

August 10, 2000

Mr. G. A. Kuehn, Jr.  
Program Director SNEC Facility  
GPU Nuclear Corporation  
2574 Interstate Drive  
Harrisburg, PA 17110

SUBJECT: SAXTON NUCLEAR EXPERIMENTAL CORPORATION FACILITY - AMENDMENT  
RE: ORGANIZATIONAL STRUCTURE (TAC NO. MA8664)

Dear Mr. Kuehn:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 16 to Amended Facility License No. DPR-4 for the GPU Nuclear Corporation and Saxton Nuclear Experimental Corporation Facility. The amendment consists of changes to the Technical Specifications (TSs) in response to your application of April 10, 2000, as supplemented on May 9, July 13, and August 8, 2000.

The amendment changes the organizational and administrative controls of the TSs to reflect changes made to GPU Nuclear following the sale of the Oyster Creek Nuclear Generating Station.

A copy of the safety evaluation supporting Amendment No. 16 is also enclosed. The notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

***/RA/***

Alexander Adams, Jr., Senior Project Manager  
Events Assessment, Generic Communications and  
Non-Power Reactors Branch  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket No. 50-146

Enclosures:

1. Amendment No. 16
2. Safety Evaluation

cc w/enclosures:

Please see next page

Saxton Nuclear  
Experimental Corporation

Docket No. 50-146

cc:

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Saxton Citizens Task Force  
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Mr. Jim Tydeman  
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Saxton, PA 16678

Mr. James H. Elder, Chairman  
Concerned Citizens for SNEC Safety  
Wall Street Ext.  
Saxton, PA 16679

Mr. Ernest Fuller  
1427 Kearney Hill Road  
Six Mile Run, PA 16679

Saxton Borough Council  
ATTN: Peggy Whited, Secretary  
9<sup>th</sup> and Spring Streets  
Saxton, PA 16678

Mr. David J. Thompson, Chair  
Bedford County Commissioners  
County Court House  
203 South Juliana Street  
Bedford, PA 15522

Mr. Larry Sather, Chairman  
Huntingdon County Commissioners  
County Court House  
Huntingdon, PA 16652

Saxton Community Library  
Front Street  
Saxton, PA 16678

Carbon Township Supervisors  
ATTN: Penny Brode, Secretary  
R. D. #1, Box 222-C  
Saxton, PA 16678

Hopewell Township Supervisors  
ATTN: Sally Giornesto, Secretary  
RR 1 Box 95  
James Creek, PA 16657-9512

Mr. D. Bud McIntyre, Chairman  
Broad Top Township Supervisors  
Broad Top Municipal Building  
Defiance, PA 16633

Mr. Don Weaver, Chairman  
Liberty Township Supervisors  
R. D. #1  
Saxton, PA 16678

U.S. Army Corps of Engineers  
Baltimore District  
ATTN: S. Snarski/P. Juhle  
P.O. Box 1715  
Baltimore, MD 21203

The Honorable Robert C. Jubelirer  
President Pro-Temp Senate of  
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30<sup>th</sup> District  
State Capitol  
Harrisburg, PA 17120

James J. Byrne  
Three Mile Island Nuclear Station  
P.O. Box 480  
Middletown, PA 17057

August 10, 2000

Mr. G. A. Kuehn, Jr.  
Program Director SNEC Facility  
GPU Nuclear Corporation  
2574 Interstate Drive  
Harrisburg, PA 17110

SUBJECT: SAXTON NUCLEAR EXPERIMENTAL CORPORATION FACILITY - AMENDMENT  
RE: ORGANIZATIONAL STRUCTURE (TAC NO. MA8664)

Dear Mr. Kuehn:

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The amendment changes the organizational and administrative controls of the TSs to reflect changes made to GPU Nuclear following the sale of the Oyster Creek Nuclear Generating Station.

A copy of the safety evaluation supporting Amendment No. 16 is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

**/RA/**

Alexander Adams, Jr., Senior Project Manager  
Events Assessment, Generic Communications and  
Non-Power Reactors Branch  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket No. 50-146

Enclosures:

- 1. Amendment No. 16
- 2. Safety Evaluation

cc w/enclosures:

Please see next page

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SAXTON NUCLEAR EXPERIMENTAL CORPORATION

GPU NUCLEAR CORPORATION

DOCKET NO. 50-146

AMENDMENT TO AMENDED FACILITY LICENSE

Amendment No. 16  
License No. DPR-4

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that
  - A. The application for an amendment to Amended Facility License No. DPR-4 filed by GPU Nuclear Corporation and the Saxton Nuclear Experimental Corporation (the licensees) on April 10, 2000, as supplemented on May 9, July 13, and August 8, 2000, conforms to the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the regulations of the Commission as stated in Chapter I of Title 10 of the *Code of Federal Regulations* (10 CFR);
  - B. The facility will be possessed in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance that (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) such activities will be conducted in compliance with the regulations of the Commission;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. This amendment is issued in accordance with the regulations of the Commission as stated in 10 CFR Part 51, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Amended Facility License No. DPR-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 16, are hereby incorporated in the license. SNEC and GPU Nuclear shall possess the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ledyard B. Marsh, Chief  
Events Assessment, Generic Communications and  
Non-Power Reactors Branch  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Enclosure:  
Appendix A, Technical  
Specifications Changes

Date of Issuance: August 10, 2000

ENCLOSURE TO LICENSE AMENDMENT NO. 16

AMENDED FACILITY LICENSE NO. DPR-4

DOCKET NO. 50-146

Replace the following pages of Appendix A, "Technical Specifications," with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

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Insert

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personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the SNEC Facility USAR.

- 3.1.1 The GPU NUCLEAR Cognizant Officer is responsible for and provides full-time dedicated staff for the purpose of conducting all activities safely, effectively and in accordance with corporate policies, applicable laws, regulations, licenses and Technical Specifications (TSs).
- 3.1.2 The Program Director SNEC Facility is responsible for administration of all SNEC facility functions, for direction of all DECOMMISSIONING ACTIVITIES, and for assuring that the requirements of License No. DPR-4 and these TSs are implemented.
- 3.1.3 The SNEC Facility Site Supervisor provides on-site management and continuing oversight of PRODUCTION ACTIVITIES.
- 3.1.4 The Radiation Safety Officer (RSO) is responsible for the conduct and oversight of all SNEC radiation safety activities through implementation of the Radiation Protection Plan. All radiological controls personnel shall have stop work authority in matters relating to or impacting radiation safety.
- 3.1.5 The Group Radiological Controls Supervisor (GRCS) directly supervises radiation safety activities.
- 3.1.6 Other GPU Inc. personnel provide SNEC facility management with technical support, project management capabilities and manpower.
- 3.2 Facility Staffing Requirements:
  - 3.2.1 At least two individuals, one of which must be knowledgeable in radiation monitoring and the radiological hazards associated with the facility, shall perform radiological surveys necessary to support the initial entry into the CV for the day.
  - 3.2.2 The RSO or a GRCS shall be present on site whenever CV entry, PRODUCTION ACTIVITIES, maintenance, characterization and/or RADIOACTIVE WASTE MANAGEMENT ACTIVITIES are being performed in Radiologically Controlled Areas (RCA's).

3.3 Facility Staff Qualifications

3.3.1 Each Radiological Controls Technician/GRCS shall meet or exceed the qualifications of ANSI N18.1-1971, paragraph 4.5.2 and 4.3.2 respectively or shall be formally qualified through an NRC approved radiological controls training program.

3.3.2 All personnel performing decommissioning or associated activities shall be briefed on the SNEC site specific conditions and applicable requirements of the task assigned.

3.4 Training

GPU NUCLEAR shall maintain training programs as necessary for those personnel performing decommissioning work functions at the SNEC facility.

3.4.1 The job specific training or knowledge necessary to accomplish a specific task or project goal shall be dictated by the dismantlement or decommissioning work scope.

3.4.2 In lieu of training, performance of specialized tasks, techniques and equipment operation shall be permitted, provided competency is demonstrated to the Program Director SNEC Facility or his designee.

3.5 Review, Inspection and Audit

3.5.1 Technical Review

3.5.1.1 The GPU NUCLEAR Cognizant Officer is responsible for ensuring the preparation, review, and approval of documents required by the activities described in sections 3.5.1.2 through 3.5.1.7. Implementing approvals shall be performed at the cognizant manager level or above.

3.5.1.2 Each procedure required by section 3.6.1 and SUBSTANTIVE CHANGES thereto shall be prepared by a designated individual or group knowledgeable in the area of the affected procedure. Each procedure and substantive change thereto, shall be given a technical review by an individual or group other than the preparer, but who may be from the same organization as the individual who prepared the procedure or change.

- 3.5.1.10 Records of the review activities performed in accordance with 3.5.1.2 through 3.5.1.7 shall be maintained in accordance with section 3.9.
- 3.5.2 Independent Safety Review
- 3.5.2.1 The GPU NUCLEAR Cognizant Officer is responsible for ensuring the independent safety review of the subjects described in section 3.5.2.3.
- 3.5.2.2 Independent safety review shall be completed by an individual or group not having direct responsibility for the performance of activities under review, but who may be from the same functionally cognizant organization as the individual or group performing the original work.
- 3.5.2.3 GPU NUCLEAR shall collectively have or have access to the experience and competence required to independently review subjects in the following areas:
- nuclear unit operations
  - electrical, mechanical and nuclear engineering
  - chemistry and radiochemistry
  - metallurgy
  - instrumentation and control
  - radiological safety
  - administrative controls and quality assurance practices
  - other appropriate fields such as radioactive waste management.
- 3.5.2.4 Consultants may be utilized as determined by the GPU NUCLEAR Cognizant Officer to provide expert advice.
- 3.5.2.5 The following subjects shall be independently reviewed by Independent Safety Reviewers:
- 3.5.2.5.1 Written safety evaluations of changes in the facility and changes of procedures described in the Safety Analysis Report, and of tests or experiments not described in the Safety Analysis Report, which are completed without prior NRC approval under the provisions of 10 CFR 50.59(a)(1). This review is to verify that such changes, tests or

- a. Verification that EXCLUSION AREA access points are SECURED at the completion of each authorized entry.
- b. Verification of the OPERABILITY of the EXCLUSION AREA intrusion alarms shall be performed quarterly.
- c. The station ventilation system effluent particulate monitor channel checks, source checks, channel test and channel calibration shall be performed at a frequency specified in the ODCM.
- d. The ventilation system HEPA Filter will be tested to verify efficiencies in accordance with the requirements of the ODCM.

3.5.4 Audits

The audit function is independent of the SNEC facility management. Audits shall be performed by qualified individuals, as a minimum, for those activities designated within the scope of the SNEC facility's Quality Assurance Program. Audits are generally conducted biennially, however, frequency is based on the level of activity at the SNEC facility. Audits may also be performed at the request of the GPU NUCLEAR Cognizant Officer. Audits are performed in accordance with approved Quality Assurance Plan procedures. The audit procedures identify areas which may be included in the audit scope. Audit reports shall be forwarded to the GPU NUCLEAR Cognizant Officer within 60 days of completion of the audit.

3.5.5 TMI-2/SNEC Oversight Committee

3.5.5.1 The TMI-2/SNEC Oversight Committee shall report to the GPU NUCLEAR Cognizant Officer. The Committee will consist of at least four members. Membership will be on the recommendation of the Committee Chairman and approval of the GPU NUCLEAR Cognizant Officer. Three members shall constitute a quorum.

3.5.5.2 It shall be responsible to provide independent overview and assessment of all matters with radiological safety implications relative to activities at the SNEC facility. The Committee will review proposed License and Technical Specification changes, DECOMMISSIONING ACTIVITIES, special nuclear and radioactive material activities, facility changes, radiological conditions, audit reports and NRC Inspection reports and corrective actions for deficiencies identified.

- 3.5.5.3 Meetings shall be held at least three times per year.
- 3.5.5.4 Written minutes of all meetings shall be prepared and distributed to the GPU NUCLEAR Cognizant Officer within 30 days of the meeting date.

3.6 Procedures, Programs and Manuals

3.6.1 Procedures

3.6.1.1 Activities which are designated as within the scope of the SNEC facility's Quality Assurance Program shall be prescribed by written, reviewed and approved procedures of a type appropriate to the circumstances.

3.6.1.2 Written procedures shall be established, implemented and maintained for the activities listed below:

3.6.1.2.1 Characterization, decommissioning and maintenance activities determined to be within the scope of the QA program.

3.6.1.2.2 Access control, emergency actions (including fire protection program implementation), facility inspections and audits.

3.6.1.2.3 Radiological exposure control, survey activities and radwaste shipping and handling.

3.6.1.2.4 Activities which could result in a MEASURABLE RELEASE to the environment.

3.6.1.3 These procedures shall require that the following actions be taken:

3.6.1.3.1 All DECOMMISSIONING ACTIVITIES and maintenance work under Health Physics control shall be consistent with 10 CFR Part 20 requirements to minimize the radiation exposure of personnel and to prevent the release of radioactivity in excess of allowable limits to the environment.

3.6.1.3.2 All radiation surveys, tests, counting work, radiation exposure control measures and all other work performed in radiologically controlled areas shall conform with the requirements of the Radiation Protection Plan.

10 CFR, Part 20.1301; 40 CFR, Part 190; 10 CFR, Section 50.36(a); and Appendix I to 10 CFR, Part 50 and not adversely impact the accuracy or reliability of effluent, dose or setpoint calculations.

- b. Shall become effective after review and approval of the Program Director, SNEC Facility.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the areas of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

### 3.7 Fire Protection

- 3.7.1 Procedures will be established and implemented for fire prevention and responding to fires.
- 3.7.2 Portable fire extinguishing equipment shall be available to support DECOMMISSIONING ACTIVITIES in progress.
- 3.7.3 Personnel standing fire watch will be trained and requalified on an annual frequency.

### 3.8 Reporting

In addition to those reports required by applicable NRC regulations (i.e., violation of license or technical specification condition) GPU NUCLEAR shall submit the following:

- 3.8.1 A report of any occurrence of a possible unsafe condition relating to the facility or to the public. For each occurrence, SNEC shall promptly, within 24 hours of discovery, notify by telephone or facsimile the NRC Operations Center, and shall submit a written follow-up report to the Document Control Desk within 30 days, which describes the circumstances and the corrective action taken. These reports shall include:

- 3.8.1.1 Any unplanned or uncontrolled release of radioactive material beyond the SITE BOUNDARY.
- 3.8.2 An annual report shall be submitted to the Document Control Desk, within 6 months after the end of the calendar year, of the status of the deactivated facility including:
  - 3.8.2.1 Information relating to changes in those management and supervisory positions designated in section 3.1 as being responsible for decommissioning the facility;
  - 3.8.2.2 A summary of decommissioning, design, and maintenance changes made to the deactivated facility;
  - 3.8.2.3 Results of surveys and monitoring performed in accordance with Specifications 3.6.2.1 and 3.6.2.2;
  - 3.8.2.4 A review of the performance of access control and surveillance measures.

3.9 Records

In addition to the records required by applicable NRC regulations, including Subpart L of 10 CFR, Part 20, 20.2101 through 20.2110 inclusive, GPU NUCLEAR shall retain records of the following for the duration of the license:

- 3.9.1 Records of all reportable events submitted to the Commission;
- 3.9.2 Records of principal DECOMMISSIONING ACTIVITIES;
- 3.9.3 Records of training and qualification of members of the facility staff;
- 3.9.4 Records of entries into the CONTAINMENT VESSEL involving radiation work permits and the reason for entry;
- 3.9.5 Radioactivity releases or discharges into the air or water beyond the effective control of SNEC as measured at or prior to the point of such release or discharge;
- 3.9.6 Records of reviews performed for changes made to procedures or equipment pursuant to 10 CFR 50.59;

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 16 TO

AMENDED FACILITY LICENSE NO. DPR-4

GPU NUCLEAR CORPORATION

SAXTON NUCLEAR EXPERIMENTAL CORPORATION

DOCKET NO. 50-146

1.0 INTRODUCTION

By letter dated April 10, 2000 (Reference 1), as supplemented by letters dated May 9, 2000 (Reference 3), July 13, 2000 (Reference 4), and August 8, 2000 (Reference 5), GPU Nuclear Corporation (the licensee) submitted Technical Specification Change Request (TSCR) No. 60 to amend Section 3.0, "Administrative Controls," of the Saxton Nuclear Experimental Corporation (SNEC) Facility (SNEF) Technical Specifications (TSs) to reflect the organizational and administrative controls that will exist at the SNEF following the sale of the Oyster Creek Nuclear Generating Station to AmerGen. The supplements did not expand the scope of the application, as noticed in the Federal Register (65 FR 39956, dated June 28, 2000), or change the proposed no significant hazards consideration determination.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36, "Technical Specifications." That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. Addressing administrative controls, 10 CFR 50.36 states that they "are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner."

The proposed changes described in TSCR No. 60 would modify Section 3 of the SNEF TSs to (1) replace reference to the President of GPU Nuclear and division Vice Presidents with a "GPU Nuclear Cognizant Officer," (2) replace reference to "other GPU Nuclear personnel" with "other GPU Inc. personnel," (3) replace reference to the "Radiation Safety Committee" with the "TMI2/SNEC Oversight Committee," (4) replace "GPU Nuclear audit program procedures" with "approved Quality Assurance Plan procedures," and (5) make changes to the SNEF TSs to reflect NRC organizational changes. These changes reflect the consolidation of responsibilities and organization that follows the reduction in size of GPU Nuclear to reflect the exit of the

corporation from the nuclear electric generation business. The primary purpose of TSCR No. 60 is to incorporate these organizational and administrative changes into the SNEF licensing basis.

### 3.0 EVALUATION

Section 13.4, "Operational Review", of NUREG-0800, the "Standard Review Plan" (SRP), provides the acceptance criteria used by the staff to evaluate TS provisions related to administrative controls. This acceptance criteria is based on meeting the relevant requirements of 10 CFR 50.40(b) as it relates to the licensee being technically qualified to engage in licensed activities, and of Appendix B to 10 CFR 50 as it relates to the review and audit functions required by the licensee's quality assurance program.

We reviewed the changes described in TSCR No. 60 against the acceptance criteria set forth in Section 13.4 of the SRP and have concluded that they simply reflect title and/or organizational changes that have no impact on compliance with the relevant requirements of 10 CFR 50.36(c)(5) and Appendix B to 10 CFR Part 50 for a decommissioning status facility.

Regarding the proposed change affecting audit program procedures at the SNEF, the staff issued a request for additional information (RAI) on May 4, 2000 (Reference 2), to seek clarification from GPU Nuclear on whether the term "Quality Assurance Plan" as used in TSCR No. 60 refers to GPU Nuclear Corporate Policy & Procedure Manual Number 1000-PLN-3000.05, "Saxton Nuclear Experimental Corporation Facility Decommissioning Quality Assurance Plan," and/or to identify the decommissioning quality assurance plan or program description upon which the audit procedures required by SNEF TS 3.5.4 will be based following the sale of the Oyster Creek Nuclear Generating Station.

On May 9, 2000 (Reference 3), GPU Nuclear replied to the staff's RAI and confirmed that the term "Quality Assurance Plan" as used in TSCR No. 60 does refer to GPU Nuclear Corporate Policy & Procedure Manual Number 1000-PLN-3000.05 which receives review and approval by the NRC staff pursuant to 10 CFR 50.54(a).

The staff noted a change in TS 3.6.3.1.2a.2 (page 17) that was not discussed in the application. A discussion between the SNEF Project Manager and the Manager of Decontamination and Decommissioning for GPU Nuclear on July 14, 2000, revealed that the change was a typographical error and the TS should not be changed.

### 4.0 STATE CONSULTATION

In accordance with the regulations of the Commission, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comment.

### 5.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 6.0 CONCLUSION

The staff has reviewed the proposed changes to Section 3.0 of the SNEF TSs described in TSCR No. 60 and concluded that they are administrative in nature and do not impact the licensee's ability to continue to meet the relevant requirements of 10 CFR 50.36(c)(5) for a decommissioning status facility, and of Appendix B to 10 CFR 50 as it relates to the review and audit functions required by the licensee's quality assurance program. Therefore, they are acceptable.

The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding which was published in the *Federal Register* on June 28, 2000 (65 FR 39956).

The staff has concluded, on the basis of the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by site activities conducted in the proposed manner, (2) such activities will be conducted in compliance with the regulations of the Commission, and (3) the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

## 7.0 REFERENCES

1. GPU Nuclear (G. A. Kuehn) letter to USNRC, "Saxton Nuclear Experimental Corporation Facility (SNEC), Operating License No. DPR-4, Docket No. 50-146, Technical Specification Change Request No. 60," April 10, 2000.
2. USNRC (A. Adams) letter to GPU Nuclear (G. A. Kuehn), "Saxton Nuclear Experimental Facility - Request for Additional Information, RE: Request for License Amendment (TAC No. MA8664)," May 4, 2000.
3. GPU Nuclear (G. A. Kuehn) letter to USNRC, "Saxton Nuclear Experimental Corporation Facility (SNEC), Operating License No. DPR-4, Docket No. 50-146, Technical Specification Change Request No. 60, Response to Request for Additional Information," May 9, 2000.
4. GPU Nuclear (A.H. Rone) letter to USNRC, "Saxton Nuclear Experimental Corporation Facility (SNEC), Operating License No. DPR-4, Docket No. 50-146, Technical Specification Change Request No. 60," July 13, 2000.
5. GPU Nuclear (M. W. Laggart) letter to USNRC "Saxton Nuclear Experimental Corporation Facility (SNEC), Operating License No. DPR-4, Docket No. 50-146, Technical Specification Change Request No. 60," August 8, 2000.

Principal Contributor: J. Peralta

Date: August 10, 2000