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July 7, 2000

Re: Indian Point Unit No. 2
Docket No. 50-247
NL-00-085

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

SUBJECT: 10 C.F.R. § 50.46 Report

In accordance with 10 C.F.R. § 50.46(a)(3)(ii), Consolidated Edison Company of New York, Inc. hereby submits the following with respect to the Emergency Core Cooling System ("ECCS") Evaluation Model changes and errors that affect the Peak Cladding Temperature ("PCT") for the limiting transient.

Westinghouse, Consolidated Edison's vendor for performing large and small break loss of cooling accident analyses, has reviewed the 1999 analyses for Indian Point Unit 2 and has advised us that any changes which have been made to the calculation of ECCS performance has not resulted in a change to the calculated PCT for the most limiting transient. However, the station is continuing to work with Westinghouse on the resolution of an outstanding issue, documented in NSAL-98-004, relative to the potential effect of differences in the as-built and the analyzed ECCS accumulator piping configurations. Should the differences, if any, result in a significant change to the calculated peak cladding temperature for the limiting transient, Consolidated Edison will notify the NRC within 30 days as required by 10 C.F.R. § 50.46(a)(3)(ii).

If the staff has any question regarding this matter, please contact Mr. John F. McCann, Manager, Nuclear Safety and Licensing at (914) 734-5074.

Sincerely,



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cc: Mr. Patrick D. Milano
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