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Quad Cities Generating Station
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June 16, 2000

SVP-00-106

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Quad Cities Nuclear Power Station, Units 1 and 2
Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: Failure of Control Room Emergency Ventilation Refrigeration Control Unit

Enclosed is Licensee Event Report (LER) 254/00-004, Revision 00, for Quad Cities Nuclear Power Station.

This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(v)(D). The licensee shall report any event or condition that alone could have prevented the fulfillment of the safety function of structures or systems that are needed to mitigate the consequences of an accident.

We are committing to the following actions:

The root cause of the Refrigeration Control Unit failure has not been determined. A supplemental report will be issued upon completion of the root cause determination. Any additional corrective actions will be included in the supplemental report.

Any other actions described in the submittal represent intended or planned actions by Commonwealth Edison (ComEd) Company. They are described for the NRC's information and are not regulatory commitments.

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Should you have any questions concerning this letter, please contact Mr. C.C. Peterson at (309) 654-2241, extension 3609.

Respectfully,

A handwritten signature in cursive script, appearing to read "Joel P. Dimmette, Jr.", written in dark ink.

Joel P. Dimmette, Jr.
Site Vice President
Quad Cities Nuclear Power Station

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Quad Cities Nuclear Power Station

LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the information and Records Management Branch (1-6 133), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office Of Management And Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1)
Quad Cities Nuclear Power Station, Unit 1

DOCKET NUMBER (2)
05000254

PAGE (3)
1 of 3

TITLE (4)
Trip of Safety-Related Control Room Emergency Ventilation System Refrigeration Control Unit

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MON TH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	19	2000	2000	004	00	06	16	2000	Quad Cities Nuclear Power Station, Unit 2	05000265
									FACILITY NAME N/A	DOCKET NUMBER N/A

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more) (11)							
1	100	20.2201(b)	20.2203(a)(2)(v)	50.73(a)(2)(i)	50.73(a)(2)(viii)				
		20.2203(a)(i)	20.2203(a)(3)(i)	50.73(a)(2)(ii)	50.73(a)(2)(x)				
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)	50.73(a)(2)(iii)	73.71				
		20.2203(a)(2)(ii)	20.2203(a)(4)	50.73(a)(2)(iv)	OTHER				
		20.2203(a)(2)(iii)	50.36(c)(1)	X 50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A				
		20.2203(a)(2)(iv)	50.36(c)(2)	50.73(a)(2)(vii)					

LICENSEE CONTACT FOR THIS LER (12)

NAME: Charles Peterson, Regulatory Assurance Manager
TELEPHONE NUMBER (Include Area Code): (309) 654-2241 ext 3609

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/>	YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/>	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
					07	31	2000

ABSTRACT (Limit to 1400 spaces, i. e., approximately 15 single-spaced typewritten lines) (16)

On May 19, 2000, during performance of QCOS 5750-02, "Control Room Emergency Filtration Monthly Test," the safety related control room emergency ventilation (CREV) refrigeration control unit (RCU) tripped due to a loose local control switch cam follower retaining screw. The CREV RCU was declared inoperable and at 1334 hours a 4-hour Emergency Notification System phone call was made in accordance with 10CFR50.72(b)(2)(iii)(D).

On May 23, 2000, while the switch was being verified to operate correctly, it was discovered that the switch was incorrectly assembled. The switch was reassembled correctly and tested.

The root cause determination is not complete for this event. A supplemental report will be issued providing the root cause and corrective actions for this event.

LICENSEE EVENT REPORT (LER)

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Quad Cities Nuclear Power Station, Unit 1	05000254	2000	004	00	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power

Energy Industry Identification System (EIIIS) Codes are identified in the text as [XX] and are obtained from IEEE Standard 805-1984, IEEE Recommended Practice for System Identification in Nuclear Power Plants and Related Facilities.

EVENT IDENTIFICATION:

Trip of Safety-Related Control Room Emergency Ventilation System Refrigeration Control Unit

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 1	Event Date: May 19, 2000	Event Time: 1045 hours
Reactor Mode: 1	Mode Name: Power Operation	Power Level: 100

Power Operation (1) - Mode switch in the RUN position with average reactor coolant temperature at any temperature.

B. DESCRIPTION OF EVENT:

On May 19, 2000, during performance of QCOS 5750-02, "Control Room Emergency Filtration Monthly Test," the safety related control room emergency ventilation (CREV) [VI] refrigeration control unit (RCU) tripped on low suction pressure. Subsequent investigation revealed that the RCU local control switch [HS] cam follower retaining screw was loose. This rendered the CREV RCU inoperable. A 4-hour Emergency Notification System phone call was made in accordance with 10CFR50.72(b)(2)(iii)(D) at 1334 hours.

On May 23, 2000, an operability determination was completed with interim corrective actions. The operability determination included a compensatory action to verify weekly that the switch operated correctly. While this action was initially being implemented, it was discovered that the switch was incorrectly assembled. The cam follower was not aligned in a manner that would allow the retaining screw to have full contact with the shaft. Prior to this event, the last work performed on the switch was on December 16, 1999. The switch was reassembled correctly and tested.

C. CAUSE OF EVENT:

The root cause of the RCU failure has not been determined. A supplemental report will be issued upon completion of the root cause determination. Any additional corrective actions will be included in the supplemental report.

D. SAFETY ANALYSIS

The safety significance of this event was minimal. The CREV system filtration capability was not lost, and control room temperature was maintained during this event. In addition, the non-safety related train of control room ventilation was operable throughout this event.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

E. CORRECTIVE ACTIONS:

Corrective Action Completed:

The switch was reassembled correctly and tested on May 23, 2000. On June 7, 2000, the switch was replaced.

Corrective Actions to be Completed:

The root cause of the RCU failure has not been determined. A supplemental report will be issued upon completion of the root cause determination. Any additional corrective actions will be included in the supplemental report.

F. PREVIOUS OCCURRENCES:

Previous occurrences for this event will be provided in a supplemental report.

G. COMPONENT FAILURE DATA:

The component failure data for this event will be provided in a supplemental report.