency, review the Initiating Conditions of the respective status tree criteria that will be monitored and used to determine the event classification for the modes listed on the classification matrix.
- **3.4.2** If a Critical Safety Function (CSF) is listed as an Initiating Condition the respective status tree criteria will be monitored and used to determine the event classification for the modes listed on the classification matrix.
- **3.4.3** Declare the highest emergency class based on <u>events that are in progress</u> at the time that the classification is made. If follow-up investigation shows that a higher classification was met, then report that as information only, to the Operation Duty Specialist (ODS) and the NRC. Do not declare or upgrade to a higher emergency class if the conditions do not exist unless it is a noted exception.
- **3.4.4** Following termination of an emergency declaration, if follow-up investigations show that a higher classification was met then report that, as information only, to the ODS and the NRC. Do not declare or upgrade to a higher emergency class if the conditions do not exist.
- **3.4.5** During an event when plant conditions have returned to a <u>non-emergency state</u> before any emergency can be classified, then the highest emergency class that was appropriate shall be reported and <u>shall not</u> be declared unless it is a noted exception. If follow up investigations show that a higher classification was met then report that, as information only, to the ODS and the NRC. Do not declare or upgrade to a higher emergency class if the conditions do not exist.

3.4 Classification Determination (Continued)

- 3.4.6 The NRC shall be notified within one hour of all classifications. Once made and reported, a declaration cannot be canceled or rescinded even if it is later determined to be invalid. If there is reason to doubt that a given condition has occurred, the SM or SED shall follow indications and proceed with classification, as required by this procedure, until otherwise proven false. The State shall be notified within 15 minutes of the classification. If the State is notified of a declaration that is **invalidated before the NRC is notified**, terminate the classification, if not already done, and report the declaration to the NRC.
- **3.4.7** The **ACCEPTABLE** time frame for notification to the ODS is considered to be five (5) minutes. This is the time period between declaration of the emergency and notifying the ODS.
- 4.0 RECORDS

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4.1 QA Records

None.

4.2 Non-QA Records

None.

EMERGENCY PLAN CLASSIFICATION MATRIX

EPIP-1

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SQN	EMERGENCY PLAN
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EMERGENCY PLAN CLASSIFICATION MATRIX

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DEFINITIONS/ACRONYMS

BOMB: An explosive device. (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons within the EAB violently protesting onsite operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs; Subcriticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the initiating conditions associated with the event exist. Implicit in this definition is the need for timely assessment within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): That area surrounding the reactor, in which the reactor licenses has the authority to determine all activities including exclusion or removal of personnel and property from the area. For purposes of Emergency Action Levels, based on radiological field measurements and dose assessments, and for design calculations, the Site Boundary shall be defined as the EAB.

EXPLOSION: Rapid, violent, unconfined combustion, or a catastrophic failure of pressurized or electrical equipment that imparts energy of sufficient force to potentially damage permanent structures or equipment.

EXTORTION: An attempt to cause an action at the site by threat or force.

FAULTED: (Steam Generator) Existence of secondary side leakage (e.g., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

CIRE: Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical components do not constitute a fire. Deservation of flame is preferred but is NOT required if large quantities of smoke and/or heat are observed.

***LAMMABLE GAS:** Combustible gases at concentrations > than the LOWER EXPLOSIVE LIMIT (LEL).

HOSTAGE: A person(s) held as leverage against the site to ensure that femands will be met by the site.

IMMINENT: Within two hours.

NEFFECTIVE: When the specified restoration action(s) does not result in a reduction in the level of severity of the RED or ORANGE PATH condition within 15 minutes from identification of the CSF Status Tree RED or ORANGE PATH.

'NITIATION CONDITIONS: Plant Parameters, radiation monitor readings or versonnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in the protected area without authorization.

ODCM: Offsite Dose Calculation Manual is a supporting document to the Tech Specs. that contain Rad Effluent Controls, Environs Monitoring controls, and nethodology for calculating gaseous and liquid effluent offsite doses and nonitor alarm/trip setpoints.

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that he CSF(s) is under severe challenge; prompt operator action is required.

PROJECTILE: An object ejected, thrown or launched towards a plant structure resulting in damage sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein. The source of the projectile may be onsite or offsite.

PROTECTED AREA: The area encompassed by the security fence and to which access is controlled.

RCS: The **RCS** primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary and secondary isolation valves.

RED PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than the capacity one charging pump.

SABOTAGE: Deliberate damage, misalignment, or misoperation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback >15% thermal reactor power; (2) Electrical load rejection >25% full electrical load; (3) Reactor Trip; (4) Safety Injection System Activation; (5) Thermal Power Oscillations ≥10%.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine, CO_2 , etc.)

UNPLANNED: An event or action that is not the expected result of normal operations, testing or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED RELEASE: A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, (e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank).

VALID: An indication, report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indication on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analysis. Damage is sufficient to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, or paint blistering. Surface blemishes (e.g., paint chipping, scratches, etc.) should NOT be included as visible damage.

VITAL AREA: Any area within the PROTECTED AREA which contains equipment, systems, devices, or material which the failure, destruction, or release of, could directly or indirectly endanger the public health and safety by exposure to radiation.

MODES 1,2,3,4

FISSION PRODUCT BARRIER MATRIX

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1.1 Fuel Clad Barrier		1.2 RCS Barrier			
	1. Critical Safety Function Status		1. Critical Safety Function Status		
LOSS	Potential LOSS	LOSS	Potential LOSS		
Core Cooling Red	Core Cooling Orange	Not Applicable,	Pressurized Thermal		
(FR-C.1)	(FR-C.2)		Shock Red (FR-P.1).		
			OR		
	Heat Sink Red (RHR SD cooling not in service)		Heat Sink Red (RHR SD		
	(FR-H.1).		cooling not in service) (FR-H.1).		
<u>!</u>	(1.1(-11.1)).		(11(-11,1)).		
	DR-	-C	DR-		
, 2. Primary Coolant Act	ivity Level	2. RCS Leakage/LOCA			
LOSS	Potential LOSS	LOSS	Potential LOSS		
RCS sample activity is	Not Applicable.	RCS leak results in	Non isolatable RCS leak		
greater Than 300 μCi/gm		subcooling < 40 °F as	exceeding the capacity of		
dose equivalent lodine-131		indicated on XI-94-101 OR	one charging pump in the		
L		102 (EXOSENSOR).	normal charging		
	DR-		alignment.		
3. Incore TCs Hi Quad	Average		OR		
LOSS	Potential LOSS		RCS Leakage Results in		
Greater than 1200 °F on	Greater than or equal to		Entry Into E-1.		
XI-94-101 OR 102	700 °F on XI-94-101 or				
(EXOSENSOR).	102 (EXOSENSOR).				
		-C	PR-		
	R-	3. Steam Generator Tu	be Rupture		
4. Reactor Vessel Wate	····	LOSS	Potential LOSS		
LOSS	Potential LOSS	SGTR that results in a	Not Applicable.		
Not Applicable.	VALID RVLIS level < 40%	safety injection actuation.			
\ .	on LI-68-368 or 371 with no RCP running.	OR			
J.	no KCF funning.				
)R-	Entry into E-3.			
5. Containment Radiati					
LOSS	Potential LOSS		R-		
VALID reading of Greater	Not Applicable.	4. Reactor Vessel Wate			
Than:		LOSS	Potential LOSS		
ļ		VALID RVLIS level < 40%	Not Applicable.		
<u>2.8E + 01</u> Rem/hr On		on LI-68-368 or 371 with	Not Applicable.		
RM-90-271 and 272.		no RCP running.			
OR					
2.9E + 01 Rem/hr On					
RM-90-273 and 274.					
	DR-		R-		
6. Site Emergency Dire		5. Site Emergency Dire			
Any condition that, in the jud		Any condition that, in the jud			
indicates loss or potential lo	ss of the Fuel Clad Barrier	indicates loss or potential los			
comparable to the condition	s listed above.	comparable to the conditions			

EMERGENCY PLAN CLASSIFICATION MATRIX

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1.3 Contair	nment Barrier	INSTRUCTIONS
. Critical Safety Function	Status	
LOSS	Potential LOSS	NOTE: A condition is considered to be MET if, in
ot Applicable.	Containment Red	judgment of the Site Emergency Director
	(FR-Z.1)	condition will be MET imminently (i.e., w
		2 hours). The classification shall be made
	OR Actions of FD C.4 (Ded Dath)	soon as this determination is made.
	Actions of FR-C.1 (Red Path)	
	are INEFFECTIVE (i.e.: core	1. In the matrix to the left, REVIEW the Initiating Cond
	TC's trending up).	
		in all three barrier columns and CIRCLE the Conditi
-1	OR-	that are Met.
Containment Pressure/	Hydrogen	2. In each of the three barriers columns. IDENTIFY if a
LOSS	Potential LOSS	
apid unexplained pressure	Containment hydrogen	Loss or Potential Loss Initiating Conditions have be
ecrease following initial	increases to > 4% by volume	Met.
crease on PdI-30-44 or 45	on H_2 I-43-200 and 210.	
OR	OR	COMPARE the number of barrier Losses and Poter
ontainment pressure or	Pressure > 2.81 PSID (Phase	Losses to the Criteria below and make the appropria
imp level not increasing on		declaration.
-63-178 or 179 with a LOCA	B) with no containment spray operating when required	
		NOTE: MONITOR the respective status tree criteria if a
progress.	(FR-Z.1).	is listed as an Initiating Condition.
	DR-	
Containment Isolation S	Status	
LOSS	Potential LOSS	EMERGENCY CLASS CRITERIA
ontainment isolation, when	Not Applicable.	
quired, is incomplete and a		
lease path to the		GENERAL EMERGENCY
vironment exists.		
		LOSS of any two barriers and Potential
····	DR-	
	JR-	LOSS of third barrier.
Containment Bypass		
LOSS	Potential LOSS	
econdary side release	Unexpected VALID increase in	SITE AREA EMERGENCY
itside containment from a	area or ventilation RAD	
UPTURED S/G that cannot	monitors adjacent to	
e terminated in < 15 minutes	containment (with LOCA in	LOSS or Potential LOSS of any two
-2 and E-3).	progress).	barriers.
<u>OR</u>		
4 hours secondary side		
lease outside containment		ALERT
om a S/G with a S/G tube		
ak > T/S limits (AOP-R.01,		Any LOSS or Potential LOSS of Fuel Clad
op A).		barrier.
	DR-	OR
Significant Radioactivit		Any LOSS or Potential LOSS of RCS
LOSS	Potential LOSS	
ot Applicable.	VALID Reading of greater than:	barrier.
	<u>3.6 E + 02</u> Rem/hr on	
	RM-90-271 and RM-90-272.	
	OR	
	2.8 E + 02 Rem/hr on	UNUSUAL EVENT
	RM-90-273 and RM-90-274.	
		LOSS or Potential LOSS of Containment
	DR-	Barrier.
	r ludamont	
Site Emergency Directo		
Site Emergency Directo by condition that, in the judgm	ent of the SM or SED, indicates	
Site Emergency Directo by condition that, in the judgm		

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MODES 1,2,3,4

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FISSION PRODUCT BARRIER MATRIX

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END OF SECTION 1.

EMERGENCY PLAN CLASSIFICATION MATRIX

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 - 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

HAZARDS and SED JUDGEMENT

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5.2 Tornado

- 5.4 River Level High
- 5.5 River Level Low
- 5.3 Aircraft/Projectile 5.6 Watercraft Crash Table 5-1 Figure 5-A

SHUTDOWN SYSTEM DEGRADATION

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RADIOLOGICAL EFFLUENTS

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EMERGENCY PLAN CLASSIFICATION MATRIX

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VITAL AREA: Any area within the PROTECTED AREA which contains equipment, systems, devices, or material which the failure, destruction, or release of, could directly or indirectly endanger the public health and safety by exposure to radiation.

SYSTEM DEGRADATION

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	2.1 Loss of Instrumentation
Mode	Initiating / Condition
	Refer to "Fission Product Barrier Matrix" (Section 1) and "Radiological Effluents" (Section7) and Continue in This Column.
1,	On either unit an inability to monitor a SIGNIFICANT TRANSIENT in progress (1 and 2 and 3 and 4): 1.Loss of > 75% of MCR annunciators and the annunciator printer or > 75% of safety system indications.
2,	2. Loss of Plant Computer.
3,	3. Inability to directly monitor any of the following CSFs: Subcriticality PTS Core Cooling Containment Heat Sink Inventory
4	4. SIGNIFICANT TRANSIENT in progress.
1,	On either unit an UNPLANNED loss of >75% MCR annunciators and annunciator printer or > 75% of safety system indications for > 15 minutes with a SIGNIFICANT TRANSIENT in progress or plant computer unavailable. <i>(1 and 2 and 3):</i> 1. UNPLANNED loss of >75% MCR annunciators and the annunciator printer for >15 minutes or > 75% of safety
	system indications for > 15 minutes. 2. SM/SED judgment that increased surveillance is
3,	required (> shift compliment) to safely operate the unit. 3. (a or b)
4	a. SIGNIFICANT TRANSIENT in progress. <u>OR</u> b. Loss of plant computer.
	On either unit an UNPLANNED loss > 75% MCR
1,	annunciators and annunciator printer or > 75% of safety system indications for > 15 minutes and plant computer available. <i>(1 and 2 and 3):</i>
2,	 UNPLANNED loss of >75% of MCR annunciators and the annunciator printer for > 15 minutes or > 75% of safety system indicators for > 15 minutes.
3,	 SM/SED judgment that increased surveillance is required (> shift compliment) to safely operate the unit.
4	 The plant computer is capable of displaying requested data.
	1, 2, 3, 4 1, 2, 3, 4 1, 2, 3, 4

2.2 Loss of Communication			
Mode	Initiating / Condition		
	Not Applicable.		
	Not Applicable.		
	Not Applicable.		
A L L	 A. UNPLANNED loss of all in-plant communication capability (1 and 2 and 3): 1. UNPLANNED loss of EPABX phones. 2. UNPLANNED loss of all sound powered phones. 3. UNPLANNED loss of all radios. OR B. UNPLANNED loss of all offsite communication capability (1 and 2 and 3 and 4 and 5): 1. UNPLANNED loss of all EPABX phones 2. UNPLANNED loss of all CPABX phones 3. UNPLANNED loss of all PABX phones 4. UNPLANNED loss of all OPX (Microwave)system 4. UNPLANNED loss of all 1-FB-Bell lines 5. UNPLANNED loss of all FTS 2000 (NRC) system 		

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SYSTEM DEGRADATION

		2.3 Failure of Rx Protection		2.4 Fuel Clad Degradation	
	Mode	Initiating / Condition	Mode	Initiating / Condition	
GENERAL EMER	1	Reactor power > 5% and not decreasing after VALID trip signals and loss of core cooling capability. (1 and 2): 1. FR-S.1 entered and immediate operator actions did not result in a reactor power of ≤ 5% and decreasing. 2. (a or b) a. CSF status tree indicates Core Cooling Red (FR-C.1). OR b. CSF status tree indicates Heat Sink Red (FR-H.1)		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	
SITE AREA EMER	1	Reactor power > 5% and not decreasing after VALID auto and manual trip signals. NOTE: Although a mode change may occur before classification this event will still be classified and declared as SAE.		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	
A L E R T	1, 2,	 Reactor power > 5% and not decreasing after VALID auto trip signal but a manual trip from the Control Room is successful. (1 or 2) 1. Reactor power > 5% and not decreasing following auto trip signal. 2. Manual trip in the Main Control Room successfully reduces reactor power ≤ 5%. NOTE: Although a mode change will occur this event will still be classified and declared as an ALERT. 		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	
U N U S U A L E V E N T		Refer to "Fission Product Barrier Matrix" (Section 1).	1, 2, 3	Reactor coolant system specific activity exceeds LCO (Refer to SQN Tech. Spec. 3.4.8): 1. Radiochemistry analysis indicates (a or b): a. Dose equivalent lodine (I-131) >0.35 μCi/gm for > 48 hours or in excess of T/S Figure 3.4-1 with Tave ≥ 500 °F. DR b. Specific activity > 100/É μCi/gm with Tave ≥ 500 °F.	1

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Mode	.5 RCS Unidentified Leakage Initiating / Condition	Mode	2.6 RCS Identified Leakage
NIDGe	Refer to "Fission Product Barrier Matrix" (Section 1) and	Mode	Initiating / Condition
	Continue in This Column.		Refer to "Fission Product Barrier Matrix" (Section 1) an Continue in This Column.
	Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.		Refer to "Fission Product Barrier Matrix" (Section 1) ar Continue in This Column.
	Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
1,	 RCS unidentified or pressure boundary leakage > 10 GPM. 1. Unidentified or pressure boundary leakage (as defined by Tech. Spec.) > 10 GPM as indicated by (a or b); 	1,	 RCS Identified leakage > 25 GPM. 1. Identified RCS leakage (as defined by Tech. Spec.) > 25 GPM as indicated by (a or b):
2,	a. SI-OPS-068-137.0 results.	2,	a. SI-OPS-068-137.0 results.
2			OR
3, 4	<u>OR</u> b. With RCS temperature and PZR level stable, the VCT level on LI-62-129 or LI-62-130 is dropping at a rate > 10 GPM.	3, 4	 b. Level rise in excess of 25 GPM into PRT, RCDT of CVCS holdup tank (Refer to TI-28).
	Refer to "Shutdown Systems Degradation" (Section 6.3).		Refer to "Shutdown Systems Degradation" (Section 6.3).

SYSTEM DEGRADATION

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		2.7 Uncontrolled Cooldown		2.8 Turbine Failure
	Mode	Initiating / Condition	Mode	Initiating / Condition
GENERAL EMER		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
SITE AREA EMER		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
A L E R T		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	1, 2, 3	Turbine failure has generated projectiles that cause visible damage to any area containing safety related equipment. 1. Turbine generated PROJECTILES have resulted in VISIBLE DAMAGE to any of the following areas: Control Building Diesel Generator Bldg. Auxiliary Building RWST Unit #1 Containment Intake Pumping Station Unit #2 Containment Common Sta. Ser. Xfmr's ERCW Pumping Station Condensate Storage Tanks Additional Equipment Bldgs. Additional Equipment Bldgs.
UNUSUAL E	1, 2, 3	 UNPLANNED rapid depressurization of the main steam system resulting in a rapid RCS cooldown and safety injection initiation. (1 and 2): 1. Rapid depressurization of any or all steam generators or the main steam system to < 600 psig on PI-1-2A, 2B or 9A, 9B or 20A, 20B or 27A, 27B. 2. Safety injection has initiated or is required. 	1, 2, 3	 Turbine failure results in casing penetration or main generator seal damage. 1. Turbine failure which results in penetration of the turbine casing or damage to main generator seals.
L V E N T				Refer to "Hazards and SED Judgement" (Section 4.3)

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		2.9 Safety Limit
	Mode	
G E N E R A L E		Not Applicable.
M E R		
S I T E		Not Applicable.
A R E A E		
M E R		Not Applicable
		Not Applicable.
A L R T		-
U N U S	1,	 Safety Limits have been exceeded. (1 or 2): 1. The combination of thermal power, RCS temperature and RCS pressure > safety limit indicated by SQN Tech Specific and 2.4.4 (Beater Core Sofeth Limit)
U A	2,	Tech. Spec. Figure 2.1-1 "Reactor Core Safety Limit". <u>QR</u>
L E	3,	 RCS/Pressurizer pressure exceeds safety limit (> 2735 psig).
	4	

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END OF SECTION 2.

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DEFINITIONS/ACRONYMS

BOMB: An explosive device. (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons within the EAB violently protesting onsite operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs; Subcriticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the initiating conditions associated with the event exist. Implicit in this definition is the need for timely assessment within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): That area surrounding the reactor, in which the reactor licenses has the authority to determine all activities including acclusion or removal of personnel and property from the area. For purposes of Emergency Action Levels, based on radiological field measurements and dose assessments, and for design calculations, the Site Boundary shall be defined as the EAB.

EXPLOSION: Rapid, violent, unconfined combustion, or a catastrophic failure of pressurized or electrical equipment that imparts energy of sufficient force to ² potentially damage permanent structures or equipment.

EXTORTION: An attempt to cause an action at the site by threat or force.

FAULTED: (Steam Generator) Existence of secondary side leakage (e.g., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Sources of smoke such as slipping drive betts or overheated electrical components do not constitute a fire. Deservation of flame is preferred but is NOT required if large quantities of smoke and/or heat are observed.

FLAMMABLE GAS: Combustible gases at concentrations > than the LOWER EXPLOSIVE LIMIT (LEL).

HOSTAGE: A person(s) held as leverage against the site to ensure that demands will be met by the site.

IMMINENT: Within two hours.

INEFFECTIVE: When the specified restoration action(s) does not result in a reduction in the level of severity of the RED or ORANGE PATH condition within 15 minutes from identification of the CSF Status Tree RED or ORANGE PATH.

INITIATION CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan --Classification.

 INTRUSION/INTRUDER: Suspected hostile individual present in the protected area without authorization.

ODCM: Offsite Dose Calculation Manual is a supporting document to the Tech Specs. that contain Rad Effluent Controls, Environs Monitoring controls, and methodology for calculating gaseous and liquid effluent offsite doses and monitor alarm/trip setpoints.

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge; prompt operator action is required.

PROJECTILE: An object ejected, thrown or launched towards a plant structure resulting in damage sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein. The source of the projectile may be onsite or offsite.

PROTECTED AREA: The area encompassed by the security fence and to which access is controlled.

RCS: The RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary and secondary isolation valves.

RED PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than the capacity one charging pump.

SABOTAGE: Deliberate damage, misalignment, or misoperation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: AN UNPLANNED event involving one or more of the following: (1) An automatic turbine runback >15% thermal reactor power; (2) Electrical load rejection >25% full electrical load; (3) Reactor Trip; (4) Safety Injection System Activation; (5) Thermal Power Oscillations ≥10%.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine, CO_2 , etc.)

UNPLANNED: An event or action that is not the expected result of normal operations, testing or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED RELEASE: A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, (e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank).

VALID: An indication, report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indication on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analysis. Damage is sufficient to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, or paint blistering. Surface blemishes (e.g., paint chipping, scratches, etc.) should NOT be included as visible damage.

VITAL AREA: Any area within the PROTECTED AREA which contains equipment, systems, devices, or material which the failure, destruction, or release of, could directly or indirectly endanger the public health and safety by exposure to radiation.

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LOSS OF POWER

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		3.1 Loss of AC (Power Ops)		3.2
	Mode	Initiating / Condition	Mode	
G E N E R A	1, 2,	 Prolonged loss of all offsite and all onsite AC power to either unit. (1 and 2): 1. Both unit related 6.9 KV shutdown boards de-energized for > 15 minutes. 2. (a or b) 2. Care Cooling Status Two Ded to Ocean D the 		Not A
E M E R	3, 4	 a. Core Cooling Status Tree Red or Orange Path. <u>OR</u> b. Restoration of either a 6.9 KV shutdown board or a 6.9 KV unit board is not likely within 4 hours of the loss. 		
SITE AREA EMER	1, 2, 3, 4	Loss of all offsite and all onsite AC power to either unit for > 15 Minutes. 1. Both unit related 6.9 KV shutdown boards de-energized for > 15 minutes.		Not A
A L E R T	1, 2, 3, 4	 Loss of offsite power to either unit with degraded onsite AC power for > 15 minutes. (1a and b or 2): 1a. All four (4) 6.9KV unit boards de-energized for > 15 minutes. b. One (1) unit related 6.9 KV shutdown board de-energized for > 15 minutes. DR 2. Any AC power condition lasting > 15 minutes where a single additional failure will result in a unit blackout. 	5, 6, D E F U E L E D	UNPL powe 1. Bot fo
U N U S U A L E V E N T	1, 2, 3, 4	 Loss of offsite power to either unit for > 15 minutes. (1 and 2): 1. All four (4) 6.9KV unit boards de-energized for > 15 minutes. 2. Both unit related 6.9KV shutdown boards are energized. 	5, 6, D E F U E L E D	UNPL for > 1 1. All f 2. One en

	2.21 and of AC (Churddown)
Mode	3.2 Loss of AC (Shutdown) Initiating / Condition
	Not Applicable.
	Not Applicable.
5,	UNPLANNED loss of all offsite and all onsite AC power to either unit for > 15 minutes.
6,	
D	 Both unit related 6.9KV shutdown boards de-energized for > 15 minutes,
E	
F	
U	
Б L	
E	
D	Also Refer to "Loss of Shutdown Systems" (6.1) and continue in this column.
5,	UNPLANNED loss of all offsite power to either unit
6,	for > 15 minutes. <i>(1 and 2):</i>
D	1. All four (4) 6.9KV unit boards de-energized for
E	> 15 minutes.
F	 One (1) unit related 6.9KV shutdown board de- energized for > 15 minutes.
U	storgeou for a formules.
Ε	
L	
Е	
D	

END OF SECTION 3.

		3.3 Loss of DC Power
	Mode	
GENERAL EMER		Refer to "Fission Product Barrier Matrix" (Section 1) and "Loss of Communication" (2.2) and Continue in This Column.
S		Loss of all vital DC power for > 15 minutes.
ί Ι μ Τ Ε	1,	
A R	2,	1. Voltage < 105 V DC on 125V DC vital battery board buses I <u>and</u> II <u>and</u> III <u>and</u> IV for > 15 minutes.
E A	3,	
E M E S R	4	Also Refer to "Fission Product Barrier Matrix" (Section 1), "Loss of Communication" (2.2) and, "Loss of Instrumentation" (2.1) and Continue in This Column.
		Refer to "Fission Product Barrier Matrix" (Section 1), "Loss of Communication" (2.2), and "Loss of
		Instrumentation" (2.1).
A		
L		-
E R		
T		
U		UNPLANNED loss of a required train of DC power for
Ν		> 15 minutes: (1 or 2).
U S U	5,	 Voltage < 105 V DC on 125V dc vital battery board buses I and III for > 15 minutes.
А	6	OR
		 Voltage < 105 V DC on 125V dc vital battery board busses II and IV for > 15 minutes.
E V		
E V E N T		

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- 2.8 Turbine Failure
- 2.9 Safety Limit

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IMMINENT: Within two hours.

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VITAL AREA: Any area within the PROTECTED AREA which contains equipment, systems, devices, or material which the failure, destruction, or release of, could directly or indirectly endanger the public health and safety by exposure to radiation.

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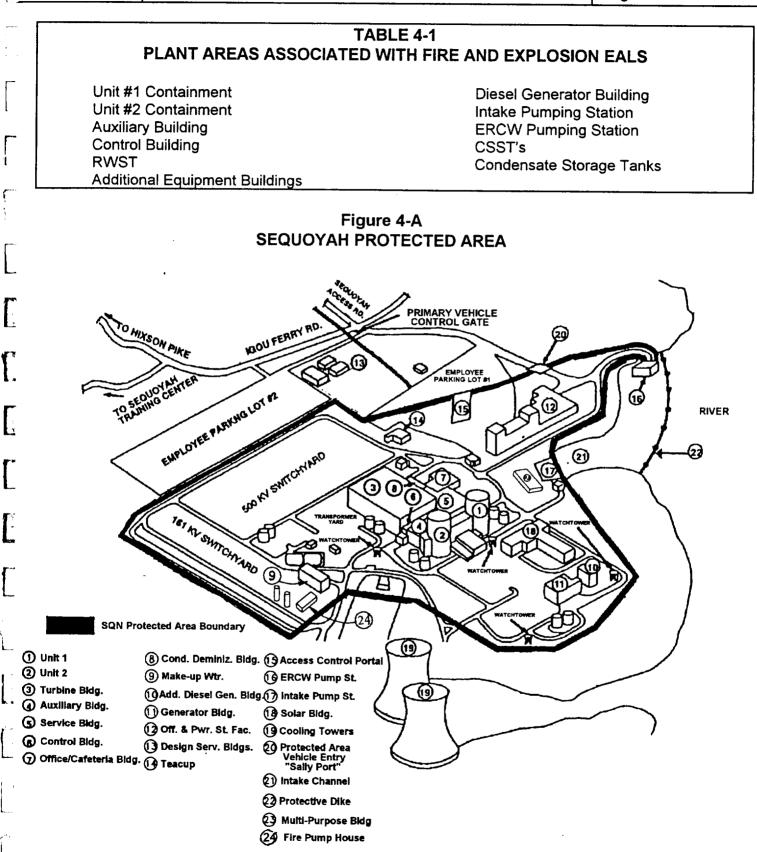
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24	4.1 Fire			4.2 Explosions		
	Mode	Initiating / Condition		Mode	Initiating / Condition	
G E N E R A L E M E R		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.			Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	
SITE AREA EMER		Refer to "Control Room Evacuation," (4.5) and Fission Product Barrier Matrix" (Section 1) and Continue in This Column.			Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	
A L E R T	A L L	 FIRE in any of the areas listed in Table 4-1 that is affecting safety related equipment required to establish or maintain safe shutdown. (1 and 2): 1. FIRE in any of the areas listed in Table 4-1. 2. (a or b) a. VISIBLE DAMAGE to permanent structure or safety related equipment in the specified area is observed due to the FIRE. DR b. Control room indication of degraded safety system or component response due to the FIRE. 		A L L	 EXPLOSION in any of the areas listed in Table 4-1 that is affecting safety related equipment required to establish or maintain safe shutdown. (1 and 2): 1. EXPLOSION in any of the areas listed in Table 4-1. 2. (a or b) a. VISIBLE DAMAGE to permanent structures or to safety related equipment in the specified area is due to the EXPLOSION. <u>OR</u> b. Control room indication of degraded safety system or component response due to the EXPLOSION. <i>Refer to "Security" (Section 4.6).</i> 	
U N U S U A L E V E N T	A L L	FIRE within the PROTECTED AREA (Figure 4-A) threatening any of the areas listed in Table 4-1 that is not extinguished within 15 minutes from the time of control room notification or verification of control room alarm.		A L L	UNPLANNED EXPLOSION within the PROTECTED AREA (Figure 4-A) resulting in VISIBLE DAMAGE to any permanent structure <u>or</u> equipment. <i>Refer to "Security" (Section 4.6).</i>	

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HAZARDS AND SED JUDGEMENT

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HAZARDS AND SED JUDGEMENT

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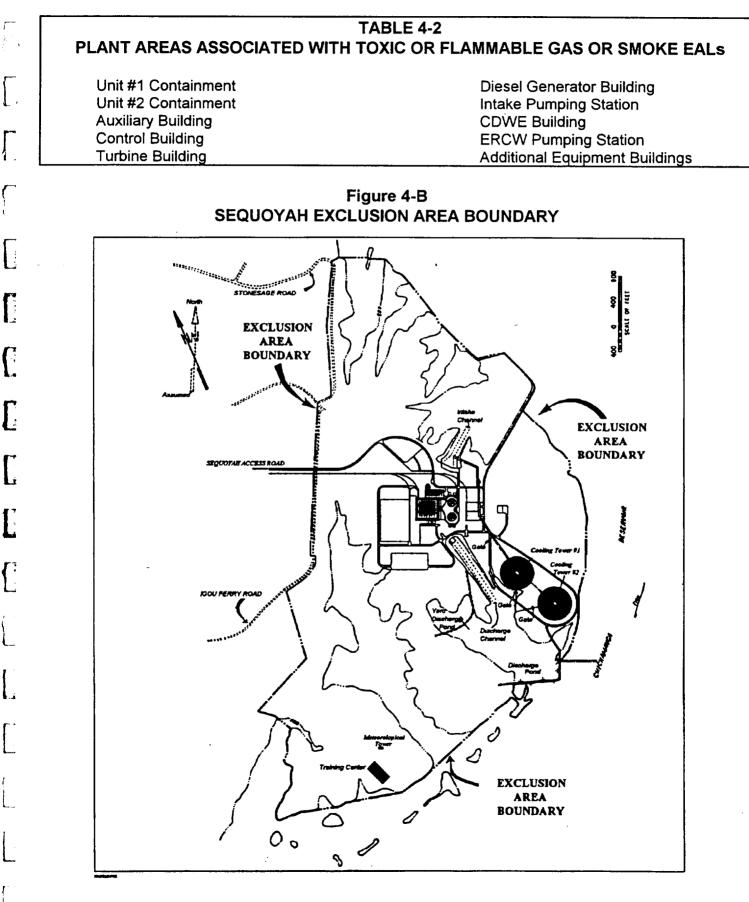
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		4.3 Flammable Gas		4
4	Mode	Initiating / Condition	Mode	
G E N E R A L E M E R		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.		Rei Coi
SITE AREA EMER		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.		Ref Cor
ALERT	A L L	 UNPLANNED release of FLAMMABLE GAS within a facility structure containing safety related equipment or associated with safe operation of the plant. 1. Plant personnel report the average of three (3) readings taken in an ~10 ft. Triangular Area is > 25% Lower Explosive Limit, as indicated on the monitoring instrument within any building listed in Table 4-2. Refer to the MSDS for the LEL. 	A L L	Refe stru requ (1 a 1. P 2. (a 3. P Refe
ÙNUSUAL EVENT	A L L	 A. UNPLANNED release of FLAMMABLE GAS within the EXCLUSION AREA BOUNDARY that may affect normal operations. Plant personnel report the average of three readings taken in an ~10 ft. Triangular Area is > 25% of the Lower Explosive Limit, as indicated on the monitoring instrument within the EXCLUSION AREA BOUNDARY (Figure 4-B). <u>QR</u> B. Confirmed report by Local, County, or State officials that a large offsite FLAMMABLE GAS release has occurred within one (1) mile of the site (Figure 4-C) with potential to enter the EXCLUSION AREA BOUNDARY (Figure 4-B) in concentrations > 25% of Lower Explosive Limit: (Refer to the MSDS for the LEL). 	A L L	B. C.

	4.4 Toxic Gas or Smoke
Mode	Initiating / Condition
	Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
	Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
A L L	 Release of TOXIC GAS or smoke within a facility structure which prohibits safe operation of systems required to establish or maintain Cold S/D. (1 and 2 and 3): Plant personnel report TOXIC GAS or smoke within any building listed in Table 4-2. (a or b) a. Plant personnel report severe adverse health reactions due to TOXIC GAS or smoke (i.e., burning eyes, nose, throat, dizziness). <u>OR</u> b. Sampling indication > Permissible Exposure Limit (PEL). Plant personnel unable to perform actions to establish and maintain Cold Shutdown while utilizing appropriate personnel protection equipment. Refer to the MSDS for the PEL.
A L L	A. Safe operations impeded due to access restrictions caused by TOXIC GAS or smoke concentrations within a facility structure listed in Table 4-2. B. Confirmed report by Local, County, or State officials that an offsite TOXIC GAS release has occurred within one (1) mile of the site (Figure 4- C) with potential to enter the EXCLUSION AREA BOUNDARY (Figure 4-B) in concentrations > the Permissible Exposure Limit (PEL) causing a site evacuation. (Refer to the MSDS for the PEL).

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HAZARDS AND SED JUDGEMENT

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	4.5 Control Room Evacuation	·.	4.6 Security
Mode		Mode	Initiating / Condition
G E N E R A L E M E R	Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	A L L	 Security event resulting in loss of control of the plant. Hostile armed force has taken control of the plant or control room or remote shutdown capacity.
SI TE ARA EAL EM E	 Evacuation of the control room has been initiated and control of all necessary equipment has not been established within 15 minutes of staffing the auxiliary control room. (1 and 2): 1. AOP-C.04 "Control Room Inaccessibility" entered. 2. Control has not been established within 15 minutes of staffing the auxiliary control room and completing transfer of switches, listed on checklist AOP-C.04-1, on panels L11A and L11B to the AUX position. 	A L L	 Security event has or is occurring which results in actual or likely failures of plant functions needed to protect the public. 1. VITAL AREA, other than the control room, has been penetrated by a hostile armed force.
R A L E R T	Evacuation of the Control Room is Required. 1. AOP-C.04 "Control Room Inaccessibility" has been entered	A L L	Confirmed security event which indicates an actual or potential substantial degradation in the level of safety of the plant. (1 or 2 or 3): 1. BOMB discovered within a VITAL AREA. OR 2. CIVIL DISTURBANCE ongoing within the PROTECTED AREA (Figure 4-A). OR 3. PROTECTED AREA (Figure 4-A) has been penetrated by a hostile armed force.
U N U S U A L E V E N T	Not Applicable.	A L L	Confirmed security event which indicates a potential degradation in the level of safety of the plant. <i>(1 or 2)</i> 1. BOMB discovered within the PROTECTED AREA (Figure 4-A). <u>OR</u> 2. Security Shift Supervisor reports any of the events listed in Table 4-3.

HAZARDS AND SED JUDGEMENT

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	17	Emorgonov Director Judgement
	Mode	Emergency Director Judgement
G E N E R A L E M E R	A L L	Events are in process or have occurred which involve Actual or Imminent Substantial Core Degradation or Melting With Potential for Loss of Containment Integrity. Releases can be reasonably expected to exceed EPA Plume Protective Action Guidelines Exposure Levels outside the EXCLUSION AREA BOUNDARY (Figure 4-B)
S I T E A R E A R E A R E R E R	A L L	Events are in process or have occurred which involve Actual or Likely Major Failures of Plant Functions needed for the Protection of the Public. Any releases are not expected to result in Exposure Levels which Exceed EPA Plume Protective Action Guideline Exposure Levels outside the EXCLUSION AREA BOUNDARY (Figure 4-B)
A L E R T	A L L	Events are in process or have occurred which involve an Actual or Potential Substantial Degradation of the Level of Safety of the Plant. Any releases are expected to be limited to small fractions of the EPA Plume Protective Action Guideline Exposure Levels.
U N U S U A L E V E N T	A L L	Events are in Process or have occurred which indicate a Potential Degradation of the Level of Safety of the Plant. No releases of Radioactive Material requiring Offsite Response or Monitoring are expected unless further degradation of Safety System occurs.

TABLE 4-3 SECURITY EVENT EXAMPLES a. SABOTAGE/INTRUSION has occurred or is occurring within the PROTECTED AREA (Figure 4-A). b. HOSTAGE/EXTORTION situation that threatens to interrupt plant operations.

- c. CIVIL DISTURBANCE ongoing between the EXCLUSION AREA BOUNDARY (Figure 4-B) and the PROTECTED AREA (Figure 4-A).
- d. Hostile STRIKE ACTION within the PROTECTED AREA which threatens to interrupt normal plant operations (judgement based on behavior of strikers and/or intelligence received).

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Skull Islan 1 Mile 0 Access Re And RR Sequoyah Nuclear Plant L 0 C U Ferry Τ TENNESSEE R RIVER Biwe Spring: **Chigger** Point 00 CHICKAMAUGA LAKE

Figure 4-C SEQUOYAH ONE MILE RADIUS

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EMERGENCY PLAN CLASSIFICATION MATRIX

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-ISSION PRODUCT BARRIER MATRIX (Modes 1-4)

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- 1.2 RCS Barrier
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- 7.1 Gaseous Effluent
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EMERGENCY PLAN CLASSIFICATION MATRIX

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DEFINITIONS/ACRONYMS

BOMB: An explosive device. (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons within the EAB violently protesting onsite operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs; Subcriticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the initiating conditions associated with the event exist. Implicit in this definition is the need for timely assessment within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): That area surrounding the reactor, in which the reactor licenses has the authority to determine all activities including exclusion or removal of personnel and property from the area. For purposes of Emergency Action Levels, based on radiological field measurements and dose assessments, and for design calculations, the Site Boundary shall be defined as the EAB.

EXPLOSION: Rapid, violent, unconfined combustion, or a catastrophic failure of pressurized or electrical equipment that imparts energy of sufficient force to potentially damage permanent structures or equipment.

EXTORTION: An attempt to cause an action at the site by threat or force.

.: **AULTED:** (Steam Generator) Existence of secondary side leakage (e.g., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical components do not constitute a fire. Observation of flame is preferred but is NOT required if large quantities of smoke and/or heat are observed.

FLAMMABLE GAS: Combustible gases at concentrations > than the LOWER EXPLOSIVE LIMIT (LEL).

'IOSTAGE: A person(s) held as leverage against the site to ensure that Jemands will be met by the site.

!MMINENT: Within two hours.

...**NEFFECTIVE:** When the specified restoration action(s) does not result in a reduction in the level of severity of the RED or ORANGE PATH condition within 15 minutes from identification of the CSF Status Tree RED or ORANGE PATH.

INITIATION CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

NTRUSION/INTRUDER: Suspected hostile individual present in the protected area without authorization.

ODCM: Offsite Dose Calculation Manual is a supporting document to the Tech Specs. that contain Rad Effluent Controls, Environs Monitoring controls, and methodology for calculating gaseous and liquid effluent offsite doses and monitor alarm/trip setpoints.

DRANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that ... he CSF(s) is under severe challenge; prompt operator action is required.

PROJECTILE: An object ejected, thrown or launched towards a plant structure resulting in damage sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein. The source of the projectile may be onsite or offsite.

PROTECTED AREA: The area encompassed by the security fence and to which access is controlled.

RCS: The **RCS** primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary and secondary isolation valves.

RED PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than the capacity one charging pump.

SABOTAGE: Deliberate damage, misalignment, or misoperation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback >15% thermal reactor power; (2) Electrical load rejection >25% full electrical load; (3) Reactor Trip; (4) Safety Injection System Activation; (5) Thermal Power Oscillations ≥10%.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine, CO_2 , etc.)

UNPLANNED: An event or action that is not the expected result of normal operations, testing or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED RELEASE: A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, (e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank).

VALID: An indication, report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indication on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analysis. Damage is sufficient to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, or paint blistering. Surface blemishes (e.g., paint chipping, scratches, etc.) should NOT be included as visible damage.

VITAL AREA: Any area within the PROTECTED AREA which contains equipment, systems, devices, or material which the failure, destruction, or release of, could directly or indirectly endanger the public health and safety by exposure to radiation.

DESTRUCTIVE PHENOMENON

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		5.1 Earthquake			
	Mode	Initiating / Condition]	Mode	
GENERAL EME		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.			Refer to *F (Section 1,
R		· · ·			
S I T E A R E A R E A E M E R		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.			Refer to "F (Section 1)
A L E R T	A L L	 Earthquake detected by site seismic instrumentation. (1 and 2): 1. Panel XA-55-15B alarm window 30 (E-2) plus window 22 (D-1) activated. 2. (a or b) a. Ground motion sensed by plant personnel. <u>OR</u> b. National Earthquake Information Center at 1-(303) 273-8500 can confirm the event. 		A L L	Tornado o Table 5-1 a 1. Tornado minute listed in 2. (a or b) a. Co b. Co sys Note: Nate 1-(4
U N U S U A L E V E N T	A L L	Earthquake detected by site seismic instruments. (1 and 2): 1. Panel XA-55-15B alarm window 22 (D-1) activated. 2. (a or b) a. Ground motion sensed by plant personnel. <u>OR</u> b. National Earthquake Information Center at 1-(303) 273-8500 can confirm the event.		A L L	Tornado w 1. Plant p within (Figure

	5.2 Tornado	
Mode	Initiating / Condition	
	Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	
	Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	
	Tornado or high winds strikes any structure listed in Table 5-1 and results in VISIBLE DAMAGE. (1 and 2):	
A L	 Tornado or high winds (sustained >80 m.p.h. > one minute on the plant computer) strikes any structure listed in Table 5-1. 	1
L	 (a or b) a. Confirmed report of any VISIBLE DAMAGE. <u>OR</u> 	
	 b. Control room indications of degraded safety system or component response due to event. 	1
	Note: National Weather Service Morristown 1-(423)-586-8400, can provide additional info.	
	Tornado within the EXCLUSION AREA BOUNDARY.	
A L L	 Plant personnel report a tornado has been sighted within the EXCLUSION AREA BOUNDARY (Figure 5-A) 	
		1

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		5.3 Aircraft/Projectile Impact		
	Mode			
G E		Refer to the "Fission Product Barrier Matrix" (Section 1).		
N E				
R			TA	BLE 5-1
A L				e Associated W
E Vi			Unit #1 Containment	Auxiliary Building
E R			Unit #2 Containment	Diesel Generator
			Control Building	ERCW Pumping
S I		Refer to the "Fission Product Barrier Matrix" (Section 1).	Intake Pumping Station	Turbine Building
T E			CDWE Building	Common Station Transformer's
Ą			RWST	Condensate Stor
R E			Additional Equipment Bldgs	
/I E R		Aircraft or PROJECTILE impacts (strikes) any plant		
		structure listed in Table 5-1 resulting in VISIBLE DAMAGE. <i>(1 and 2):</i>		
	А	 Plant personnel report aircraft or PROJECTILE has impacted any structure listed in Table 5-1. 		
	L	2. (a or b) a. Confirmed report of VISIBLE DAMAGE.		
	L			
		 b. Control Room indications of degraded safety system or component response due to the event within any structure listed in Table 5-1. 		
		Aircraft crash or projectile impact (strikes) within the EXCLUSION AREA BOUNDARY.		
J S J		1. Plant personnel report aircraft crash or PROJECTILE impact within the EXCLUSION AREA BOUNDARY (Figure 5-A).		
	A			
	L			
	-			
	-			
	_			

DESTRUCTIVE PHENOMENON

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1			5.4 River Level HIGH		5.5 River Level LOW
j		Mode	Initiating / Condition	Mode	
	G		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.	1	Refer to "Fission Product Barrier Matrix"
	E N				(Section 1) and Continue in This Column.
	E				
	R A				
	Ĺ				
	F				
	E M				
	Е				
	R				
	S		Refer to "Fission Product Barrier Matrix"	1	Refer to "Fission Product Barrier Matrix"
1	l T		(Section 1) and Continue in This Column.		(Section 1) and Continue in This Column.
	E				
	^				
	A R				
	Ε				
í.	A				
	E M				
	M E				
*	R				
ļ.			River reservoir level is at Stage II Flood Warning as		River reservoir level is < 670 Feet as reported by
			reported by River Operations.		River Operations.
	А				
	L	A		A	
	E R		-		
	T	L			
-					
	U N		River reservoir level is at Stage I Flood Warning as reported by River Operations.		River reservoir level is < 673 Feet as reported by River Operations.
	U				
	S U				
	A L				
		L			
	E				
1	V E				
	N				
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		E.C. Watercraft Creek
	Mode	5.6 Watercraft Crash
GENERAL EMER		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
S I T E A R E A E M E R		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
A L R T		Refer to "Fission Product Barrier Matrix" (Section 1) and Continue in This Column.
U N U S U A L E V E N T	A L L	 Watercraft strikes the ERCW pumping station resulting in a reduction of Essential Raw Cooling Water (ERCW). (1 and 2): 1. Plant personnel report a watercraft has struck the ERCW pumping station. 2. (a or b) a. ERCW supply header pressure Train A 1(2)-PI-67-493A is < 15 psig. DR b. ERCW supply header pressure Train B 1(2)-PI-67-488A is < 15 psig.

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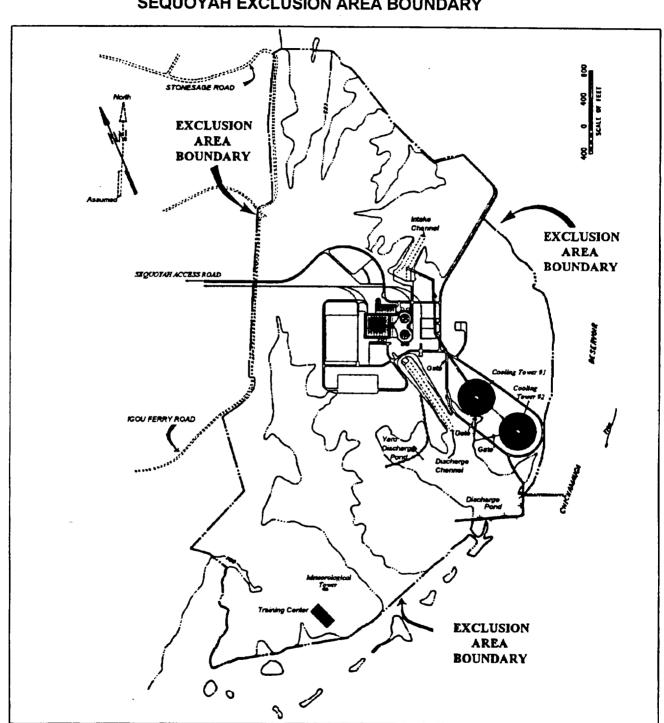


Figure 5-A SEQUOYAH EXCLUSION AREA BOUNDARY

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END OF SECTION 5

EMERGENCY PLAN CLASSIFICATION MATRIX

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- 1.3 Containment Barrier

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- 2.7 Uncontrolled Cool Down
- 2.3 Failure of Reactor Protection
- 2.4 Fuel Clad Degradation
- 2.5 RCS Unidentified Leakage

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- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

HAZARDS and SED JUDGEMENT

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- 4.3 Flammable Gas
- 4.2 Explosion
- 4.4 Toxic Gas or Smoke
- Table 4-1
- Table 4-2 Figure 4-B

- Figure 4-A
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- 5.2 Tornado
- 5.3 Aircraft/Projectile Table 5-1

5.4 River Level High 5.5 River Level Low

5.6 Watercraft Crash Figure 5-A

SHUTDOWN SYSTEM DEGRADATION

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- 3.2 Loss of Shutdown Capability
- 3.3 Loss of RCS Inventory

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- 7.2 Liquid Effluent Table 7-1 Figure 7-A
- 7.3 Radiation Levels
- 7.4 Fuel Handling Table 7-2

- 4.5 Control Room Evacuation
- 4.6 Security
- 4.7 SED Judament Table 4-3 Figure 4-C

- 2.8 Turbine Failure

- 2.9 Safety Limit

EMERGENCY PLAN CLASSIFICATION MATRIX

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DEFINITIONS/ACRONYMS

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IOSTAGE: A person(s) held as leverage against the site to ensure that Jemands will be met by the site.

IMMINENT: Within two hours.

NEFFECTIVE: When the specified restoration action(s) does not result in a reduction in the level of severity of the RED or ORANGE PATH condition within 15 minutes from identification of the CSF Status Tree RED or ORANGE PATH.

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PROTECTED AREA: The area encompassed by the security fence and to which access is controlled.

RCS: The RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary and secondary isolation valves.

RED PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than the capacity one charging pump.

SABOTAGE: Deliberate damage, misalignment, or misoperation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback >15% thermal reactor power; (2) Electrical load rejection >25% full electrical load; (3) Reactor Trip; (4) Safety Injection System Activation; (5) Thermal Power Oscillations ≥10%.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine, CO_2 , etc.)

UNPLANNED: An event or action that is not the expected result of normal operations, testing or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED RELEASE: A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, (e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank).

VALID: An indication, report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indication on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analysis. Damage is sufficient to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, or paint blistering. Surface blemishes (e.g., paint chipping, scratches, etc.) should NOT be included as visible damage.

VITAL AREA: Any area within the PROTECTED AREA which contains equipment, systems, devices, or material which the failure, destruction, or release of, could directly or indirectly endanger the public health and safety by exposure to radiation.

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SHUTDOWN SYSTEM DEGRADATION

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	6	.1 Loss of Shutdown Systems		6.2 Loss of S/D Capability
	Mode	Initiating / Condition	Mode	Initiating / Condition
G E N E R A L		Refer to "Gaseous Effluents" (Section 7.1) and Continue in This Column.		Not Applicable.
E M R R				
S I T E		Loss of water level in the reactor vessel that has or will uncover active fuel in the reactor vessel with containment closure established. (1 and 2 and 3 and 4):	1,	Complete loss of function needed to achieve or maintain hot shutdown. <i>(1 and 2a or 2b):</i> 1. Hot shutdown required.
А	5,	 Loss of RHR capability. VALID indication that reactor vessel water level 	2,	2a. CSF status tree indicated Core Cooling Red (FR-C.1). OR
R E A	6	< el. 695'. 3. Incore TCs (if available) indicate RCS temperature > 200 °F.	3,	2b. CSF status tree indicates Heat Sink Red (FR-H.1) (RHR shutdown cooling not in service).
E M M R		4. Containment closure is established. Note: If containment is open refer to "Gaseous Effluents" (Section 7.1) and continue in this column.	4	Note: Refer to "Reactor Protection System Failure" (Section 2.3) and Continue in This Column.
		Inability to maintain unit in cold shutdown when required with containment closure established. (1 and 2 and 3):	1,	Complete loss of function needed to achieve cold shutdown when cold shutdown required by Tech. Specs. (1 and 2 and 3):
A L	5,	 Cold shutdown required by Technical Specs. Incore TCs (if available) indicate core exit temperature 	2,	 Cold shutdown required by Tech. Specs. Loss of RHR shutdown cooling capability.
E R	6	> 200 °F. 3. Containment closure is established.	3,	3. Loss of secondary heat sink and main condenser
Т				
		Note: If containment is open refer to "Gaseous Effluents" (Section 7.1) and continue in this column.	4	Note: Also refer to "Failure of Rx Protection" (Section 2.3) and Continue in This Column.
U N U S		Not Applicable.	1,	Inability to reach required shutdown within Tech. Spec. limits. 1. The unit has not been placed in the required mode
U A			2,	within the time prescribed by the LCO action statement.
L			3,	
E V E			4	
N T				

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		6.2 Loop of PCS Inventory
	Mode	6.3 Loss of RCS Inventory
GENERAL EMER		Refer to "Gaseous Effluents" (Section 7.1) and Continue in This Column.
SITE AREA EMER ALERT		Refer to "Gaseous Effluents" (Section 7.1) and Continue in This Column. Refer to "Gaseous Effluents" (Section 7.1) and Continue in This Column.
U N U S U	5,	Loss of REACTOR COOLANT SYSTEM inventory with inadequate makeup. <i>(1 and 2 and 3):</i> 1. Reactor coolant system is pressurized above atmospheric pressure.
A L E V E N T	6	 Unplanned decrease in RCS or pressurizer level requiring initiation of makeup to the RCS. With reactor coolant system temperature stable, the pressurizer level continues to decrease following initiation of RCS makeup.

END OF SECTION 6

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2.6 RCS Identified Leakage

2.7 Uncontrolled Cool Down

2.8 Turbine Failure

2.9 Safety Limit

FISSION PRODUCT BARRIER MATRIX (Modes 1-4)

- 1.1 Fuel Clad Barrier
- 1.2 RCS Barrier
- 1.3 Containment Barrier

SYSTEM DEGRADATION

- 2.1 Loss of Instrumentation
- 2.2 Loss of Communication
- 2.3 Failure of Reactor Protection
- 2.4 Fuel Clad Degradation
- 2.5 RCS Unidentified Leakage

LOSS OF POWER

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

HAZARDS and SED JUDGEMENT

4.1 Fire

- 4.3 Flammable Gas
- 4.2 Explosion Table 4-1 Figure 4-A
- 4.3 Flammable Gas
 4.4 Toxic Gas or Smoke Table 4-2 Figure 4-B

DESTRUCTIVE PHENOMENON

- 5.1 Earthquake
- 5.2 Tornado
- 5.4 River Level High 5.5 River Level Low
- 5.3 Aircraft/Projectile Table 5-1
- 5.5 River Level Low 5.6 Watercraft Crash

Figure 5-A

SHUTDOWN SYSTEM DEGRADATION

- 6.1 Loss of Shutdown Systems
- 3.2 Loss of Shutdown Capability
- 3.3 Loss of RCS Inventory

RADIOLOGICAL EFFLUENTS

- 7.1 Gaseous Effluent
- 7.2 Liquid Effluent Table 7-1 Figure 7-A
- 7.3 Radiation Levels7.4 Fuel Handling Table 7-2

- 4.5 Control Room Evacuation
- 4.6 Security
- 4.7 SED Judgment Table 4-3 Figure 4-C

EMERGENCY PLAN CLASSIFICATION MATRIX

DEFINITIONS/ACRONYMS

BOMB: An explosive device. (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons within the EAB violently protesting onsite operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs; Subcriticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the initiating conditions associated with the event exist. Implicit in this definition is the need for timely assessment within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): That area surrounding the reactor, in which the reactor licenses has the authority to determine all activities including exclusion or removal of personnel and property from the area. For purposes of Emergency Action Levels, based on radiological field measurements and dose assessments, and for design calculations, the Site Boundary shall be defined as the EAB.

EXPLOSION: Rapid, violent, unconfined combustion, or a catastrophic failure of pressurized or electrical equipment that imparts energy of sufficient force to potentially damage permanent structures or equipment.

EXTORTION: An attempt to cause an action at the site by threat or force.

_=AULTED: (Steam Generator) Existence of secondary side leakage (e.g., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

TIRE: Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical components do not constitute a fire. Observation of flame is preferred but is NOT required if large quantities of smoke and/or heat are observed.

FLAMMABLE GAS: Combustible gases at concentrations > than the LOWER EXPLOSIVE LIMIT (LEL).

HOSTAGE: A person(s) held as leverage against the site to ensure that Jemands will be met by the site.

IMMINENT: Within two hours.

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INEFFECTIVE: When the specified restoration action(s) does not result in a reduction in the level of severity of the RED or ORANGE PATH condition within 15 minutes from identification of the CSF Status Tree RED or ORANGE PATH.

NITIATION CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

NTRUSION/INTRUDER: Suspected hostile individual present in the protected area without authorization.

DDCM: Offsite Dose Calculation Manual is a supporting document to the Tech Specs. that contain Rad Effluent Controls, Environs Monitoring controls, and .nethodology for calculating gaseous and liquid effluent offsite doses and monitor alarm/trip setpoints.

DRANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge; prompt operator action is required.

PROJECTILE: An object ejected, thrown or launched towards a plant structure resulting in damage sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein. The source of the projectile may be onsite or offsite.

PROTECTED AREA: The area encompassed by the security fence and to which access is controlled.

RCS: The **RCS** primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary and secondary isolation valves.

RED PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than the capacity one charging pump.

SABOTAGE: Deliberate damage, misalignment, or misoperation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: AN UNPLANNED event involving one or more of the following: (1) An automatic turbine runback >15% thermal reactor power; (2) Electrical load rejection >25% full electrical load; (3) Reactor Trip; (4) Safety Injection System Activation; (5) Thermal Power Oscillations ≥10%.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine, CO₂, etc.)

UNPLANNED: An event or action that is not the expected result of normal operations, testing or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED RELEASE: A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, (e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank).

VALID: An indication, report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indication on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analysis. Damage is sufficient to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, or paint blistering. Surface blemishes (e.g., paint chipping, scratches, etc.) should NOT be included as visible damage.

VITAL AREA: Any area within the PROTECTED AREA which contains equipment, systems, devices, or material which the failure, destruction, or release of, could directly or indirectly endanger the public health and safety by exposure to radiation.

RADIOLOGICAL EFFLUENTS

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	7.1 Gaseous Effluents		7.2 Liquid Effluen
Mode	Initiating / Condition	Mode	Initiating / Condi
A L L	 EAB dose, resulting from an actual or imminent release of gaseous radioactivity > 1 Rem TEDE or > 5 Rem thyroid CDE for the actual or projected duration of the release. (1 or 2 or 3): 1. A VALID rad monitor reading exceeds the values under General Emergency in Table 7-1 for >15 minutes, unless assessment within that 15 minutes confirms that the criterion is not exceeded. <u>OR</u> 2. Field surveys indicate >1 Rem/hr β-γ or an I-131 concentration of 3.9E-06 µCi/cm³ at the EAB (Fig. 7-A) <u>OR</u> 3. Dose assessment results indicate EAB dose >1 Rem TEDE or >5 Rem 	-	Not Applicable.
a L	 thyroid CDE for the actual or projected duration of the release (Fig. 7-A). EAB β-γ dose resulting from an actual or imminent release of gaseous radioactivity >100 mrem TEDE or >500 mrem thyroid CDE for actual or projected duration of the release. (1 or 2 or 3): 1. A VALID rad monitor reading > Table 7-1 values under Site Area for >15 minutes, unless assessment within that 15 minutes confirms that the criterion is not exceeded. <u>OR</u> 2. Field surveys indicate >100 mrem/hr β-γ or an I-131 concentration of 3.9E-07 µCi/cm³ at the EAB (Fig. 7-A). <u>OR</u> 3. Dose assessment results indicate EAB dose >100 mrem TEDE or >500 mrem thyroid CDE for actual or projected duration of the release (Fig. 7-A). 		Not Applicable
A -	Any UNPLANNED release of gaseous radioactivity that exceeds 200 times the ODCM Section 1.2.2.1 Limit for >15 minutes. (1 or 2 or 3 or 4) 1. A VALID rad monitor reading > Table 7-1 values under Alert for >15 minutes, unless assessment within that 15 minutes confirms that the criterion is not exceeded. <u>OR</u> 2. Field surveys indicate >10 mrem/hr β - γ at the EAB for >15 minutes (Fig 7-A). OR 3. Dose assessment results indicate EAB dose >10 mrem TEDE for the duration of the release (Fig. 7-A). <u>OR</u> 4. Sample results exceed 200 times the ODCM limit value for an unmonitored release of gaseous radioactivity >15 minutes in duration.	A L L	 Any UNPLANNED release of radioactivity that exceeds the ODCM Section 1.2.1.1 > 15 minutes. (1 or 2) 1. A VALID rad monitor reading values under Alert for >15 m unless assessment within this confirms that the criterion is r <u>QR</u> 2. Sample results indicate an E exceed 200 times the ODCM for an unmonitored release of radioactivity >15 minutes in or the other of the other oth
A L L	 Any UNPLANNED release of gaseous radioactivity that exceeds 2 times the ODCM Section 1.2.2.1 Limit for >60 minutes. (1 or 2 or 3 or 4) 1. A VALID rad monitor reading > Table 7-1 values under UE for >60 minutes, unless assessment within that 60 minutes confirms that the criterion is not exceeded. <u>OR</u> 2. Field surveys indicate >0.1 mrem/hr β-γ at the EAB for >60 minutes (Fig 7-A) <u>OR</u> 3. Dose assessment results indicate EAB dose >0.1 mrem TEDE for the duration of the release (Fig. 7-A). <u>OR</u> 4. Sample results exceed 2 times the ODCM limit value for an unmonitored release of gaseous radioactivity >60 minutes in duration. 	A L L	Any UNPLANNED release of radioactivity to the environm exceeds 2 times the ODCM S 1.2.1.1 Limit for >60 minutes. 1. A VALID rad monitor reading values under UE for >60 min assessment within this time p confirms that the criterion is not exceed <u>QR</u> 2. Sample results indicate an E >2 times the ODCM limit valu unmonitored release of liquid

TABLE 7-1 EFFLUENT RADIATION MONITOR EALS

NOTE:

The monitor values below, if met or exceeded, indicate the need to perform the required assessment. If the assessment can not be completed within 15 minutes (60 minutes for UE), the appropriate emergency classification shall be made based on the VALID reading.

GASEOUS MONITORS	Units ⁽²⁾	UE	Alert	SAE	General Emer
Site Total Release Limit	μCi/s	4.90E+05	4.90E+07	1.31E+08	1.31E+09
<i>U-1 Shield Building</i> 1-RI-90-400 (EFF LEVEL)	μCi/s	4.90E+05	4.90E+07	1.31E+08	1.31E+09
<i>U-2 Shield Building</i> 2-RI-90-400 (EFF LEVEL)	μCi/s	4.90E+05	4.90E+07	1.31E+08	1.31E+09
Auxiliary Building 0-RM-90-101B Limit	cpm	1.03E+05	Offscale ⁽¹⁾	Offscale ⁽¹⁾	Offscale ⁽¹⁾
Service Building 0-RM-90-132B Limit	cpm	2.62E+06	Offscale ⁽¹⁾	Offscale ⁽¹⁾	Offscale ⁽¹⁾
<i>U-1 Main Steam Line Monitors⁽²⁾</i> 1-RI-90-421 thru 424	μCi/cc	1.49E-01	1.49E+01	3.98E+01	3.98E+02
<i>U-2 Main Steam Line Monitors⁽²⁾</i> 2-RI-90-421 thru 424	μCi/cc	1.49E-01	1.49E+01	3.98E+01	3.98E+02
<i>U-1 Condenser Vac Exh</i> 1-RM-90-255 or 256 Limit	mR/h	4.10E+02	4.10E+04	1.09E+05	1.09E+06
<i>U-2 Condenser Vac Exh</i> 2-RM-90-255 or 256 Limit	mR/h	4.10E+02	4.10E+04	1.09E+05	1.09E+06
RELEASE DURATION	minutes	>60	>15	>15	>15
LIQUID MONITORS	Units	UE	Alert	Site Area	General Emer
Site Total Release Limit	μCi/ml	8.33E-04	8.33E-02	N/A	N/A
RM-90-122-RadWaste	cpm	1.95E+05	Offscale ⁽¹⁾	N/A	N/A
RM-90-120,121-S/G Bidn	срт	1.46E+05	Offscale ⁽¹⁾	N/A	N/A
RM-90-225-Cond Demin	cpm	2.59E+05	Offscale ⁽¹⁾	N/A	N/A
RM-90-212- TB Sump	cpm	4.10E+03	4.10E+05	N/A	N/A
RELEASE DURATION	minutes	>60	>15	>15	>15

ASSESSMENT METHOD: SQN EPIP-14 Appendix D, "Methodology For Projecting TEDE."

(1) The calculated value is outside of the upper range for this detector. The maximum monitor output which can be read is 1.0E+07 cpm.

(2) These unit values are based on flow rates through one PORV of 890,000 lb/hr at 1085 psig 600 degrees F. Before using these values, ensure a release to the environment is ongoing, (e.g., PORV).

- NOTE 1: These EALs are based on the assumption that an emergency release is restricted to one pathway from the plant. In all cases, the total site EAL is the limiting value. Therefore, in the case where there are multiple release paths from the plant, it is the total release EAL (obtained from ICS and/or SQN 0-TI-CEM-030-030, "Manual Calculation of Plant Gas, Iodine, and Particulate Release Rates for Offsite Dose Calculation Manual (ODCM) Compliance"TI-30) that will determine whether an emergency classification is warranted.
- NOTE 2: In the case when there is no CECC dose assessment available, the length and relative magnitude of the release is the key in determining the classification. For example, in the case of the NOUE EAL of 2 times the Tech Spec limit, the classification is based more on the fact that a release above the limit has continued unabated for more than 60 minutes, than on the projected offsite dose.

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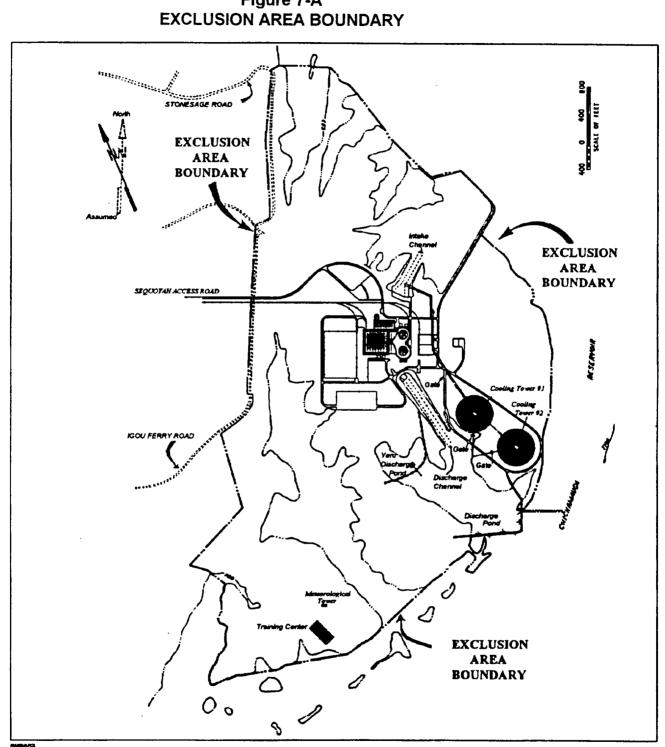


Figure 7-A

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	7.3 Radiation Levels		7.4 Fuel Handling
Mode		Mode	Initiating / Condition
	Refer to "Fission Product Barrier Matrix" (Section 1) or "Gaseous Effluents" (Section 7.1) and Continue in This Section.		Refer to "Gaseous Effluents" (Section 7.1) and Continue in This Section.
	Refer to "Fission Product Barrier Matrix" (Section 1) or "Gaseous Effluents" (Section 7.1) and Continue in This Section.		Refer to "Gaseous Effluents" (Section 7.1) and Continue in This Section.
	UNPLANNED increases in radiation levels within the facility that impedes safe operations or		Major damage to irradiated fuel or loss of water level that has or will uncover irradiated fuel outside the
	 establishment or maintenance of cold shutdown. (1 or 2): 1. VALID area radiation monitor readings or survey results exceed 15 mrem/hr in the control room or SAS. <u>OR</u> 2. (a and b): a. VALID area radiation monitor readings exceed values listed in Table 7-2. b. Access restrictions impede operation of systems necessary for safe operation or the ability to establish cold shutdown (See Note Below). 	A L L	 reactor vessel. (1 and 2): 1. VALID alarm on RM-90-101 or RM-90-102 or RM-90-103 or RM-90-130/131 or RM-90-112. <u>AND</u> 2. (a or b): a. Plant personnel report damage to irradiated fuel sufficient to rupture fuel rods. DR b. Plant personnel report water level drop has or will exceed makeup capacity such that irradiated fuel will be uncovered in the spent fuel pool or transfer
	 UNPLANNED increase in radiation levels within the facility. 1. A VALID area radiation monitor reading increases by 1000 mrem/hr over the highest reading in the past 24 hours excluding the current peak value. Note: In either the UE or ALERT EAL, the SED must determine the cause of increase in radiation levels and review other initiating conditions for applicability (e.g., a dose rate of 15 mrem/hr in the control room could be caused by a release associated with a DBA). 	A L L	canal. UNPLANNED loss of water level in spent fuel pool or reactor cavity or transfer canal with fuel remaining covered. <i>(1 and 2 and 3):</i> 1. Plant personnel report water level drop in spent fuel pool or reactor cavity, or transfer canal. 2. VALID alarm on RM-90-101 or RM-90-102 or RM-90-103. 3. Fuel remains covered with water.

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Table 7-2

ALERT - RADIATION LEVELS

Monitor No.	Location Area and Eleva	tion	Meter Reading
For purposes of comparing the	meter/monitor reading values	to this table,	it can be assumed that mR is equivalent to mrem.
1,2-RM-90-1	Spent Fuel Pit ARM		1.5E+03 mR/hr
0-RM-90-3	Waste Packaging ARM	El. 706.0	1.5E+03 mR/hr
0-RM-90-4	Decontamination Room ARM	El. 690.0	1.5E+03 mR/hr
0-RM-90-5	SFP Pumps ARM	El. 714.0	1.5E+03 mR/hr
1,2-RM-90-6	CCS HXS ARM	El. 714.0	1.5E+03 mR/hr
1,2-RM-90-7	Sample Rm ARM	El. 690.0	1.5E+03 mR/hr
1,2-RM-90-8	AFW Pumps ARM	El. 690.0	1.5E+03 mR/hr
0-RM-90-9	Waste Cnds Tks ARM	El. 669.0	1.5E+03 mR/hr
1,2-RM-90-10	CVCS Bd ARM	El. 669.0	1.5E+03 mR/hr
0-RM-90-11	Cntmt Spray and RHR Pumps Radmon	El. 653.0	1.5E+03 mR/hr
0-RM-90-102	Spent Fuel Pit Radmon	El. 734.0	1.5E+03 mR/hr
0-RM-90-103	Spent Fuel Pit Radmon	El. 734.0	1.5E+03 mR/hr
0-RM-90-230	CNDS Demineralizer ARM	El. 685.0	1.5E+03 mR/hr
0-RM-90-231	Cnds Demineralizer ARM	El. 706.0	1.5E+03 mR/hr

Note: All of the above monitors have a range of 0.1 to 1E+4 mrem/hr.

END OF SECTION 7.

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