

July 17, 2000

Mr. Michael B. Roche
Vice President and Director
GPU Nuclear, Inc.
Oyster Creek Nuclear Generating Station
P.O. Box 388
Forked River, NJ 08731

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - ISSUANCE OF
AMENDMENT RE: TECHNICAL SPECIFICATION CHANGE REQUEST
NO. 267 (TAC NO. MA5662)

Dear Mr. Roche:

The Commission has issued the enclosed Amendment No. 211 to Facility Operating License No. DPR-16 for the Oyster Creek Nuclear Generating Station (OCNGS), in response to your application dated April 15, 1999, as supplemented by your letters dated December 22, 1999, and February 24, 2000.

The amendment editorially revises a number of Technical Specifications (TSs) to enhance clarity. A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

The staff has identified a number of issues regarding the quality of your licensing submittals that impact our ability to effectively and efficiently complete our review of your requested licensing actions. These issues include the completeness of your change descriptions and justifications, correct reflection of proposed changes in the revised TS pages submitted with your requests, and the adequacy of your evaluation of the no significant hazards consideration determination criteria of 10 CFR 50.92. Resolution of these concerns would result in resource savings for both the NRC and GPU Nuclear, Inc. staff by reducing the need for supplemental submittals to address deficiencies. These issues have been discussed with your staff.

M. B. Roche

- 2 -

I am available at your convenience to discuss these concerns further. Please contact me at (301) 415-1261 to discuss these issues or any questions regarding the enclosed amendment.

Sincerely,

/RA/

Helen N. Pastis, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures: 1. Amendment No. 211 to DPR-16
2. Safety Evaluation

cc w/encls: See next page

M. B. Roche

- 2 -

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**See previous concurrence

GPU NUCLEAR, INC.

AND

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear, Inc. et al., (the licensee), dated April 15, 1999, as supplemented December 22, 1999, and February 24, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-16 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 211, are hereby incorporated in the license. GPU Nuclear, Inc. shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Marsha Gamberoni, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: July 17, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 211

FACILITY OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
1.0-3	1.0-3
1.0-4	1.0-4
2.3-4	2.3-4
2.3-5	2.3-5
2.3-6	2.3-6
2.3-7	2.3-7
2.3-8	2.3-8
3.4-1	3.4-1
3.4-2	3.4-2
3.5-7	3.5-7
3.5-9	3.5-9
3.5-11	3.5-11
3.7-1	3.7-1
3.17-1	3.17-1
4.5-13	4.5-13

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 211
TO FACILITY OPERATING LICENSE NO. DPR-16
GPU NUCLEAR, INC. AND
JERSEY CENTRAL POWER & LIGHT COMPANY
OYSTER CREEK NUCLEAR GENERATING STATION
DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated April 15, 1999, as supplemented by letters dated December 22, 1999, and February 24, 2000, GPU Nuclear, Inc. (the licensee) submitted a request for revisions to the Oyster Creek Nuclear Generating Station (Oyster Creek) Technical Specifications (TSs). The proposed revisions would clarify a number of the TSs. These revisions resulted from the licensee's review of the Oyster Creek TSs. The December 22, 1999, and February 24, 2000, letters provided clarifying information within the scope of the original application and did not change the staff's initial proposed no significant hazards consideration determination.

2.0 EVALUATION

This amendment request revises a number of items in Sections 2 and 3 of the TSs, expands two definitions in Section 1 and revises the Bases statements in Sections 2, 3, and 4, as explained below.

1. Page 1.0-3 involves TS 1.14, "Secondary Containment Integrity". The current TS reads:

"Secondary containment integrity means that the reactor building is closed and the following conditions are met:

A. At least one door at each access opening is closed."

The licensee proposes to add a note to TS1.14 A and it would read as follows:

"A. At least one door at each access opening is closed .
(Note: Momentary opening and closing of the trunnion room does not constitute a loss of secondary containment integrity)"

The licensee's proposed addition of this note clarifies that secondary containment integrity is not affected by the momentary opening and closing of the trunnion room door. The staff finds this addition acceptable because it differentiates between the opening of personnel and equipment passages into the reactor building and opening of the trunnion room door. To prevent loss of vacuum from the reactor building each personnel and equipment passage door is designed with airlocks. The trunnion room door does not have an airlock because it provides access only to the trunnion room and not to the remainder of the reactor building. The trunnion room leads through one normally locked door and provides access to two outboard main steam isolation valves (MSIVs) and feedwater check valves. Traffic through the trunnion room door is limited and administratively controlled because the room has been designated as a "locked high radiation door". Thus, the staff agrees with the licensee's assertion that momentary opening and closing of the trunnion room door will not constitute a loss of secondary containment integrity.

2. Page 1.0-4 would be revised to expand the definition of the Facility Description and Safety Analysis Report (FDSAR) to indicate that there are a total of 79 amendments included in the FDSAR. This revision is simply editorial and does not impact safety.

3. Page 2.3-5: A previous amendment added two paragraphs to this Bases section. They were meant to replace two other paragraphs, but the existing paragraphs were not deleted at that time. The licensee proposes to delete and move the appropriate paragraphs. This revision would affect the pagination of the remaining pages of the Bases. The staff finds that the licensee revised the Bases appropriately.

4. Pages 2.3-4 and 2.3-7: The licensee proposes to add information to the Bases to expand and clarify the derivation of the limiting safety system settings.

First, the licensee proposes to add the statement "and the unnecessary challenge to the operators" to the sentence which explains the adverse effects on reactor safety of reducing the operating margin before the fuel clad integrity safety limit is reached. Previously, the Bases included only the adverse effect of the resulting thermal stresses. Extending the Bases to include both the adverse effects of the resulting thermal stresses and the unnecessary challenge to the operators from reducing the operating margin is appropriate. Reactor vessels are designed to experience a set number of thermal cycles without affecting reactor safety. Therefore, reducing potential thermal cycles that the reactor vessel experiences by maintaining the current operating margin prevents the licensee from exceeding the design value. Similarly, operator error has been shown to be a contributor to reduced reactor safety. Therefore, reducing the number of times the operators are challenged with a transient, reduces the potential for the operators to contribute to a reduction in reactor safety.

Second, the licensee proposes to clarify the low-level water level trip set-point analysis. The licensee proposed to add this clarification as a result of an NRC engineering team inspection conducted on February 23 to April 2, 1998 (reference 1). During the inspection, the NRC asked questions of the licensee about the low-level water level trip setting analysis because the licensee used a fuel length of 144 inches on the original analysis. However, the current fuel is 145.24 inches long. After reviewing the calculation, the licensee determined that the change in fuel length does not affect the results of the analysis because the original fuel utilized enriched uranium through the entire length of the fuel, whereas the current longer fuel utilizes natural uranium in the top 6 inches of the fuel rod. Thus, the change in the fuel length with the

and losing cattle in a 60-day period. The State of Texas has a population of approximately 28 million people, with the next 36 hours within the next 36 hours from the 2013-2014, the

limited and the population of the State of Texas is approximately 28 million people, with the next 36 hours within the next 36 hours from the 2013-2014, the

excluded from the license proposal

