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ROBERT C. MECREDY
Vice President
Nuclear Operations

June 22, 2000

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: 10CFR50.46 Annual ECCS Report
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Ref. (a): Westinghouse letter RGE-00-202
Subject: 10CFR50.46, Annual Notification and Reporting for 1999,
dated February 23, 2000.

Dear Mr. Vissing:

In accordance with the requirement in 10CFR50.46, paragraph (a) (3) (ii), this annual ECCS report is hereby submitted.

Westinghouse, provider of LOCA analysis for the R.E. Ginna Nuclear Power Plant, has provided RG&E with an update to the peak cladding temperature (PCT) margin for Ginna Station (Reference a). There has been no change to the large break LOCA PCT. The large break LOCA PCT is 2097°F and is summarized in Attachment 1 to this letter.

There has been no change to the small break LOCA PCT. The small break LOCA PCT is 1333°F and is summarized in Attachment 1 to this letter.

Very truly yours,



Robert C. Mecredy

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Attachments

**xc: Mr. Guy S. Vissing (Mail Stop 8C2)
Project Directorate I-1
Washington, D.C. 20555**

**Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406**

U.S. NRC Ginna Senior Resident Inspector

ATTACHMENT I
LOCA PCT SUMMARY
JUNE 2000 UPDATE

ATTACHMENT I

LOCA PT SUMMARY

Large Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: UPI SECY
 $F_Q = 2.45$ $F_{\Delta H} = 1.75$

Fuel: OFA
SGTP = 15%

- | | | |
|----|---|----------------------------|
| A. | Analysis of Record (5/95) (effective 6/96) | PCT = 2051° F |
| B. | 1995 10CFR50.46 Model Assessments | |
| | 1. Fixed heat transfer node assignment
Error/Accumulator water injection error | $\Delta PCT = 48^\circ F$ |
| C. | 1996 10CFR50.46 Model Assessments | |
| | 1. None | $\Delta PCT = 0^\circ F$ |
| D. | 1997 10CFR50.46 Model Assessment | |
| | 1. Accumulator Initial Water Volume
Restart Data Transportation Error
Plant Specific Analytical Reassessment
of 1995 Model Assessments | $\Delta PCT = 58^\circ F$ |
| | 2. Accumulator Initial Water
Vol. = 1125 ft ³ | $\Delta PCT = -25^\circ F$ |
| | 3. 1-D Transition Boiling Heat Transfer Error | $\Delta PCT = -13^\circ F$ |
| | 4. Vessel Channel DX Error | $\Delta PCT = 18^\circ F$ |
| | 5. Input Consistency | $\Delta PCT = -41^\circ F$ |
| E. | 1998 10CFR50.46 Model Assessments | |
| | 1. None | $\Delta PCT = 0^\circ F$ |
| F. | 1999 10CFR50.46 Model Assessments | |
| | 1. None | $\Delta PCT = 0^\circ F$ |
| G. | 10CFR50.59 Evaluations | |
| | 1. Service Water Temp. $\geq 30^\circ$
(1997 evaluation; SEV-1090) | $\Delta PCT = 1^\circ F$ |
| H. | Other Margin Allocations | |
| | 1. None | $\Delta PCT = 0^\circ F$ |

Licensing Basis

PCT = 2097° F

Revision Date: 6/2000

ATTACHMENT I

LOCA PT SUMMARY

Small Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: NOTRUMP Fuel: OFA
 $F_Q = 2.50$ $F_{\Delta H} = 1.75$ SGTP = 15%

- | | | |
|----|--|-----------------------------|
| A. | Analysis of Record (6/95) (effective 6/96) | $\Delta PCT = 1308^\circ F$ |
| B. | 1995 10CFR50.46 Model Assessments | |
| | 1. Notrump Specific Enthalpy Error | $\Delta PCT = 20^\circ F$ |
| | 2. SALIBRARY Double Precision Errors | $\Delta PCT = -15^\circ F$ |
| C. | 1996 10CFR50.46 Model Assignments | |
| | 1. SBLOCA Fuel Rod Initialization Error | $\Delta PCT = 10^\circ F$ |
| D. | 1997 10CFR50.46 Model Assessment | |
| | 1. None | $\Delta PCT = 0^\circ F$ |
| E. | 1998 10CFR50.46 Model Assessments | |
| | 1. None | $\Delta PCT = 0^\circ F$ |
| F. | 1999 10CFR50.46 Model Assessments | |
| | 1. None | $\Delta PCT = 0^\circ F$ |
| G. | 10CFR50.59 Evaluations | |
| | 1. Annular Axial Pellets | $\Delta PCT = 10^\circ F$ |
| | (1997 evaluation; SEV-1108) | |
| H. | Other Margin Allocations | |
| | 1. None | $\Delta PCT = 0^\circ F$ |

Licensing Basis

PCT = 1333°F

Revision Date: 6/2000