

October 18, 2000

Mr. James A. Hutton
Director-Licensing
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control
P.O. Box 160
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SUBJECT: LIMERICK GENERATING STATION, UNITS 1 AND 2 - ISSUANCE OF
AMENDMENT RE: RELOCATION OF THE PRIMARY CONTAINMENT
ISOLATION VALVE TABLE FROM THE TECHNICAL SPECIFICATIONS TO THE
TECHNICAL REQUIREMENTS MANUAL AND DELETING REFERENCES TO
TABLES (TAC NOS. MA8101 AND MA8102)

Dear Mr. Hutton:

The Commission has issued the enclosed Amendment No. 146 to Facility Operating License No. NPF-39 and Amendment No. 107 to Facility Operating License No. NPF-85 for the Limerick Generating Station (LGS), Units 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated December 15, 1999, as supplemented August 24, 2000.

These amendments revise the LGS Units 1 and 2 TSs by removing TS Table 3.6.3-1 "Primary Containment Isolation Valves," and references to the TSs Table from the TSs and relocating the information from the TS Table to the Technical Requirements Manual. In addition, an administrative change was made which deleted references to TS Tables 3.6.5.2.1-1 and 3.6.5.2.2-1.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Bartholomew C. Buckley, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-352 and 50-353

Enclosures: 1. Amendment No. 146 to
License No. NPF-39
2. Amendment No. 107 to
License No. NPF-85
3. Safety Evaluation

cc w/encls: See next page

Limerick Generating Station, Units 1 & 2

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October 18, 2000

SUBJECT: LIMERICK GENERATING STATION, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT RE: RELOCATION OF THE PRIMARY CONTAINMENT ISOLATION VALVE TABLE FROM THE TECHNICAL SPECIFICATIONS TO THE TECHNICAL REQUIREMENTS MANUAL AND DELETING REFERENCES TO TABLES (TAC NOS. MA8101 AND MA8102)

Dear Mr. Hutton:

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These amendments revise the LGS Units 1 and 2 TSs by removing TS Table 3.6.3-1 "Primary Containment Isolation Valves," and references to the TS Table from the TSs and relocating the information from the TS Table to the Technical Requirements Manual. In addition, an administrative change was made which deleted references to TS Tables 3.6.5.2.1-1 and 3.6.5.2.2-1.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
 /RA/

Bartholomew C. Buckley, Sr. Project Manager, Section 2
 Project Directorate I
 Division of Licensing Project Management
 Office of Nuclear Reactor Regulation

Docket Nos. 50-352 and 50-353

- Enclosures: 1. Amendment No. 146 to License No. NPF-39
 2. Amendment No. 107 to License No. NPF-85
 3. Safety Evaluation

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Accession Number: ML003730429

*See previous concurrence

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DATE	10/10/00	10/4/00	9/6/00	8/18/00	9/18/00	10/17/00	

Official Record Copy

PECO ENERGY COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 146
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by PECO Energy Company (the licensee) dated December 15, 1999, as supplemented August 24, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 146, are hereby incorporated into this license. PECO Energy Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days. Implementation includes relocation of information from Technical Specification Table 3.6.3-1 to the Technical Requirements Manual.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: October 18, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 146

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
xii	xii
1-5	1-5
3/4 3-16	3/4 3-16
3/4 3-17	3/4 3-17
3/4 3-26	3/4 3-26
3/4 6-1	3/4 6-1
3/4 6-2	3/4 6-2
3/4 6-3	3/4 6-3
3/4 6-17	3/4 6-17
3/4 6-18	3/4 6-18
3/4 6-19 - 3/4 6-43a	3/4 6-19
B 3/4 3-2	B 3/4 3-2
B 3/4 6-4	B 3/4 6-4

PECO ENERGY COMPANY

DOCKET NO. 50-353

LIMERICK GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 107
License No. NPF-85

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by PECO Energy Company (the licensee) dated December 15, 1999, as supplemented August 24, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-85 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 107 , are hereby incorporated in the license. PECO Energy Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days. Implementation includes relocation of information from Technical Specification Table 3.6.3-1 to the Technical Requirements Manual.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: October 18, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 107

FACILITY OPERATING LICENSE NO. NPF-85

DOCKET NO. 50-353

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
xii	xii
1-5	1-5
3/4 3-16	3/4 3-16
3/4 3-17	3/4 3-17
3/4 3-26	3/4 3-26
3/4 6-1	3/4 6-1
3/4 6-2	3/4 6-2
3/4 6-3	3/4 6-3
3/4 6-17	3/4 6-17
3/4 6-18	3/4 6-18
3/4 6-19 - 3/4 6-43a	3/4 6-19
B 3/4 3-2	B 3/4 3-2
B 3/4 6-4	B 3/4 6-4

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 146 AND 107 TO FACILITY OPERATING
LICENSE NOS. NPF-39 AND NPF-85
PECO ENERGY COMPANY
LIMERICK GENERATING STATION, UNITS 1 AND 2
DOCKET NOS. 50-352 AND 50-353

1.0 INTRODUCTION

By application dated December 15, 1999, as supplemented August 24, 2000, PECO Energy Company (PECO or the Licensee) proposed an amendment to the Appendix A Technical Specifications (TSs) for the Limerick Generating Station (LGS), Units 1 and 2. The proposed amendments revise the LGS TSs to remove TS Table 3.6.3-1, "Primary Containment Isolation Valves," and references to the table, from the TSs and relocate the information from the TS Table to the Technical Requirements Manual (TRM), a licensee-controlled document. The licensee's proposed change is consistent with the guidance of Generic Letter (GL) 91-08, "Removal of Components Lists from Technical Specifications." In addition, an administrative change to the TSs is proposed to delete references to TS Tables 3.6.5.2.1-1 and 3.6.5.2.2-1, which were removed in previous amendments. The August 24, 2000, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination or expand the scope of the original Federal Register Notice.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act, as amended (the "Act"), requires applicants for nuclear power plant operating licenses to incorporate TSs as part of the license. The Commission's regulatory requirements related to the content of the TSs are set forth in Section 50.36 of Title 10 of the Code of Federal Regulations (10 CFR) (Section 50.36). That regulation requires that the TSs include items in five categories, including: (1) safety limits, limiting safety system settings, and limiting control settings, (2) limiting condition for operation, (3) surveillance requirements, (4) design features, and (5) administrative controls. It also states that the Commission may include such additional TSs as it finds to be appropriate. However, the regulation does not specify the particular TS to be included in a plant's license.

Pursuant to 10 CFR 50.36(c)(2)(ii), a TS is required for a Limiting Condition of Operation (LCO) meeting one or more of the following criteria:

- (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary;

- (2) a process variable, design feature, or operating restriction that is an initial condition of a design-basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier;
- (3) a structure, system or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of, or represents a challenge to the integrity of a fission product barrier; and
- (4) a structure, system or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

As a result, existing LCO requirements which fall within or satisfy any of the criteria in 10 CFR 50.36(c)(2)(ii) must be retained in the TSs, while those LCO requirements which do not fall within or satisfy these criteria may be deleted from the TSs.

Based on the above criteria, the Nuclear Regulatory Commission (NRC) issued GL 91-08, "Removal of Component Lists from Technical Specifications," on May 6, 1991. In this GL, the staff noted that its intent was to provide guidance for the removal of component lists from the TSs.

3.0 EVALUATION

Deletion of TS Table 3.6.3.-1

In its submittal, the licensee requested TS changes which will revise the LGS TSs to remove the Primary Containment Isolation Valve Table 3.6.3-1, and references to the table, from the TSs and relocate the information from the TS Table to the TRM in accordance with NRC GL 91-08. The TRM is a licensee-controlled document to which changes are adequately controlled under the requirements of the TS Administrative Section 6.0 10 CFR 50.59. The following specific changes are proposed:

1. Delete TS Table 3.6.3-1.
2. Revise the TS Index Item 3/4.6.3, "Primary Containment Isolation Valves," on page xii to reflect the removal of the primary containment isolation valve table from the TSs by deleting the words "Table 3.6.3-1 Primary Containment Isolation Valves 3/4 6-19".
3. Revise TS Definition 1.29, "Primary Containment Integrity," on page 1-5 to state the exception to closure of primary containment penetrations in more general terms rather than making a specific reference to TS Table 3.6.3-1 by replacing the words "as provided in Table 3.6.3-1 of" with the words "for valves that are opened under administrative control as permitted by."
4. Revise the Table Notations for TS Table 3.3.2-1, "Isolation Actuation Instrumentation," to reflect the removal of the primary containment isolation valve table by deleting Item (a), "See Specification 3.6.3, Table 3.6.3-1 for primary containment isolation valves which are actuated by these isolation signals" on page 3/4 3-16.

5. Revise the Table Notations for TS Table 3.3.2-3, "Isolation System Instrumentation Response Time," to reflect the removal of the primary containment isolation valve table by deleting the reference to Table 3.6.3-1, "shown in Tables 3.6.3-1," from Item # on page 3/4 3-26.
6. Revise TS Surveillance Requirement (SR) 4.6.1.1.b, "Primary Containment Integrity," on page 3/4 6-1 to state the exception to closure of primary containment penetrations in more general terms rather than making a specific reference to TS Table 3.6.3-1 by replacing the words "as provided in Table 3.6.3-1 of" with the words "for valves that are opened under administrative control as permitted by."
7. Revise Limiting Condition for Operation (LCO) 3.6.1.2.b, "Primary Containment Leakage," on page 3/4 6-2, to reflect the removal of the primary containment isolation valve table by deleting the references to Table 3.6.3-1 in this LCO concerning the primary containment combined leakage rate. LCO 3.6.2.1.b is being reworded in its entirety for clarity regarding the exceptions to the combined leakage rate. The proposed wording is as follows:

"A combined leakage rate in accordance with the Primary Containment Leakage Rate Testing Program for all primary containment penetrations and all primary containment isolation valves that are subject to Type B and C tests, except for: main steam line isolation valves*, valves which are hydrostatically tested, and those valves where an exemption to Appendix J of 10CFR50 has been granted."

8. Revise LCO Action 3.6.1.2.b on page 3/4 6-2, to reflect the removal of the primary containment isolation valve table by deleting the references to Table 3.6.3-1 in this Action statement concerning the primary containment combined leakage rate. LCO Action 3.6.2.1.b is being reworded in its entirety for clarity regarding the exceptions to the combined leakage rate. The proposed wording is as follows:

"The measured combined leakage rate exceeding the leakage rate specified in the Primary Containment Leakage Rate Testing Program for all primary containment penetrations and all primary containment isolation valves that are subject to Type B and C tests, except for: main steam line isolation valves*, valves which are hydrostatically tested, and those valves where an exemption to Appendix J of 10CFR50 has been granted."

9. Revise LCO Restoration Action 3.6.1.2.b on page 3/4 6-3, to reflect the removal of the primary containment isolation valve table by deleting the references to Table 3.6.3-1 in this Action statement concerning the primary containment combined leakage rate. LCO Restoration Action 3.6.1.2.b is being reworded in its entirety for clarity regarding the exceptions to the combined leakage rate. The proposed wording is as follows:

"The combined leakage rate to be in accordance with the Primary Containment Leakage Rate Testing Program for all primary containment penetrations and all primary containment isolation valves that are subject to Type B and C tests, except for: main steam line isolation valves*, valves which are hydrostatically tested, and those valves where an exemption to Appendix J of 10CFR50 has been granted."

10. Revise LCO 3.6.3, "Primary Containment Isolation Valves," on page 3/4 6-17, to reflect the removal of the primary containment isolation valve table by deleting the references to Table 3.6.3-1 and the reference to isolation times shown in Table 3.6.3-1 as denoted by the following wording:

"Each primary containment isolation valve and each instrumentation line excess flow check valve shall be OPERABLE."

11. Revise the Primary Containment Isolation Valve LCO Actions 3.6.3.a. and 3.6.3.b., on page 3/4 6-17, and Surveillance Requirements 4.6.3.1, 4.6.3.2, 4.6.3.3, and 4.6.3.4, on page 3/4 6-18, to reflect the removal of the primary containment isolation valve table by deleting the references to Table 3.6.3-1, "shown in Table 3.6.3-1," in these sections and adds a note to SR 4.6.3.4 stating "the reactor vessel head seal leak detection line (penetration 29A) excess flow check valve is not required to be tested pursuant to this requirement."

12. Delete TS Table 3.6.3-1, "Primary Containment Isolation Valves," and retain a single page 3/4 6-19 with the following note:

"THE INFORMATION FROM THIS TECHNICAL SPECIFICATION SECTION HAS BEEN RELOCATED TO THE TECHNICAL REQUIREMENTS MANUAL (TRM), PCIV SECTION.

TECHNICAL SPECIFICATION PAGES 3/4 6-19 THROUGH 3/4 6-43a HAVE BEEN INTENTIONALLY OMITTED."

13. Revise TS Bases Section 3/4.3.2, "Isolation Actuation Instrumentation," to add the following statement providing a reference for the location of the primary containment isolation valves that are actuated by the isolation signals specified in TS Table 3.3.2-1:

"Primary containment isolation valves that are actuated by the isolation signals specified in Technical Specification Table 3.3.2-1 are identified in Technical Requirements Manual Table 3.6.3-1."

14. Revise TS Bases Section 3/4.6.3, "Primary Containment Isolation Valves," to add the following statement regarding the scope of primary containment isolation valves to which TS Section 3.6.3 is applicable:

"Primary containment isolation valves governed by this Technical Specification are identified in Table 3.6.3-1 of the TRM."

15. Revise TS Bases Section 3/4.6.3, "Primary Containment Isolation Valves," to add the following statement concerning the administrative controls to be used when opening a valve that was locked or sealed closed to satisfy TS LCO 3.6.3 Action statements:

"The opening of a containment isolation valve that was locked or sealed closed to satisfy Technical Specification 3.6.3 Action statements may be reopened on an intermittent basis under administrative controls. These controls consist of stationing a dedicated individual at the controls of the valve, who is in continuous communication

with the control room. In this way, the penetration can be rapidly isolated when a need for primary containment isolation is indicated."

The staff has reviewed the licensee's proposed deletion of TS Table 3.6.3-1, references to it, and associated TS changes and determined that they are in accordance with the guidance of GL 91-08. Deletion of TS Table 3.6.3-1 and references to it from the TSs and the relocation of the information in TS Table 3.6.3-1 to the TRM does not alter the existing requirements or the components they apply to, thus ensuring that components will continue to be capable of performing their intended safety function. Lists of containment isolation valves are provided in a plant procedure that is subject to the change control provisions for plant procedures in the Administrative Controls Section of the TSs. The licensee must continue to evaluate any changes to the controlled document in accordance with the Administrative Section of the TSs and evaluate any plant modifications that affect any of the components in accordance with 10 CFR 50.59. Should such evaluation conclude that an unreviewed safety question is involved, the NRC approval and a license amendment would be required prior to implementation of the change. The set of valves subject to the requirements of TS Table 3.6.3-1 will not change due to this relocation. The staff, therefore, finds the proposed change acceptable.

The staff has determined that the items in Table 3.6.3-1 are not:

- (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary;
- (2) a process variable, design feature, or operating restriction that is an initial condition of a design-basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier;
- (3) a structure, system or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of, or represents a challenge to the integrity of a fission product barrier; and
- (4) a structure, system or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

Since the relocation of Table 3.6.3-1 to the TRM does not involve any of the above four criteria defined in 10 CFR 50.36(c)(2)(ii), the staff finds the proposed change acceptable.

Administrative Changes:

The licensee also proposed an administrative change to the TSs to delete references to TS Tables 3.6.5.2.1-1 and 3.6.5.2.2-1. On September 14, 1995, the licensee submitted amendment Technical Specification Change Request (TSCR) 95-15-0, "Removal of Secondary Containment Isolation Valve Tables," to the NRC which requested a change to the TSs to remove the Secondary Containment Isolation Valve Tables 3.6.5.2.1-1 and 3.6.5.2.2-1 from the TSs in accordance with GL 91-08. The NRC approved these proposed changes by issuance of TS Amendment Nos. 105 for Unit 1 and 69 for Unit 2 on November 20, 1995. During the development of this amendment request (December 15, 1999, as supplemented August 24, 2000), the licensee identified that references to TS Tables 3.6.5.2.1-1 and

3.6.5.2.2-1 contained in: (1) Table Notations for TS Table 3.3.2-1, Item (c), on page 3/4 3-17, and (2) Table Notations for TS Table 3.3.2-3, Item #, on page 3/4 3-26, were not requested to be deleted from the TSs as part of TSCR 95-15-0. Therefore, this amendment proposes an administrative change to the TSs to delete these references since the Tables to which they make reference were previously deleted by the Amendments identified above. The following specific changes are proposed:

1. Revise the Table Notations for TS Table 3.3.2-1, "Isolation Actuation Instrumentation," to reflect the previously approved removal of the reactor enclosure and refueling area secondary containment isolation valve tables from the TSs by deleting the references to these tables, "shown in Table 3.6.5.2.1-1 and/or 3.6.5.2.2-1 and" from Item (c) on page 3/4 3-17.
2. Revise the Table Notations for TS Table 3.3.2-3, "Isolation System Instrumentation Response Time," to reflect the previously approved removal of the reactor enclosure and refueling area secondary containment isolation valve tables from the TSs by deleting the references to these tables, "3.6.5.2.1-1 and 3.6.5.2.2-1," from Item # on page 3/4 3-26.

The staff finds the licensee's proposal to revise the Table Notations in TS Tables 3.3.2-1 and 3.3.2-3 to remove references to TS Tables 3.6.5.2.1-1 and 3.6.5.2.2-1 to be an administrative change, in that the tables themselves were removed in a previously approved amendment, and this deletion of the references to the tables makes the Table Notations consistent with this previously approved change. The proposed deletion is, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (65 FR 12294). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Buckley

Date: October 18, 2000