July 14, 2000

LICENSEE: Wisconsin Electric Power Company

FACILITY: Point Beach Nuclear Plant, Units 1 and 2

SUBJECT: MEETING AND TELECONFERENCES BETWEEN THE WISCONSIN ELECTRIC POWER COMPANY AND THE NRC STAFF CONCERNING INPUT ASSUMPTIONS FOR CALCULATION OF A CONTROL ROOM DOSE ANALYSIS

The NRC staff met with representatives from Wisconsin Electric Power Company (the licensee) at NRC Headquarters on May 25, 2000, to discuss input assumptions for calculation of control room dose analysis. The licensee and the staff followed up with two conference calls to continue discussions of this subject, one on May 31, 2000, and the second on June 21, 2000. Enclosure 1 lists the meeting and teleconference participants. Enclosure 2 lists the input assumptions that were discussed.

The licensee plans to submit an amendment in the spring of 2001 which will include a reanalysis of its control room dose during accident conditions. The licensee requested a meeting with the staff to discuss the input parameters for the control room dose analysis in order to adequately prepare its amendment.

The meeting was followed up with two teleconferences to continue the discussion between the staff and the licensee regarding the input parameters. On May 31, 2000, a teleconference was held to discuss the filter efficiency assumptions and details of the Point Beach, Units 1 and 2, control room ventilation system. On June 21, 2000, a teleconference was held between the staff and the licensee to discuss iodine release and spiking assumptions.

/RA/
Beth A. Wetzel, Senior Project Manager, Section1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures: 1. Meeting Participants
2. Input Assumptions

cc w/encls: See next page

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Mark Blumberg	NRC/SPSB
Leta Brown	NRC/SPSB
Janak Raval	NRC/SPLB
Harold Scott	NRC, Office of Research
Harold Walker	NRC/SPLB
John Segala	NRC/SPLB
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David Black	WE/Point Beach
Sara Scott	WE/Point Beach

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WE = Wisconsin Electric Power Company
NRR = Office of Nuclear Reactor Regulation

ENCLOSURE 1

INPUT ASSUMPTIONS

1. Dose acceptance criteria for non-loss-of-coolant accidents (LOCAs)
2. Use of ARCON96 for X/Q factors
3. Control Room Occupancy Factors for LOCA
4. Emergency Core Cooling System leakage
5. Iodine partition factors / flashing fraction
6. Filter efficiency assumptions
7. Steam generator tube rupture (SGTR) release assumptions
8. SGTR iodine spiking factor
9. Basis for determining release termination for SGTR and main steamline break
10. Fuel pool scrubbing factor for fuel handling accident
11. Approval process for using fuel enthalpy as fuel damage criteria for locked rotor analysis

ENCLOSURE 2