Mr. Michael B. Sellman Senior Vice President and Chief Nuclear Officer Wisconsin Electric Power Company 231 West Michigan Street Milwaukee, WI 53201

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: SELECTION OF CONTROL TENDONS FOR THE PRIMARY CONTAINMENT (TAC NOS. MA8158 AND MA8159)

Dear Mr. Sellman:

The Commission has issued the enclosed Amendment No.196 to Facility Operating License No. DPR-24 and Amendment No. 201 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated January 19, 2000.

These amendments revise TS 15.4.4-II.A to clarify that a different primary containment tendon may be designated a control tendon providing that the new control tendon has not previously been physically changed (e.g., retensioned).

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA by A. Kugler for/

Beth A. Wetzel, Senior Project Manager, Section 1 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures: 1. Amendment No. 196 to DPR-24

2. Amendment No. 201 to DPR-27

3. Safety Evaluation

cc w/encls: See next page

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WASHINGTON, D.C. 20555-0001

June 27, 2000

Mr. Michael B. Sellman Senior Vice President and **Chief Nuclear Officer** Wisconsin Electric Power Company 231 West Michigan Street Milwaukee, WI 53201

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: SELECTION OF CONTROL TENDONS FOR THE PRIMARY CONTAINMENT (TAC NOS. MA8158 AND MA8159)

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A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Beth A. Wetzel, Senior Project Manager, Section 1 **Project Directorate III Division of Licensing Project Management** Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

- Enclosures: 1. Amendment No. 196
 - 2. Amendment No. 201
- to DPR-24
- to DPR-27
- 3. Safety Evaluation

cc w/encls: See next page

Point Beach Nuclear Plant, Units 1 and 2

cc:

Mr. John H. O'Neill, Jr. Shaw, Pittman, Potts & Trowbridge 2300 N Street, NW Washington, DC 20037-1128

Mr. Richard R. Grigg President and Chief Operating Officer Wisconsin Electric Power Company 231 West Michigan Street Milwaukee, WI 53201

Mr. Mark E. Reddemann Site Vice President Point Beach Nuclear Plant Wisconsin Electric Power Company 6610 Nuclear Road Two Rivers, WI 54241

Mr. Ken Duveneck Town Chairman Town of Two Creeks 13017 State Highway 42 Mishicot, WI 54228

Chairman Public Service Commission of Wisconsin P.O. Box 7854 Madison, WI 53707-7854

Regional Administrator, Region III U.S. Nuclear Regulatory Commission 801 Warrenville Road Lisle, IL 60532-4351

Resident Inspector's Office U.S. Nuclear Regulatory Commission 6612 Nuclear Road Two Rivers, WI 54241 Ms. Sarah Jenkins Electric Division Public Service Commission of Wisconsin P.O. Box 7854 Madison, WI 53707-7854



WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 196 License No. DPR-24

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated January 19, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:
 - B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. ¹⁹⁶, are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Sett Q. Wetzel, for

Claudia M. Craig, Chief, Section 1 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: June 27, 2000



WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.201 License No. DPR-27

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated January 19, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:
 - B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 201 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Beth a. Wetzel, for

Claudia M. Craig, Chief, Section 1 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: June 27, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 196

TO FACILITY OPERATING LICENSE NO. DPR-24

AND LICENSE AMENDMENT NO. 201

TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

15.4.4-1

15.4.4-1

15.4.4 CONTAINMENT TESTS

Applicability

Applies to containment leakage and structural integrity.

Objective

To verify that potential leakage from the containment and the pre-stressing tendon loads are maintained within acceptable values.

Specification

I. Perform required visual examinations and leakage rate testing in accordance with the Containment Leakage Rate Testing Program.

II. <u>TENDON SURVEILLANCE</u>

A. <u>Object</u>

In order to insure containment structural integrity, selected tendons shall be periodically inspected for symptoms of material deterioration or lift-off force reduction. The tendons for inspection shall be randomly but representatively selected from each group for each inspection; however, to develop a history and to correlate the observed data, one tendon from each group shall be physically unchanged. Tendons selected for inspection will consist of five hoop tendons, three vertical tendons located approximately 120° apart, and three dome tendons, one from each of the three dome tendon groups.

B. Frequency

Tendon surveillance shall be conducted at five-year intervals in accordance with the following schedule:

<u>Unit</u>	Year	Surveillance Required
1	1984	Physical
2	1984	Visual
1	1989	Visual
2	1989	· Physical

^{*}Subsequent five-year interval inspections repeat this pattern.

 Unit 1 - Amendment ±69 196
 15.4.4-1

 Unit 2 - Amendment ±73 201
 15.4.4-1



WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 196

TO FACILITY OPERATING LICENSE NO. DPR-24

AND AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By application dated January 19, 2000, the Wisconsin Electric Power Company (the licensee) requested changes to the Technical Specifications (TSs) for Point Beach Nuclear Plant, Units 1 and 2. The proposed changes would revise TS 15.4.4-II.A to clarify that a different primary containment tendon may be designated a control tendon providing that the new control tendon has not previously been physically changed (e.g., retensioned).

2.0 EVALUATION

2.1 Background

The surveillance testing performed on the containment tendons in accordance with TS 15.4.4-II is intended to ensure that the prestressing tendon loads are maintained within acceptable values. As part of this surveillance program, one tendon from each group of tendons is designated as the control tendon. The intent of the program is that the control tendons remain physically unchanged (e.g., they have not been retensioned) in order to develop a history for, and correlate the observed data from, the associated tendon group. This program is also discussed in NRC Regulatory Guide (RG) 1.35, "Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containments," Revision 3, dated July 1990.

At Point Beach, Units 1 and 2, all of the original control tendons have been retensioned as part of the surveillance program. The licensee recognized that the use of retensioned tendons as control tendons would not yield the historical data concerning relaxation of the tendons that was intended in the TS and RG 1.35. In a letter dated June 29, 1999, the licensee indicated that it had designated new control tendons (that had not been retensioned) for each group. The licensee indicated that it believed the designation of these new control tendons would allow it to obtain useful history data for the relaxation of the tendons.

2.2 Licensee Request

The current TS requirements for the surveillance of Point Beach containment tendons are based on the provisions of RG 1.35, Revision 3. TS 15.4.4-II.A is based on regulatory position 2.4 of RG 1.35. The TS states in part, "The tendons for inspection shall be randomly but representatively selected from each group for each inspection; however, to develop a history and to correlate the observed data, <u>one tendon from each group shall be kept</u> <u>unchanged after initial selection</u>" (emphasis added). In its January 19, 2000, application, the licensee is proposing to change the underlined phrase to, "one tendon from each group shall be physically unchanged."

2.3 Staff Evaluation

The licensee's proposed change to the wording in the TS arises as a result of a lack of clarity in the TS requirement to keep the control tendon "unchanged." As used in the TS, "unchanged" could mean either the control tendon initially selected for periodic inspection must remain the control tendon for all the subsequent inspections, or it could mean that the physical characteristics (as-installed condition) of the control tendon must not be changed by detensioning or retensioning, or both. In the "Basis for Change" included in the January 19, 2000, application, the licensee explains that during earlier inspections, the selected tendons had to be detensioned and retensioned under the TS surveillance, which was based on Revision 1 of the RG. In other words, the tendons had to be physically changed. Furthermore, the licensee states, "It is our intent that the newly selected control tendons will not be changed and will henceforth remain the designated control tendons. However, if a control tendon is found to need retensioning, another nearby tendon will be selected and designated as a control tendon."

The staff finds that the licensee's proposed change is consistent with the intent of Revision 3 of the RG. The word "unchanged" was intended to ensure that the prestress force in the control tendon be measured periodically, without detensioning or retensioning, to establish a continuous history of tendon force. It was not the intent of the RG to limit the control tendon only to the initially selected tendon if unexpected conditions or events warrant the selection of different control tendon.

The staff further notes that the requirement in paragraph IWL-2521(b) of Subsection IWL of Section XI of the American Society of Mechanical Engineers Code (1992 thru 1996 addenda) provides flexibility in selecting the common tendon (same meaning as "control tendon" in the RG and the TS) when conditions warrant such a change in designating a common tendon. Thus, the staff finds that the licensee's proposed change, as explained in its "Basis for Change," meets the intent of the RG, and is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (65 *FR* 12295). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Kugler

Date: June 27, 2000