

Commonwealth Edison Company  
Quad Cities Generating Station  
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Tel 309-654-2241



June 22, 2000

SVP-00-111

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

Quad Cities Nuclear Power Station, Unit 1  
Facility Operating License No. DPR-29  
NRC Docket No. 50-254

**Subject:** Failure to Demonstrate Operability of Offsite Power Sources In  
Response to Emergency Diesel Generator Inoperability in Timeframe  
Required by Technical Specifications

Enclosed is Licensee Event Report (LER) 254/00-005, Revision 00, for Quad Cities  
Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal  
Regulations, Title 10, Part 50.73(a)(2)(i)(B). The licensee shall report any operation or  
condition prohibited by the Plant's Technical Specifications.

We are committing to the following actions:

A Technical Specification (TS) operations scorecard that establishes specific  
standards and expectations of TS review, compliance and accountability and  
provides for ongoing evaluation of these standards and expectations will be  
developed.

Each Shift Manager will perform the TS operations scorecard at least once  
each shift rotation for 60 days and the data from these scorecards will be  
evaluated by senior management to determine future periodicity and any  
further corrective actions warranted.

The Emergency Diesel Generator Outage Report procedures  
(QCOS 6600-11, QCOS 6600-12, and QCOS 6600-13) will be revised to  
provide for separation of the performance of required TS 1 hour and 8 hour  
actions into two distinct action steps.

The Shift Manager will provide increased overview of the Unit Supervisor  
involved in the event to ensure the effectiveness of the remedial coaching,  
and will provide an individual performance summary to the Shift Operations  
Supervisor.

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Any other actions described in the submittal represent intended or planned actions by Commonwealth Edison (ComEd) Company. They are described for the NRC's information and are not regulatory commitments.

Should you have any questions concerning this letter, please contact Mr. C.C. Peterson at (309) 654-2241, extension 3609.

Respectfully,

A handwritten signature in cursive script that reads "Joel P. Dimmette, Jr." followed by a flourish.

Joel P. Dimmette, Jr.  
Site Vice President  
Quad Cities Nuclear Power Station

cc: Regional Administrator – NRC Region III  
NRC Senior Resident Inspector – Quad Cities Nuclear Power Station

**LICENSEE EVENT REPORT (LER)**

Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the Information and Records Management Branch (F-6 f33), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office Of Management And Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1) <b>Quad Cities Nuclear Power Station, Unit 1</b>	DOCKET NUMBER (2) <b>05000254</b>	PAGE (3) <b>1 of 4</b>
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TITLE (4)  
**Failure to Demonstrate Operability of Offsite Power Sources In Response to Emergency Diesel Generator Inoperability in Timeframe Required by Technical Specifications**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MON TH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	23	2000	2000	005	00	06	22	2000	N/A	N/A
									N/A	N/A

OPERATING MODE (9) <b>1</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more) (11)									
POWER LEVEL (10) <b>100</b>	20.2201(b)	20.2203(a)(2)(v)	X	50.73(a)(2)(i)	50.73(a)(2)(viii)					
	20.2203(a)(i)	20.2203(a)(3)(i)		50.73(a)(2)(ii)	50.73(a)(2)(x)					
	20.2203(a)(2)(i)	20.2203(a)(3)(ii)		50.73(a)(2)(iii)	73.71					
	20.2203(a)(2)(ii)	20.2203(a)(4)		50.73(a)(2)(iv)	OTHER					
	20.2203(a)(2)(iii)	50.36(c)(1)		50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A					
	20.2203(a)(2)(iv)	50.36(c)(2)		50.73(a)(2)(vii)						

**LICENSEE CONTACT FOR THIS LER (12)**

NAME <b>Charles Peterson, Regulatory Assurance Manager</b>	TELEPHONE NUMBER (include Area Code) <b>(309) 654-2241 ext 3609</b>
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**COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)**

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

**SUPPLEMENTAL REPORT EXPECTED (14)**

YES (If yes, complete EXPECTED SUBMISSION DATE.)	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i. e., approximately 15 single-spaced typewritten lines) (16)

On May 23, 2000, at 0837 hours, the Unit 1 Emergency Diesel Generator (EDG) was taken out of service (OOS) for scheduled quarterly and six-year preventive maintenance. During the afternoon shift it was discovered that the demonstration of operability of offsite power sources required by Technical Specifications (TS) to be performed within one hour had not been performed. The demonstration of operability of offsite power sources was completed satisfactorily at 1600 hours. At 1921 hours the EDG was declared operable.

The root causes of the missed TS surveillance are inadequate procedural adherence and weaknesses in the sensitivity, review and management of TS on the part of the Unit Supervisor involved in this event.

The safety significance of the event was minimal. Although the demonstration of the operability of offsite circuit power sources was not performed within the timeframe required by TS, offsite circuit power sources were operable, the Unit 1/2 EDG was operable, and no physical work was being performed in the switchyard.



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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

- Weaknesses in the sensitivity, review and management of TS on the part of the Unit Supervisor involved in this event.

**D. SAFETY ANALYSIS**

The safety significance of the event was minimal. Although the demonstration of the operability of offsite circuit power sources was not performed within the timeframe required by TS, offsite circuit power sources were operable, as verified when the surveillance was performed. In addition, the Unit 1/2 EDG was operable throughout the event, and no physical work was being performed in the switchyard.

**E. CORRECTIVE ACTIONS:**

Corrective Actions Completed:

The demonstration of offsite circuit power sources was performed satisfactorily.

Remedial coaching was provided for the Unit Supervisor involved in the event.

Corrective Actions to be Completed:

A TS operations scorecard that establishes specific standards and expectations of TS review, compliance and accountability and provides for ongoing evaluation of these standards and expectations will be developed.

Each Shift Manager will perform the TS operations scorecard at least once each shift rotation for 60 days and the data from these scorecards will be evaluated by senior management to determine future periodicity and any further corrective actions warranted.

The Emergency Diesel Generator Outage Report procedures (QCOS 6600-11, QCOS 6600-12, and QCOS 6600-13) will be revised to provide for separation of the performance of required TS 1 hour and 8 hour actions into two distinct action steps.

The Shift Manager will provide increased overview of the Unit Supervisor involved in the event to ensure the effectiveness of the remedial coaching, and will provide an individual performance summary to the Shift Operations Supervisor.

**F. PREVIOUS OCCURRENCES:**

There have been two Licensee Event Reports (LERs) written at Quad Cities Nuclear Power Station in the last two years involving missed TS surveillances.

LER 254/98-021, "Weekly Position Verification of Reactor Building to Suppression Chamber Check Valve Vacuum Breakers was Misinterpreted and Not Performed Since the Implementation of Upgraded Technical Specification (TS) Due to Inadequate Development of the Specification Prior to Implementation," dated September 18, 1998, and

LER 254/99-001, "A Technical Specification Surveillance for Primary Containment Isolation Valves was Not Met When Three Spare Manual Isolation Valves Were Inappropriately Deleted From the Surveillance Procedure due to Inadequate Procedure Revision Review," dated May 12, 1999.

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TEXT (if more space is required, use additional copies of NRC Form 366A) (17)

Both of these events involved changes; one to procedures and one to TS. The corrective actions would not have prevented this event.

**G. COMPONENT FAILURE DATA:**

There was no component failure associated with this event.