



Carolina Power & Light Company

Robinson Nuclear Plant
3581 West Entrance Road
Hartsville SC 29550

Serial: RNP-RA/00-0108

JUN 22 2000

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23

**TRANSMITTAL OF COPIES OF AMENDMENT 187 TO
THE OPERATING LICENSE AND TECHNICAL SPECIFICATIONS
AND REVISION 15 TO THE TECHNICAL SPECIFICATIONS BASES**

Ladies and Gentlemen:

This letter transmits copies of replacement pages to the Operating License (OL), Technical Specifications (TS), TS Bases, and associated Lists of Effective Pages (LEPs) for the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. The attached copies incorporate Amendment No. 187 to the OL and TS, and incorporate Revision 15 to the TS Bases. Amendment No. 187 revised Limiting Condition for Operations (LCO) 3.7.8, "Ultimate Heat Sink," and other Technical Specifications to raise the allowed limit for Service Water temperature to 97°F. This amendment was implemented on May 24, 2000. Revision 15 to the TS Bases represents changes associated with Amendment No. 187. Revision 15 to the TS Bases also clarified channel set operability for the Containment Spray actuation logic. This change to the TS Bases was approved in accordance with TS 5.5.14, "Technical Specifications (TS) Bases Control Program. A description of the change is described in Attachment I.


Attachment II provides copies of replacement pages to the LEP for the OL and TS, the facility OL, the TS, the LEP for the TS Bases, and the TS Bases, and includes instructions for removing and inserting the pages on the cover page.

One (1) copy is provided for Mr. B. R. Bonser at NRC Region II, one (1) copy is provided for the NRC Resident Inspector, and three (3) copies are provided for Mr. R. Subbaratnam at NRC Headquarters. The recipients are requested to remove and insert pages in accordance with the instructions provided.

ADD

If you have any questions concerning this matter, please contact Mr. H. K. Chernoff.

Sincerely,


R. L. Warden
Manager - Regulatory Affairs

ALG/alg

Attachments

- I. Summary of Change to Technical Specifications Bases in Revision 15
 - II. Instructions for Removal and Insertion of Pages to the Lists of Effective Pages and Technical Specifications Bases
- c:
- L. A. Reyes, NRC, Region II (w/o attachment)
 - B. R. Bonser, NRC, Region II
 - R. Subbaratnam, NRC, NRR (3 copies attachment)
 - NRC Resident Inspector, HBRSEP

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

SUMMARY OF CHANGE TO TECHNICAL SPECIFICATIONS BASES IN REVISION 15

Description of Change and Technical Justification

The following is a change to the Bases to Technical Specifications that is in addition to the changes reviewed and approved as part of Amendment No. 187.

Bases to Limiting Condition for Operation (LCO) 3.3.2, "Engineered Safety Features Actuation System (ESFAS)"

The Containment Spray signal provides protection against a LOCA or an SLB inside containment. The transmitters (d/p cells) are located outside of containment with the sensing line (high pressure side of the transmitter) located inside containment. The transmitters and electronics are located outside of containment. Two-out-of-three logic, on two sets of three (total of six channels), is used to generate the Containment Pressure- High High signal. Technical Specifications Table 3.3.2-1 indicates that six channels in 2 sets of three are required to be operable.

The Technical Specifications (TS) Bases have been clarified to consistently discuss the arrangement of two sets of three channels in the APPLICABLE SAFETY ANALYSES, LCO, and APPLICABILITY discussions, to consistently discuss that one channel per set can be placed in trip status, and to eliminate the distinction between "failed" and out-of-service channels.

The Improved TS submittal, dated August 27, 1996, revised the operability requirement from 3 per set to "6 (2 sets of 3)." This change was an administrative change to the previous TS. However, the TS Bases discussion of channels in the ACTIONS paragraphs was inconsistent with the discussion in the APPLICABLE SAFETY ANALYSES, LCO, and APPLICABILITY paragraphs. The Bases were revised to consistently describe the operability requirement throughout in terms of channel sets. The clarification is consistent with the requirements as originally licensed in the TS. Therefore, this change to the TS Bases is acceptable.

Remove

Insert

Technical Specifications Bases
Effective Pages

Page i.1
Page i.4
Page i.5
Page i.9
Page i.10
Page i.11

Page i.1
Page i.4
Page i.5
Page i.9
Page i.10
Page i.11

Technical Specifications Bases

Page B 3.3-69
-
Page B 3.3-78
Page B 3.3-82
Page B 3.6-2
Page B 3.6-7
Page B 3.6-26
Page B 3.6-30
Page B 3.6-31
Page B 3.6-35
Page B 3.6-36
Page B 3.6-52
Page B 3.7-8
Page B 3.7-13
Page B 3.7-19
Page B 3.7-37
Page B 3.7-42
Page B 3.7-49
Page B 3.7-50
Page B 3.7-50a
Page B 3.7-51

Page B 3.3-69
Page B 3.3-69a
Page B 3.3-78
Page B 3.3-82
Page B 3.6-2
Page B 3.6-7
Page B 3.6-26
Page B 3.6-30
Page B 3.6-31
Page B 3.6-35
Page B 3.6-36
Page B 3.6-52
Page B 3.7-8
Page B 3.7-13
Page B 3.7-19
Page B 3.7-37
Page B 3.7-42
Page B 3.7-49
Page B 3.7-50
--
Page B 3.7-51

Remove

Insert

Technical Specifications Bases
Effective Pages

Page i.1
Page i.4
Page i.5
Page i.9
Page i.10
Page i.11

Page i.1
Page i.4
Page i.5
Page i.9
Page i.10
Page i.11

Technical Specifications Bases

Page B 3.3-69
-
Page B 3.3-78
Page B 3.3-82
Page B 3.6-2
Page B 3.6-7
Page B 3.6-26
Page B 3.6-30
Page B 3.6-31
Page B 3.6-35
Page B 3.6-36
Page B 3.6-52
Page B 3.7-8
Page B 3.7-13
Page B 3.7-19
Page B 3.7-37
Page B 3.7-42
Page B 3.7-49
Page B 3.7-50
Page B 3.7-50a
Page B 3.7-51

Page B 3.3-69
Page B 3.3-69a
Page B 3.3-78
Page B 3.3-82
Page B 3.6-2
Page B 3.6-7
Page B 3.6-26
Page B 3.6-30
Page B 3.6-31
Page B 3.6-35
Page B 3.6-36
Page B 3.6-52
Page B 3.7-8
Page B 3.7-13
Page B 3.7-19
Page B 3.7-37
Page B 3.7-42
Page B 3.7-49
Page B 3.7-50
--
Page B 3.7-51

OPERATING LICENSE EFFECTIVE PAGES

<u>Page</u>	<u>Rev.</u>
Title Page	-
List of Effective Pages	
1	187
2	187
3	187
4	187
5	187
6	187
7	187
8	187
Technical Specifications	
Interpretations LEP	1

<u>Facility Operating License</u>	<u>Amendment No.</u>
DPR-23	
1	-
2	-
3	187
4	-
4a	-
4b	-
4c	-
4d	187

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

Appendix A

<u>Page</u>	<u>Amendment No.</u>
i	176
ii	176
iii	176
iv	176
v	176
1.1-1	176
1.1-2	176
1.1-3	176
1.1-4	176
1.1-5	176
1.1-6	176
1.2-1	176
1.2-2	176
1.2-3	176
1.3-1	176
1.3-2	176
1.3-3	176
1.3-4	176
1.3-5	176
1.3-6	176
1.3-7	176
1.3-8	176
1.3-9	176
1.3-10	176
1.3-11	176
1.3-12	176
1.3-13	176
1.4-1	176
1.4-2	176
1.4-3	176
1.4-4	176
2.0-1	176
2.0-2	176
3.0-1	176
3.0-2	176
3.0-3	176
3.0-4	176
3.0-5	176
3.1-1	176
3.1-2	176
3.1-3	176
3.1-4	176
3.1-5	176
3.1-6	176
3.1-7	176

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

Appendix A

<u>Page</u>	<u>Amendment No.</u>
3.1-8	176
3.1-9	176
3.1-10	176
3.1-11	176
3.1-12	176
3.1-13	176
3.1-14	176
3.1-15	176
3.1-16	176
3.1-17	176
3.1-18	176
3.1-19	176
3.1-20	176
3.1-21	176
3.2-1	176
3.2-2	176
3.2-3	176
3.2-4	176
3.2-5	176
3.2-6	176
3.2-7	176
3.2-8	176
3.2-9	176
3.2-10	176
3.2-11	176
3.2-12	176
3.2-13	176
3.2-14	176
3.3-1	176
3.3-2	176
3.3-3	176
3.3-4	176
3.3-5	176
3.3-6	176
3.3-7	176
3.3-8	176
3.3-9	176
3.3-10	176
3.3-11	176
3.3-12	176
3.3-13	176
3.3-14	176
3.3-15	176
3.3-16	176
3.3-17	176
3.3-18	176
3.3-19	176
3.3-20	176

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

Appendix A

<u>Page</u>	<u>Amendment No.</u>
3.3-21	176
3.3-22	176
3.3-23	176
3.3-24	176
3.3-25	176
3.3-26	187
3.3-27	187
3.3-28	176
3.3-29	176
3.3-30	176
3.3-31	176
3.3-32	176
3.3-33	176
3.3-34	176
3.3-35	176
3.3-36	176
3.3-37	176
3.3-38	176
3.3-39	176
3.3-40	176
3.3-41	176
3.3-42	176
3.3-43	176
3.3-44	176
3.3-45	176
3.3-46	176
3.3-47	176
3.4-1	176
3.4-2	176
3.4-3	176
3.4-4	176
3.4-5	176
3.4-6	176
3.4-7	176
3.4-8	176
3.4-9	176
3.4-10	176
3.4-11	176
3.4-12	176
3.4-13	176
3.4-14	176
3.4-15	176
3.4-16	176
3.4-17	176
3.4-18	176
3.4-19	176
3.4-20	176
3.4-21	176
3.4-22	176

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

Appendix A

<u>Page</u>	<u>Amendment No.</u>
3.4-23	176
3.4-24	176
3.4-25	176
3.4-26	176
3.4-27	176
3.4-28	176
3.4-29	176
3.4-30	176
3.4-31	176
3.4-32	176
3.4-33	176
3.4-34	176
3.4-35	176
3.4-36	176
3.4-37	176
3.4-38	176
3.4-39	176
3.4-40	182
3.4-41	176
3.4-42	176
3.4-43	176
3.4-44	176
3.4-45	176
3.4-46	176
3.4-47	176
3.4-48	176
3.4-49	176
3.4-50	176
3.4-51	176
3.5-1	176
3.5-2	176
3.5-3	176
3.5-4	176
3.5-5	176
3.5-6	176
3.5-7	176
3.5-8	176
3.5-9	176
3.5-10	176
3.5-11	176
3.6-1	176
3.6-2	176
3.6-3	176
3.6-4	176
3.6-5	176
3.6-6	176
3.6-7	176
3.6-8	176

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

Appendix A

<u>Page</u>	<u>Amendment No.</u>
3.6-9	176
3.6-10	176
3.6-11	176
3.6-12	176
3.6-13	176
3.6-14	176
3.6-15	176
3.6-16	176
3.6-17	176
3.6-18	176
3.6-19	176
3.6-20	187
3.6-21	187
3.7-1	176
3.7-2	176
3.7-3	176
3.7-4	176
3.7-5	176
3.7-6	176
3.7-7	176
3.7-8	176
3.7-9	187
3.7-10	176
3.7-11	176
3.7-12	176
3.7-13	176
3.7-14	176
3.7-15	176
3.7-16	176
3.7-17	186
3.7-18	176
3.7-19	176
3.7-20	176
3.7-21	187
3.7-22	176
3.7-23	176
3.7-24	176
3.7-25	176
3.7-26	176
3.7-27	176
3.7-28	176
3.7-29	176
3.7-30	176
3.7-31	176
3.7-32	176
3.7-33	176
3.8-1	176
3.8-2	176

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

Appendix A

<u>Page</u>	<u>Amendment No.</u>
3.8-3	176
3.8-4	176
3.8-5	176
3.8-6	176
3.8-7	176
3.8-8	176
3.8-9	176
3.8-10	176
3.8-11	176
3.8-12	176
3.8-13	176
3.8-14	176
3.8-15	176
3.8-16	176
3.8-17	176
3.8-18	176
3.8-19	176
3.8-20	176
3.8-21	176
3.8-22	176
3.8-23	176
3.8-24	176
3.8-25	176
3.8-26	176
3.8-27	176
3.8-28	176
3.8-29	176
3.8-30	176
3.8-31	176
3.8-32	176
3.8-33	176
3.8-34	176
3.8-35	176
3.8-36	176
3.9-1	176
3.9-2	180
3.9-3	180
3.9-4	176
3.9-5	176
3.9-6	176
3.9-7	176
3.9-8	176
3.9-9	176
3.9-10	176
3.9-11	176
3.9-12	176
4.0-1	176
4.0-2	176

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

Appendix A

<u>Page</u>	<u>Amendment No.</u>
5.0-1	176
5.0-2	176
5.0-3	176
5.0-4	176
5.0-5	176
5.0-6	176
5.0-7	176
5.0-8	176
5.0-9	176
5.0-10	176
5.0-11	176
5.0-12	176
5.0-13	176
5.0-14	176
5.0-15	176
5.0-16	176
5.0-17	176
5.0-18	176
5.0-19	176
5.0-20	176
5.0-21	176
5.0-22	176
5.0-23	176
5.0-24	187
5.0-25	176
5.0-26	176
5.0-27	176
5.0-28	176
5.0-29	176
5.0-30	176
5.0-31	185
5.0-32	185
5.0-33	176
5.0-34	176

Appendix B

<u>Page</u>	<u>Amendment No.</u>
1	176

3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Section 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at a steady state reactor core power level not in excess of 2300 megawatts thermal.

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 187 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- (1) For Surveillance Requirements (SRs) that are new in Amendment 176 to Final Operating License DPR-23, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 176. For SRs that existed prior to Amendment 176, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 176.

C. Reports

Carolina Power & Light Company shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

Carolina Power & Light Company shall keep facility operating records in accordance with the requirements of the Technical Specifications.

4. Additional Conditions

The Additional Conditions contained in Appendix B, as revised through Amendment No. 187, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the additional conditions.

5. This license is effective as of the date of issuance and shall expire at midnight July 31, 2010.

Attachment
Appendix A - Technical Specifications

Date of Issuance: JUL 31 1970

Table 3.3.2-1 (page 2 of 4)
Engineered Safety Feature Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	NOMINAL TRIP SETPOINT (1)
2. Containment Spray						
a. Manual Initiation	1,2,3,4	2 trains	I	SR 3.3.2.6	NA	NA
b. Automatic Actuation Logic and Actuation Relays	1,2,3,4	2 trains	C	SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.5	NA	NA
c. Containment Pressure						
High High	1,2,3,4	6 (2 sets of 3)	E	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.7	≤ 10.45 psig	10 psig
3. Containment Isolation						
a. Phase A Isolation						
(1) Manual Initiation	1,2,3,4	2	B	SR 3.3.2.6	NA	NA
(2) Automatic Actuation Logic and Actuation Relays	1,2,3,4	2 trains	C	SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.5	NA	NA
(3) Safety Injection	Refer to Function 1 (Safety Injection) for all initiation functions and requirements.					
b. Phase B Isolation						
(1) Manual Initiation	1,2,3,4	2 trains	I	SR 3.3.2.6	NA	NA
(2) Automatic Actuation Logic and Actuation Relays	1,2,3,4	2 trains	C	SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.5	NA	NA
(3) Containment Pressure						
High High	1,2,3,4	6 (2 sets of 3)	E	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.7	≤ 10.45 psig	10 psig

(continued)

- (1) A channel is OPERABLE with an actual Trip Setpoint value found outside its calibration tolerance band provided the Trip Setpoint value is conservative with respect to its associated Allowable Value and the channel is re-adjusted to within the established calibration tolerance band of the Nominal Trip Setpoint.

Table 3.3.2-1 (page 3 of 4)
Engineered Safety Feature Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	NOMINAL TRIP SETPOINT (1)
4. Steam Line Isolation						
a. Manual Initiation	1,2(e),3(e)	1 per steam line	F	SR 3.3.2.6	NA	NA
b. Automatic Actuation Logic and Actuation Relays	1,2(e),3(e)	2 trains	G	SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.5	NA	NA
c. Containment Pressure - High High	1,2(e),3(e)	6 (2 sets of 3)	D	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.7	≤ 10.45 psig	10 psig
d. High Steam Flow in Two Steam Lines	1,2(e),3(e)	2 per steam line	D	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.7	(c)	(d)
Coincident with T _{avg} - Low	1,2(e), 3(e)(b)	1 per loop	D	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.7	≥ 541.50 °F	543°F
e. High Steam Flow in Two Steam Lines	1,2(e),3(e)	2 per steam line	D	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.7	(c)	(d)
Coincident with Steam Line Pressure - Low	1,2(e),3(e)	1 per steam line	D	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.7	≥ 605.05 psig	614 psig

(continued)

- (1) A channel is OPERABLE with an actual Trip Setpoint value found outside its calibration tolerance band provided the Trip Setpoint value is conservative with respect to its associated Allowable Value and the channel is re-adjusted to within the established calibration tolerance band of the Nominal Trip Setpoint.
- (b) Above the T_{avg} - Low interlock.
- (c) Less than or equal to a function defined as ΔP corresponding to 41.58% full steam flow below 20% load, and ΔP increasing linearly from 41.58% full steam flow at 20% load to 110.5% full steam flow at 100% load, and ΔP corresponding to 110.5% full steam flow above 100% load.
- (d) Less than or equal to a function defined as ΔP corresponding to 37.25% full steam flow between 0% and 20% load and then a ΔP increasing linearly from 37.25% steam flow at 20% load to 109% full steam flow at 100% load.
- (e) Except when all MSIVs are closed.

3.6 CONTAINMENT SYSTEMS

3.6.8 Isolation Valve Seal Water (IVSW) System

LC0 3.6.8 The IVSW System shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. IVSW system inoperable.	A.1 Restore IVSW system to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.8.1	Verify IVSW tank pressure is \geq 44.6 psig.	12 hours
SR 3.6.8.2	Verify the IVSW tank volume is \geq 85 gallons.	31 days

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.8.3	Verify the opening time of each air operated header injection valve is within limits.	In accordance with the Inservice Testing Program
SR 3.6.8.4	Verify each automatic valve in the IVSW System actuates to the correct position on an actual or simulated actuation signal.	18 months
SR 3.6.8.5	Verify the IVSW dedicated nitrogen bottles will pressurize the IVSW tank to ≥ 44.6 psig.	18 months
SR 3.6.8.6	Verify IVSW seal header flow rate is: a. ≤ 52.00 cc/minute for Header A, b. ≤ 16.50 cc/minute for Header B, c. ≤ 32.50 cc/minute for Header C, and d. ≤ 23.00 cc/minute for Header D.	18 months

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One or more bypass valves inoperable.	C.1 Close or isolate bypass valve.	8 hours
	<u>AND</u> C.2 Verify bypass valve is closed or isolated.	Once per 7 days
D. Two valves in the same flow path inoperable.	D.1 Isolate affected flow path.	8 hours
E. Required Action and associated Completion Time not met.	E.1 Be in MODE 3.	6 hours
	<u>AND</u> E.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.3.1 Verify the closure time of each MFRV and bypass valve is ≤ 30 seconds on an actual or simulated actuation signal.	In accordance with the Inservice Testing Program
SR 3.7.3.2 Verify the closure time of each MFIV is ≤ 50 seconds on an actual or simulated actuation signal.	In accordance with the Inservice Testing Program

3.7 PLANT SYSTEMS

3.7.8 Ultimate Heat Sink (UHS)

LCO 3.7.8 The UHS shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. UHS inoperable.	A.1 Be in MODE 3.	6 hours
	<u>AND</u>	
	A.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.8.1 Verify water level of UHS is \geq 218 ft mean sea level.	24 hours
SR 3.7.8.2 Verify service water temperature is \leq 97°F.	24 hours

5.5 Programs and Manuals

5.5.16 Containment Leakage Rate Testing Program

This program provides controls for implementation of the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions for Type A testing. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995. Type B and C testing shall be implemented in the program in accordance with the requirements of 10 CFR 50, Appendix J, Option A.

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a , is 40.5 psig.

The maximum allowable containment leakage rate, L_a , at P_a , shall be 0.1% of the containment air weight per day.

Leakage rate acceptance criteria are:

- a. Containment leakage rate acceptance criteria is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for the Type B and Type C tests, and $\leq 0.75 L_a$ for Type A tests.

The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

(continued)

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

<u>Page</u>	<u>Rev.</u>
i	0
i.1	15
i.2	7 E1
i.3	7 E1
i.4	15
i.5	15
i.6	7 E1
i.7	7 E1
i.8	11
i.9	15
i.10	15
i.11	15
i.12	13
i.13	7 E1
ii	0
iii	0
iv	0
B 2.0-1	0
B 2.0-2	0
B 2.0-3	3
B 2.0-4	3
B 2.0-5	3
B 2.0-6	0
B 2.0-7	0
B 2.0-8	0
B 2.0-9	0
B 3.0-1	0
B 3.0-2	0
B 3.0-3	0
B 3.0-4	0
B 3.0-5	0
B 3.0-6	0
B 3.0-7	0
B 3.0-8	0
B 3.0-9	0
B 3.0-10	0
B 3.0-11	0
B 3.0-12	0
B 3.0-13	0
B 3.0-14	0
B 3.0-15	0
B 3.1-1	0
B 3.1-2	0
B 3.1-3	0
B 3.1-4	0
B 3.1-5	0
B 3.1-6	0
B 3.1-7	0
B 3.1-8	0
B 3.1-9	0

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

<u>Page</u>	<u>Rev.</u>
B 3.3-22	0
B 3.3-23	0
B 3.3-24	0
B 3.3-25	0
B 3.3-26	0
B 3.3-27	0
B 3.3-28	0
B 3.3-29	0
B 3.3-30	0
B 3.3-31	0
B 3.3-32	0
B 3.3-33	0
B 3.3-34	0
B 3.3-35	0
B 3.3-36	0
B 3.3-37	0
B 3.3-38	0
B 3.3-39	0
B 3.3-40	0
B 3.3-41	0
B 3.3-42	0
B 3.3-43	0
B 3.3-44	0
B 3.3-45	0
B 3.3-46	0
B 3.3-47	0
B 3.3-48	0
B 3.3-49	0
B 3.3-50	0
B 3.3-51	0
B 3.3-52	0
B 3.3-53	0
B 3.3-54	0
B 3.3-54a	1
B 3.3-55	0
B 3.3-56	0
B 3.3-57	0
B 3.3-58	0
B 3.3-59	0
B 3.3-60	0
B 3.3-61	0
B 3.3-62	0
B 3.3-63	0
B 3.3-64	0
B 3.3-65	0
B 3.3-66	0
B 3.3-67	1
B 3.3-67a	1
B 3.3-68	0
B 3.3-69	15
B 3.3-69a	15
B 3.3-70	0

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

<u>Page</u>	<u>Rev.</u>
B 3.3-71	0
B 3.3-72	0
B 3.3-73	0
B 3.3-74	0
B 3.3-75	0
B 3.3-76	6
B 3.3-77	0
B 3.3-78	15
B 3.3-79	0
B 3.3-80	0
B 3.3-81	0
B 3.3-82	15
B 3.3-83	0
B 3.3-84	0
B 3.3-85	0
B 3.3-86	0
B 3.3-87	0
B 3.3-88	0
B 3.3-89	0
B 3.3-90	0
B 3.3-91	12
B 3.3-92	12
B 3.3-93	12
B 3.3-94	12
B 3.3-95, B 3.3-95a	12
B 3.3-96	0
B 3.3-97	0
B 3.3-98	0
B 3.3-99	0
B 3.3-100	0
B 3.3-101	0
B 3.3-102	0
B 3.3-103	0
B 3.3-104	0
B 3.3-105	0
B 3.3-106	0
B 3.3-107	12
B 3.3-108	0
B 3.3-109	0
B 3.3-110	0
B 3.3-111	0
B 3.3-112	0
B 3.3-113	0
B 3.3-114	0
B 3.3-115	0
B 3.3-116	0
B 3.3-117	0
B 3.3-118	0
B 3.3-119	0
B 3.3-120	0
B 3.3-121	0

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

<u>Page</u>	<u>Rev.</u>
B 3.5-22	0
B 3.5-23	0
B 3.5-24	0
B 3.5-25	0
B 3.5-26	0
B 3.5-27	0
B 3.5-28	0
B 3.5-29	0
B 3.5-30	0
B 3.6-1	0
B 3.6-2	15
B 3.6-3	0
B 3.6-4	0
B 3.6-5	0
B 3.6-6	0
B 3.6-7	15
B 3.6-8	0
B 3.6-9	0
B 3.6-10	0
B 3.6-11	0
B 3.6-12	0
B 3.6-13	0
B 3.6-14	0
B 3.6-15	0
B 3.6-16	0
B 3.6-17	0
B 3.6-18	0
B 3.6-19	0
B 3.6-20	0
B 3.6-21	0
B 3.6-22	0
B 3.6-23	0
B 3.6-24	0
B 3.6-25	0
B 3.6-26	15
B 3.6-27	0
B 3.6-28	0
B 3.6-29	0
B 3.6-30	15
B 3.6-31	15
B 3.6-32	0
B 3.6-33	0
B 3.6-34	0
B 3.6-35	15
B 3.6-36	15
B 3.6-37	0
B 3.6-38	0
B 3.6-39	0
B 3.6-40	0
B 3.6-41	0
B 3.6-42	0

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

<u>Page</u>	<u>Rev.</u>
B 3.6-43	0
B 3.6-44	0
B 3.6-45	0
B 3.6-46	0
B 3.6-47	0
B 3.6-48	0
B 3.6-49	0
B 3.6-50	0
B 3.6-51	0
B 3.6-52	15
B 3.6-53	0
B 3.6-54	0
B 3.7-1	0
B 3.7-2	0
B 3.7-3	0
B 3.7-4	0
B 3.7-5	0
B 3.7-6	0
B 3.7-7	0
B 3.7-8	15
B 3.7-9	0
B 3.7-10	0
B 3.7-11	0
B 3.7-12	0
B 3.7-13	15
B 3.7-14	0
B 3.7-15	0
B 3.7-16	0
B 3.7-17	0
B 3.7-18	0
B 3.7-19	15
B 3.7-20	0
B 3.7-21	0
B 3.7-22	0
B 3.7-23	0
B 3.7-24	0
B 3.7-25	6
B 3.7-25a	6
B 3.7-26	0
B 3.7-27	0
B 3.7-28	0
B 3.7-29	0
B 3.7-30	0
B 3.7-31	0
B 3.7-32	0
B 3.7-33	0
B 3.7-34	0
B 3.7-35	0
B 3.7-36	0
B 3.7-37	15
B 3.7-38	0

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

<u>Page</u>	<u>Rev.</u>
B 3.7-39	14
B 3.7-40	0
B 3.7-41	0
B 3.7-42	15
B 3.7-43	0
B 3.7-44	0
B 3.7-45	0
B 3.7-46	0
B 3.7-47	0
B 3.7-48	0
B 3.7-49	15
B 3.7-50	15
B 3.7-51	15
B 3.7-52	0
B 3.7-53	0
B 3.7-54	6
B 3.7-55	0
B 3.7-56	0
B 3.7-57	0
B 3.7-58	0
B 3.7-59	0
B 3.7-60	6
B 3.7-61	6
B 3.7-62	0
B 3.7-63	6
B 3.7-64	0
B 3.7-65	0
B 3.7-66	0
B 3.7-67	0
B 3.7-68	0
B 3.7-69	0
B 3.7-70	0
B 3.7-71	0
B 3.7-72	0
B 3.7-73	0
B 3.7-74	0
B 3.7-75	0
B 3.7-76	0
B 3.7-77	0
B 3.7-78	0
B 3.7-79	0
B 3.7-80	0
B 3.8-1	0
B 3.8-2	0
B 3.8-3	0
B 3.8-4	0
B 3.8-5	0
B 3.8-6	0
B 3.8-7	0
B 3.8-8	0

BASES

APPLICABLE
SAFETY ANALYSES,
LCO, and
APPLICABILITY
(continued)

c. Containment Spray—Containment Pressure Hi-Hi

This signal provides protection against a LOCA or an SLB inside containment. The transmitters (d/p cells) are located outside of containment with the sensing line (high pressure side of the transmitter) located inside containment. The transmitters and electronics are located outside of containment. Thus, they will not experience any adverse environmental conditions and the Trip Setpoint reflects only steady state instrument uncertainties.

This is the only Function that requires the bistable output to energize to perform its required action. It is not desirable to have a loss of power actuate containment spray, since the consequences of an inadvertent actuation of containment spray could be serious. Therefore, two-out-of-three logic, on two sets of three (total of six channels), is used to generate the Containment Pressure—High High signal. One channel per set may be placed in trip and still maintain adequate margin to spurious spray actuation. Note that this Function has the requirement that no more than one channel per set is permitted to be placed in trip to decrease the probability of an inadvertent actuation.

Containment Pressure—High High must be OPERABLE in MODES 1, 2, 3, and 4 when there is sufficient energy in the primary and secondary sides to pressurize the containment following a pipe break. In MODES 5, and 6, there is insufficient energy in the primary and secondary sides to pressurize the containment and reach the Containment Pressure—High High setpoints.

The Containment Pressure—High High Function also initiates a Main Steam Line Isolation signal, as described in Function 4.d.

(continued)

BASES

APPLICABLE
SAFETY ANALYSES,
LCO, and
APPLICABILITY

3. Containment Isolation (continued)

Containment Isolation provides isolation of the containment atmosphere, and all process systems that penetrate containment, from the environment. This Function is necessary to prevent or limit the release

(continued)

BASES

APPLICABLE
SAFETY ANALYSES,
LCO, and
APPLICABILITY

b. Engineered Safety Feature Actuation System
Interlocks - T_{avg} - Low (continued)

This Function must be OPERABLE in MODES 1, 2, and 3 when a secondary side break or stuck open valve could result in the rapid depressurization of the steam lines. This Function does not have to be OPERABLE in MODE 4, 5, or 6 because there is insufficient energy in the secondary side of the unit to have an accident.

The ESFAS instrumentation satisfies Criterion 3 of the NRC Policy Statement.

ACTIONS

Note 1 has been added in the ACTIONS to clarify the application of Completion Time rules. The Conditions of this Specification may be entered independently for each Function listed on Table 3.3.2-1.

In the event a channel's Trip Setpoint is found nonconservative with respect to the Allowable Value, or the transmitter, instrument Loop, signal processing electronics, or bistable is found inoperable, then all affected Functions provided by that channel must be declared inoperable and the LCO Condition(s) entered for the protection Function(s) affected. When the Required Channels in Table 3.3.2-1 are specified (e.g., on a per steam line, per loop, per SG, etc., basis), then the Condition may be entered separately for each steam line, loop, SG, etc., as appropriate.

When the number of inoperable channels in a trip function exceed those specified in one or other related Conditions associated with a trip function, then the unit is outside the safety analysis. Therefore, LCO 3.0.3 should be immediately entered if applicable in the current MODE of operation.

A.1

Condition A applies to all ESFAS protection functions.

Condition A addresses the situation where one or more channels or trains for one or more Functions are inoperable

(continued)

BASES

ACTIONS

E.1, E.2.1, and E.2.2 (continued)

- Containment Spray Containment Pressure—High High.

None of these signals has input to a control function. Thus, two-out-of-three logic is necessary to meet acceptable protective requirements. However, a two-out-of-three design would require tripping a failed channel. This is undesirable because a single failure would then cause spurious containment spray initiation. Spurious spray actuation is undesirable because of the cleanup problems presented. Therefore, these channels are designed with two-out-of-three on two sets of three logic. One channel per set may be placed in trip and still maintain adequate margin to spurious spray actuation.

To avoid the inadvertent actuation of containment spray and Phase B containment isolation, no more than one channel per set may be placed in trip. Restoring the channel to OPERABLE status, or placing the inoperable channel in trip within 6 hours, is sufficient to assure that the Function remains OPERABLE and minimizes the time that the Function may be in a partial trip condition (assuming the inoperable channel has failed high). The Completion Time is further justified based on the low probability of an event occurring during this interval. Failure to restore the inoperable channel to OPERABLE status, or place it in trip within 6 hours, requires the unit be placed in MODE 3 within the following 6 hours, MODE 4 within the next 6 hours, and MODE 5 within the next 24 hours. The allowed Completion Times are reasonable, based on operating experience, to reach the required unit conditions from full power conditions in an orderly manner and without challenging unit systems. In MODE 5, these Functions are no longer required OPERABLE.

F.1, F.2.1, and F.2.2

Condition F applies to:

- Manual Initiation of Steam Line Isolation.

For the Manual Initiation Function, this action addresses the train orientation of the relay logic. If a train or channel is inoperable, 48 hours are allowed to return it to OPERABLE status. The specified Completion Time is

(continued)

BASES

BACKGROUND
(continued)

- b. The air lock is OPERABLE, except as provided in LCO 3.6.2, "Containment Air Lock";
 - c. The equipment hatch is closed and sealed; and
 - d. The Isolation Valve Seal Water (IVSW) system is OPERABLE, except as provided in LCO 3.6.8.
-

APPLICABLE
SAFETY ANALYSES

The safety design basis for the containment is that the containment must withstand the pressures and temperatures of the limiting DBA without exceeding the design leakage rate.

The DBAs that result in a challenge to containment OPERABILITY from high pressures and temperatures are a loss of coolant accident (LOCA) and a steam line break (Ref. 2). In addition, release of significant fission product radioactivity within containment can occur from a LOCA. In the LOCA analyses, it is assumed that the containment is OPERABLE such that, for the LOCA, the release to the environment is controlled by the rate of containment leakage. The containment was designed with an allowable leakage rate of 0.1% of containment air weight per day (Ref. 2). This leakage rate, used to evaluate offsite doses resulting from accidents, is defined in 10 CFR 50, Appendix J (Ref. 1), as L_a : the maximum allowable containment leakage rate at the calculated peak containment internal pressure (P_a) resulting from the LOCA. The allowable leakage rate represented by L_a forms the basis for the acceptance criteria imposed on all containment leakage rate testing. L_a is assumed to be 0.1% per day in the safety analysis at $P_a = 40.5$ psig (Ref. 2).

Satisfactory leakage rate test results are a requirement for the establishment of containment OPERABILITY.

The containment satisfies Criterion 3 of the NRC Policy Statement.

(continued)

BASES

APPLICABLE
SAFETY ANALYSES
(continued)

leakage. The containment was designed with an allowable leakage rate of 0.1% of containment air weight per day (Ref. 2). This leakage rate is defined in 10 CFR 50, Appendix J (Ref. 1) as $L_a = 0.1\%$ of containment air weight per day, the maximum allowable containment leakage rate at the calculated peak containment internal pressure $P_a = 40.5$ psig following a DBA.

The containment air lock satisfies Criterion 3 of the NRC Policy Statement.

LCO

The containment air lock forms part of the containment pressure boundary. As part of containment, the air lock safety function is related to control of the containment leakage rate resulting from a DBA. Thus, the air lock's structural integrity and leak tightness are essential to the successful mitigation of such an event.

The air lock is required to be OPERABLE. For the air lock to be considered OPERABLE, the air lock interlock mechanism must be OPERABLE, the air lock must be in compliance with the Type B air lock leakage test, and both air lock doors must be OPERABLE. The interlock allows only one air lock door of an air lock to be opened at one time. This provision ensures that a gross breach of containment does not exist when containment is required to be OPERABLE. Closure of a single door in the air lock is sufficient to provide a leak tight barrier following postulated events. Nevertheless, both doors are kept closed when the air lock is not being used for normal entry into and exit from containment.

APPLICABILITY

In MODES 1, 2, 3, and 4, a DBA could cause a release of radioactive material to containment. In MODES 5 and 6, the probability and consequences of these events are reduced due to the pressure and temperature limitations of these MODES. Therefore, the containment air locks are not required in MODE 5 to prevent leakage of radioactive material from containment. The requirements for the containment air locks during MODE 6 are addressed in LCO 3.9.3, "Containment Penetrations."

(continued)

B 3.6 CONTAINMENT SYSTEMS

B 3.6.4 Containment Pressure

BASES

BACKGROUND

The containment pressure is limited during normal operation to preserve the initial conditions assumed in the accident analyses for a loss of coolant accident (LOCA) or steam line break (SLB). These limits also prevent the containment pressure from exceeding the containment design negative pressure differential with respect to the outside atmosphere in the event of inadvertent actuation of the Containment Spray System.

Containment pressure is a process variable that is monitored and controlled. The containment pressure limits are derived from the input conditions used in the containment functional analyses and the containment structure external pressure analysis. Should operation occur outside these limits coincident with a Design Basis Accident (DBA), post accident containment pressures could exceed calculated values.

APPLICABLE SAFETY ANALYSES

Containment internal pressure is an initial condition used in the DBA analyses to establish the maximum peak containment internal pressure. The limiting DBAs considered, relative to containment pressure, are the LOCA and SLB, which are analyzed using computer codes designed to predict the resultant pressure and temperature transient. The containment pressure analysis indicates the containment peak pressure for the limiting SLB slightly exceeds the peak pressure for the limiting LOCA (Ref. 1).

The initial pressure condition used in the containment analysis was 15.7 psia (1.0 psig). This resulted in a maximum peak pressure from a LOCA of 40.5 psig. The containment analysis (Ref. 1) confirms that this calculated peak containment pressure from the limiting LOCA is the same as the P_a determined in accordance with 10 CFR 50, Appendix J, Option B. The maximum containment pressure resulting from the worst case SLB, 41.9 psig, does not exceed the containment design pressure, 42 psig.

The containment was also designed for an external pressure load equivalent to -3.0 psig. The inadvertent actuation of the Containment Spray System was analyzed to determine

(continued)

BASES

APPLICABLE
SAFETY ANALYSES
(continued)

No two DBAs are assumed to occur simultaneously or consecutively. The postulated DBAs are analyzed with regard to Engineered Safety Feature (ESF) systems, assuming the loss of one ESF bus, which is the worst case single active failure, resulting in one train each of the Containment Spray System, Residual Heat Removal System, and Containment Cooling System being rendered inoperable.

The limiting DBA for the maximum peak containment air temperature is an SLB. The initial containment average air temperature assumed in the design basis analyses (Ref. 1) is 130°F. This resulted in a maximum containment air temperature of approximately 274°F. The maximum containment air temperature from a LOCA is approximately 262°F. The environmental qualification temperature limit is 280°F. The containment structural design temperature is 263 °F.

The temperature limit is used to establish the environmental qualification operating envelope for containment. The maximum peak containment air temperature was calculated to exceed the containment design temperature briefly during the transient. The basis of the containment design temperature, however, is to ensure the performance of safety related equipment inside containment (Ref. 2). Thermal analyses showed that the time interval during which the containment air temperature exceeded the containment design temperature was short enough that the equipment surface temperatures remained below the design temperature. Therefore, it is concluded that the calculated transient containment air temperature is acceptable for the DBA SLB.

The temperature limit is also used in the depressurization analyses to ensure that the minimum pressure limit is maintained following an inadvertent actuation of the Containment Spray System (Ref. 1).

The containment pressure transient is sensitive to the initial air mass in containment and, therefore, to the initial containment air temperature. The limiting DBA for establishing the maximum peak containment internal pressure is a LOCA. The temperature limit is used in this analysis to ensure that in the event of an accident the maximum containment internal pressure will not be exceeded.

(continued)

BASES

APPLICABLE SAFETY ANALYSES (continued) Containment average air temperature satisfies Criterion 2 of the NRC Policy Statement.

LCO During a DBA, with an initial containment average air temperature less than or equal to the LCO temperature limit, the resultant peak accident temperature is maintained below the values previously analyzed. As a result, the ability of containment to perform its design function is ensured.

APPLICABILITY In MODES 1, 2, 3, and 4, a DBA could cause a release of radioactive material to containment. In MODES 5 and 6, the probability and consequences of these events are reduced due to the pressure and temperature limitations of these MODES. Therefore, maintaining containment average air temperature within the limit is not required in MODE 5 or 6.

ACTIONS

A.1

When containment average air temperature is not within the limit of the LCO, it must be restored to within limit within 8 hours. This Required Action is necessary to return operation to within the bounds of the containment analysis. The 8 hour Completion Time is acceptable considering the sensitivity of the analysis to variations in this parameter and provides sufficient time to correct minor problems.

B.1 and B.2

If the containment average air temperature cannot be restored to within its limit within the required Completion Time, the plant must be brought to a MODE in which the LCO does not apply. To achieve this status, the plant must be brought to at least MODE 3 within 6 hours and to MODE 5 within 36 hours. The allowed Completion Times are reasonable, based on operating experience, to reach the required plant conditions from full power conditions in an orderly manner and without challenging plant systems.

(continued)

BASES

BACKGROUND

Containment Cooling System (continued)

and incore detector raceway, and outside the secondary shield in the lower areas of containment.

During normal operation, all four fan units may be operating. The fans are normally operated with SW supplied to the cooling coils. The Containment Cooling System, operating in conjunction with the Containment Ventilation system, is designed to limit the ambient containment air temperature during normal unit operation to less than the limit specified in LCO 3.6.5, "Containment Air Temperature." This temperature limitation ensures that the containment temperature does not exceed the initial temperature conditions assumed for the DBAs.

In post accident operation following an actuation signal, the Containment Cooling System fans are designed to start automatically if not already running. The temperature of the SW is an important factor in the heat removal capability of the fan units.

APPLICABLE
SAFETY ANALYSES

The Containment Spray System and Containment Cooling System limit the temperature and pressure that could be experienced following a DBA. The limiting DBAs considered are the loss of coolant accident (LOCA) and the steam line break (SLB). The LOCA and SLB are analyzed using computer codes designed to predict the resultant containment pressure and temperature transients. No DBAs are assumed to occur simultaneously or consecutively. The postulated DBAs are analyzed with regard to containment ESF systems, assuming the loss of one ESF bus, which is the worst case single active failure and results in one train of the Containment Spray System and Containment Cooling System being rendered inoperable.

The analysis and evaluation show that under the worst case scenario, the highest peak containment pressure is 41.9 psig (experienced during a SLB). The analysis shows that the peak containment temperature is approximately 274°F (experienced during an SLB). Both results meet the intent of the design basis. (See the Bases for LCO 3.6.4, "Containment Pressure," and LCO 3.6.5 for a detailed discussion.) The analyses and evaluations assume a power

(continued)

BASES

APPLICABLE
SAFETY ANALYSES
(continued)

level of 102% for the most limiting pressure response and 0% for the most limiting temperature response, a single failure of a steam line check valve, and initial (pre-accident) containment conditions of 130°F and 1.0 psig. The analyses also assume a response time delayed initiation to provide conservative peak calculated containment pressure and temperature responses.

For certain aspects of transient accident analyses, maximizing the calculated containment pressure is not conservative. In particular, the effectiveness of the Emergency Core Cooling System during the core reflood phase of a LOCA analysis increases with increasing containment backpressure. For these calculations, the containment backpressure is calculated in a manner designed to conservatively minimize, rather than maximize, the calculated transient containment pressures in accordance with 10 CFR 50, Appendix K (Ref. 2).

The effect of an inadvertent containment spray actuation has been analyzed. An inadvertent spray actuation results in a -3.0 psig containment pressure and is associated with the sudden cooling effect in the interior of the leak tight containment. Additional discussion is provided in the Bases for LCO 3.6.4.

The modeled Containment Spray System actuation from the containment analysis is based on a response time associated with exceeding the containment High-High pressure setpoint to achieving full flow through the containment spray nozzles.

Containment cooling train performance for post accident conditions is given in Reference 3. The result of the analysis is that each train can provide 100% of the required peak cooling capacity during the post accident condition. The train post accident cooling capacity under varying containment ambient conditions, is also shown in Reference 4. The modeled Containment Cooling System actuation from the containment analysis is based on a response time associated with exceeding the containment high pressure setpoint to achieving full Containment Cooling System air and cooling water flow.

The Containment Spray System and the Containment Cooling System satisfy Criterion 3 of the NRC Policy Statement.

(continued)

BASES

ACTIONS
(continued)

B.1 and B.2

If the Required Actions and associated Completion Times are not met, the plant must be brought to a MODE in which the LCO does not apply. To achieve this status, the plant must be brought to at least MODE 3 within 6 hours and to MODE 5 within 36 hours. The allowed Completion Times are reasonable, based on operating experience, to reach the required plant conditions from full power conditions in an orderly manner and without challenging plant systems

SURVEILLANCE
REQUIREMENTS

SR 3.6.8.1

This SR verifies the IVSW tank has the necessary pressure to provide motive force to the seal water. A pressure ≥ 44.6 psig ensures the containment penetration flowpaths that are sealed by the IVSW System are maintained at a pressure which is at least 1.1 times the calculated peak containment internal pressure (P_a) related to the design bases accident. Verification of the IVSW tank pressure on a Frequency of once per 12 hours is acceptable. This Frequency is sufficient to ensure availability of IVSW. Operating experience has shown this Frequency to be appropriate for early detection and correction of off normal trends.

SR 3.6.8.2

This SR verifies the IVSW tank has an initial volume of water necessary to provide seal water to the containment isolation valves served by the IVSW System. An initial volume ≥ 85 gallons ensures the IVSW System contains the proper inventory to maintain the required seal. Verification of IVSW tank level on a Frequency of once per 31 days is acceptable since tank level is continuously monitored by installed instrumentation and will alarm in the control room prior to level decreasing to 85 gallons.

SR 3.6.8.3

This SR verifies the stroke time of each automatic air operated header injection solenoid valve is within limits. The frequency is specified by the Inservice Testing

(continued)

B 3.7 PLANT SYSTEMS

B 3.7.2 Main Steam Isolation Valves (MSIVs)

BASES

BACKGROUND

The MSIVs isolate steam flow from the secondary side of the steam generators following a high energy line break (HELB). MSIV closure terminates flow from the unaffected (intact) steam generators.

One MSIV is located in each main steam line outside, but close to, containment. The MSIVs are downstream from the main steam safety valves (MSSVs) and auxiliary feedwater (AFW) pump turbine steam supply, to prevent MSSV and AFW isolation from the steam generators by MSIV closure. Closing the MSIVs isolates each steam generator from the others, and isolates the turbine, Steam Dump System, and other auxiliary steam supplies from the steam generators.

The MSIVs close on a main steam isolation signal generated by either high steam flow coincident with low T_{avg} or with low steam pressure; or high-high containment pressure. The MSIVs fail as is on loss of control or actuation power.

A bypass valve is provided around each MSIV to equalize pressure across the valve and to warm up the steam line during unit startup. The bypass valves are motor operated, manually actuated valves, which are normally closed.

A description of the MSIVs is found in the UFSAR, Section 10.3 (Ref. 1).

APPLICABLE SAFETY ANALYSES

The design basis of the MSIVs is established by the containment analysis for the large steam line break (SLB) inside containment, discussed in the UFSAR, Section 6.2 (Ref. 2). It is also affected by the accident analysis of the SLB events presented in the UFSAR, Section 15.1.5 (Ref. 3). The design precludes the blowdown of more than one steam generator, assuming a single active component failure (e.g., the failure of one MSIV to close on demand). Furthermore, the design can limit the blowdown through the break that would occur while the MSIVs are closing. This is due to a check valve installed downstream of each MSIV. Upon a failure of an MSIV, the check valve will prevent

(continued)

BASES

SURVEILLANCE
REQUIREMENTS

SR 3.7.2.1 (continued)

containment analyses with the exception of closure of the MSIVs for a MSLB at 100% RTP, in which case MSIV closure in 2 seconds is assumed for MSIVs which close in the forward flow direction. This Surveillance is normally performed upon returning the unit to operation following a refueling outage. The MSIVs should not be tested at power, since even a part stroke exercise increases the risk of a valve closure when the unit is generating power. As the MSIVs are not tested at power, they are exempt from the ASME Code, Section XI (Ref. 5), requirements during operation in MODE 1 or 2.

The Frequency is in accordance with the Inservice Testing Program. The specified Frequency for valve closure time is based on the refueling cycle. Operating experience has shown that these components usually pass the Surveillance when performed at the specified Frequency. Therefore, the Frequency is acceptable from a reliability standpoint.

This test is conducted in MODE 3 with the unit at operating temperature and pressure, as discussed in Reference 5 exercising requirements. This SR is modified by a Note that allows entry into and operation in MODE 3 prior to performing the SR. This allows a delay of testing until MODE 3, to establish conditions consistent with those under which the acceptance criterion was generated.

REFERENCES

1. UFSAR, Section 10.3.
 2. UFSAR, Section 6.2.
 3. UFSAR, Section 15.1.5.
 4. 10 CFR 100.11.
 5. ASME, Boiler and Pressure Vessel Code, Section XI.
-

BASES (continued)

SURVEILLANCE
REQUIREMENTS

SR 3.7.3.1

This SR verifies that the closure time of each MFRV and bypass valve is ≤ 30 seconds on an actual or simulated actuation signal. The MFRV, and bypass valve closure times are assumed in the accident and containment analyses (Ref. 2). This Surveillance is normally performed upon returning the unit to operation following a refueling outage. These valves should not be tested at power since even a part stroke exercise increases the risk of a valve closure with the unit generating power. This is consistent with the ASME Code, Section XI (Ref. 3).

The Frequency for this SR is in accordance with the Inservice Testing Program. The specified Frequency for valve closure is based on the refueling cycle. Operating experience has shown that these components usually pass the Surveillance when performed at the specified Frequency.

SR 3.7.3.2

This SR verifies that the closure time of each MFIV is ≤ 50 seconds on an actual or simulated actuation signal. The MFIV closure times are assumed in the accident and containment analyses (Ref. 2). This Surveillance is normally performed upon returning the unit to operation following a refueling outage. These valves should not be tested at power since even a part stroke exercise increases the risk of a valve closure with the unit generating power. This is consistent with the ASME Code, Section XI (Ref. 3).

The Frequency for this SR is in accordance with the Inservice Testing Program. The specified Frequency for valve closure is based on the refueling cycle. Operating experience has shown that these components usually pass the Surveillance when performed at the specified Frequency.

(continued)

BASES

APPLICABLE
SAFETY ANALYSES
(continued)

The design basis of the CCW System is for one CCW pump and one CCW heat exchanger to accommodate the post loss of coolant accident (LOCA) heat removal loads from the RHR heat exchangers and safety injection pump seals. Should either a required CCW pump or a CCW heat exchanger fail, one of the two standby pumps and the standby heat exchanger provide 100 percent backup.

The CCW System is designed to perform its function with a single failure of any active component, assuming a loss of offsite power.

The CCW System also functions to cool the unit from RHR entry conditions ($T_{cold} < 350^{\circ}\text{F}$), to MODE 5 ($T_{cold} < 200^{\circ}\text{F}$), during normal and post accident operations. The time required to cool from 350°F to 200°F is a function of service water temperature and the number of CCW and RHR trains operating. One CCW train is sufficient to remove decay heat during subsequent operations with $T_{cold} < 200^{\circ}\text{F}$. This assumes a maximum service water temperature of 97°F occurring simultaneously with the maximum heat loads on the system.

The CCW System satisfies Criterion 3 of the NRC Policy Statement.

LCO

The CCW trains are independent of each other to the degree that each CCW pump has separate controls and power supplies and the operation of one does not depend on the other. In the event of a DBA, one CCW train powered from an emergency power source is required to provide the minimum heat removal capability assumed in the safety analysis for the systems to which it supplies cooling water. To ensure this requirement is met, two trains of CCW powered from an emergency power source must be OPERABLE. At least one CCW train will operate assuming the worst case single active failure occurs coincident with a loss of offsite power.

A CCW train is considered OPERABLE when:

- a. The required pump and heat exchanger are OPERABLE; and

(continued)

BASES

BACKGROUND (continued)

Additional information about the design and operation of the SWS, along with a list of the components served, is presented in the UFSAR, Section 9.2.1 (Ref. 1). The principal safety related function of the SWS is the removal of decay heat from the reactor via the Component Cooling Water (CCW) System.

APPLICABLE SAFETY ANALYSES

The design basis of the SWS is to provide cooling water to those components necessary to remove core decay heat following a design basis LOCA as discussed in the UFSAR, Section 6.2 (Ref. 2). The system is sized to ensure adequate heat removal, based on highest expected temperatures of cooling water, maximum loadings, and leakage allowances. The SWS is designed to perform its function with a single failure of any active component, assuming the loss of offsite power.

The SWS, in conjunction with the CCW System, also cools the unit from residual heat removal (RHR), as discussed in the UFSAR, Section 5.4.4, (Ref. 3) entry conditions to MODE 5 during normal and post accident operations. The time required for this evolution is a function of the number of CCW and RHR System trains that are operating and SW supply temperature. One SWS train is sufficient to remove decay heat during subsequent operations in MODES 5 and 6. This assumes a maximum SWS temperature of 97°F occurring simultaneously with maximum heat loads on the system.

The SWS satisfies Criterion 3 of the NRC Policy Statement.

LCO

Two SWS trains are required to be OPERABLE to provide the required redundancy to ensure that the system functions to remove post accident heat loads, assuming that the worst case single active failure occurs coincident with the loss of offsite power.

An SWS train is considered OPERABLE during MODES 1, 2, 3, and 4 when:

- a. Two SWS pumps are OPERABLE;
- b. One SWS booster pump is OPERABLE;

(continued)

BASES

APPLICABLE
SAFETY ANALYSES

The UHS is the sink for heat removed from the reactor core following all accidents and anticipated operational occurrences in which the unit is cooled down and placed on residual heat removal (RHR) operation. Since the UHS is the normal heat sink for condenser cooling via the Circulating Water System, unit operation at full power is its maximum heat load. Its maximum post accident heat load occurs at the time that recirculation begins after a design basis loss of coolant accident (LOCA). Near this time, the unit switches from injection to recirculation and the containment cooling systems and RHR are required to remove the core decay heat.

The operating limits are based on conservative heat transfer analyses for the worst case LOCA and maintaining adequate net positive suction head (NPSH) for the SWS pumps. The UHS at the minimum allowable level of 218 ft MSL provides a 22 day supply of cooling water to the SWS pumps under worst case local meteorological conditions. After 22 days, the minimum NPSH for the SWS pumps is reached when the lake level drops to 210.64 ft MSL. The lake surface area at 210.64 ft MSL is capable of providing decay heat cooling for the plant without exceeding the 97°F maximum SWS temperature requirement. Therefore, the necessary lake level for adequate NPSH for the SWS pumps is more limiting than the lake surface area necessary for decay heat removal. The 22 day supply of water is based on the lake volume and surface area values provided in References 2 and 3, an evaporation rate of 35 ft³/sec (Ref. 4) that assumes both Unit 1 (fossil Plant) and Unit 2 operating at 100% power for 6 hours, an evaporation rate of 17 ft³/sec that assumes Unit 1 in operation and Unit 2 shut down for the remaining 22 day period under maximum evaporation conditions, a head flow of 16 ft³/sec which is based upon the minimum head flow measured at the Black Creek inlet over the past 30 years (Ref. 5), and a fully open Howell Bunger valve which provides an average flow of 260 ft³/sec. No credit is taken for natural springs, precipitation or other drainage input into the lake for the 22 day period. The opening and testing of the tainter gates is administratively limited to approximately 2.5 inches except for flood control measures necessary to protect the integrity of the dam which approximates the capacity of one Howell Bunger valve. A failure of a tainter gate to reclose when the gate is raised 2.5 inches or less is bounded by a fully open Howell Bunger valve in the analysis.

(continued)

BASES

APPLICABLE
SAFETY ANALYSES
(continued)

The UHS satisfies Criterion 3 of the NRC Policy Statement.

LCO

The UHS is required to be OPERABLE and is considered OPERABLE if it contains a sufficient volume of water at or below the maximum temperature that would allow the SWS to operate for at least 22 days following the design basis LOCA without the loss of NPSH, and without exceeding the maximum design temperature of the equipment served by the SWS. To meet this condition, the UHS temperature should not exceed 97°F and the level should not fall below 218 ft MSL during normal unit operation.

APPLICABILITY

In MODES 1, 2, 3, and 4, the UHS is required to support the OPERABILITY of the equipment serviced by the UHS and required to be OPERABLE in these MODES.

In MODE 5 or 6, the OPERABILITY requirements of the UHS are determined by the systems it supports.

ACTIONS

A.1, and A.2

If the UHS is inoperable, the unit must be placed in a MODE in which the LCO does not apply. To achieve this status, the unit must be placed in at least MODE 3 within 6 hours and in MODE 5 within 36 hours.

The allowed Completion Times are reasonable, based on operating experience, to reach the required unit conditions from full power conditions in an orderly manner and without challenging unit systems.

BASES (continued)

SURVEILLANCE
REQUIREMENTS

SR 3.7.8.1

This SR verifies that adequate long term (22 day) cooling can be maintained. The specified level also ensures that sufficient NPSH is available to operate the SWS pumps. The 24 hour Frequency is based on operating experience related to trending of the parameter variations during the applicable MODES. This SR verifies that the UHS water level is ≥ 218 ft MSL.

SR 3.7.8.2

This SR verifies that the SWS is available to cool the CCW System to at least its maximum design temperature with the maximum accident or normal design heat loads for 30 days following a Design Basis Accident. The 24 hour Frequency is based on operating experience related to trending of the parameter variations during the applicable MODES. This SR verifies that the service water temperature is $\leq 97^{\circ}\text{F}$.

REFERENCES

1. UFSAR, Section 9.2.4.
2. UFSAR Section 2.4.6.1.
3. UFSAR Section 2.1.1.2.
4. NUREG-75/024, "Final Environmental Statement Related to the Operation of H. B. Robinson Nuclear Steam-Electric Plant Unit 2," U. S. Nuclear Regulatory Commission, Washington DC 20555, April 1975, page 3-7.
5. USGS Historical Daily Values for Station Number 02130900, Black Creek Near McBee, South Carolina, Years 1960-1993.