

June 23, 2000

Mr. Ted C. Feigenbaum
Executive Vice President and
Chief Nuclear Officer
North Atlantic Energy Service Corporation
c/o Mr. James M. Peschel
P.O. Box 300
Seabrook, NH 03874

SUBJECT: SEABROOK STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT RE:
RELOCATION OF CERTAIN REFUELING OPERATIONS TECHNICAL
SPECIFICATIONS (TAC NO. MA8678)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 72 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No 1, in response to your application dated April 14, 2000.

The amendment would revise the Technical Specifications by relocating Sections 3/4.9.5, "Communications"; 3/4.9.6, "Refueling Machine"; and 3/4.9.7, "Crane Travel - Spent Fuel Storage Areas" to the Seabrook Station Technical Requirement Manual. The associated Bases and index pages will be modified to address the proposed change.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Robert M. Pulsifer, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosures: 1. Amendment No. 72 to NPF-86
2. Safety Evaluation

cc w/encls: See next page

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NORTH ATLANTIC ENERGY SERVICE CORPORATION, ET AL.*

DOCKET NO. 50-443

SEABROOK STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72
License No. NPF-86

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the North Atlantic Energy Service Corporation, et al. (the licensee), dated April 14, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*North Atlantic Energy Service Corporation (NAESCO) is authorized to act as agent for the: North Atlantic Energy Corporation, Canal Electric Company, The Connecticut Light and Power Company, Great Bay Power Corporation, Hudson Light & Power Department, Massachusetts Municipal Wholesale Electric Company, Little Bay Power Corporation, New England Power Company, New Hampshire Electric Cooperative, Inc., Taunton Municipal Light Plant, The United Illuminating Company, and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-86 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 72, and the Environmental Protection Plan contained in Appendix B are incorporated into Facility License No. NPF-86. NAESCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance (including the relocation of the Technical Specifications which are the subject of this license amendment to the Seabrook Station Technical Requirements Manual as described in the application dated April 14, 2000 and evaluated in the staff's safety evaluation dated June 23, 2000 for use by licensee personnel).

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: June 23, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 72

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove

ix
3/4 9-5
3/4 9-6
3/4 9-7
B3/4 9-3

Insert

ix
3/4 9-5
3/4 9-6
3/4 9-7
B3/4 9-3

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. NPF-86

NORTH ATLANTIC ENERGY SERVICE CORPORATION

SEABROOK STATION, UNIT NO. 1

DOCKET NO. 50-443

1.0 INTRODUCTION

By letter dated April 14, 2000, the North Atlantic Energy Service Corporation (the licensee, NAESCO) submitted a request for changes to the Seabrook Station Technical Specifications (TSs). NAESCO proposed modifying Sections 3/4.9.5, "Communications"; 3/4.9.6, "Refueling Machine"; and 3/4.9.7, "Crane Travel - Spent Fuel Storage Areas" by removing requirements that are adequately controlled by existing regulations and relocating details which are not otherwise needed to satisfy Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36. Guidance on the proposed changes was developed by the Nuclear Regulatory Commission (NRC) and provided in the staff's review of WCAP-11618 as documented in a letter dated May 9, 1988, to R. A. Newton, Chairman of the Westinghouse Owners Group. The requested changes would specifically relocate TS 3/4.9.5, 3/4.9.6, and 3/4.9.7 to the Seabrook Station Technical Requirement Manual (SSTR) which is referenced in the Seabrook Station Updated Final Safety Analysis Report (UFSAR). The SSTR is the implementing manual for the Seabrook Station Technical Specification Improvement Program which is controlled under TS Section 6.0, "Administrative Controls", paragraph 6.7.1.i. The associated Bases and index pages will be modified to address the proposed change.

2.0 BACKGROUND

Section 182.a of the Atomic Energy Act of 1954, as amended, (the Act) requires applicants for nuclear power plant operating licenses to state TSs to be included as part of the license. The NRC's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The NRC has provided guidance for the contents of TSs in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), (58 FR 39132) dated July 22, 1993, in which the Commission indicated that compliance with the Final Policy Statement satisfies §182.a of the Act. In particular, the Commission indicated that certain items could be relocated from the TSs to licensee-controlled documents.

Title 10 of the Code of Federal Regulations, Section 50.36, identifies four criteria to be used in determining whether particular safety functions are required to be included in the TSs, as follows: (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient analysis that either assumes the failure of, or presents a challenge to, the integrity of a fission product boundary; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of, or presents a challenge to, the integrity of a fission product boundary; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. As a result, TS requirements which fall within or satisfy any of the criteria in the regulations must be retained in the TSs, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

Existing TS LCOs which fall within or satisfy any of the above criteria must be retained in the TSs; those which do not fall within these criteria may be relocated to other licensee-controlled documents such that future changes could be made to these provisions pursuant to 10 CFR 50.59. In the NRC's May 9, 1988, letter referenced above, Table 2, "Westinghouse Standard Technical Specifications LCOs Which May Be Relocated," specifically references the LCOs which are the subject of the licensee's amendment request as candidates which may be relocated to a licensee controlled document.

NAESCO stated that relocating the specific requirements of TS Sections 3/4.9-5, 3/4.9-6, and 3/4.9-7 from the TS to the SSTR allows these requirements to be controlled by the 10 CFR 50.59 change process rather than the TS amendment process. The changes can reduce the regulatory burden on NAESCO and the NRC for those changes that do not satisfy the criteria of 10 CFR 50.36 for inclusion in the TSs.

3.0 EVALUATION

The following discussion sets forth details of the NRC staff's conclusions regarding the removal or relocation of selected LCOs from the TS for the Seabrook Station. The changes were reviewed in accordance with the guidance provided in, or planned for, the applicable Standard Technical Specifications (STS), NUREG-1431, Revision 1. In addition, these changes were reviewed in accordance with the guidance provided in the staff's review of WCAP-11618 as documented in a letter dated May 9, 1988, to R. A. Newton, Chairman of the Westinghouse Owners Group.

3.1 TS 3/4.9.5 Communications

TS 3/4.9.5 LCO requires maintaining direct communications between the control room and personnel at the refueling station during core alterations. The basis for this requirement is to ensure that refueling station personnel can be promptly informed of significant changes in the facility status or core reactivity conditions during fuel core alterations. This LCO was identified as a candidate for relocation in the Westinghouse Standard Technical Specifications (WSTS) and the staff's review of this LCO indicates that it does not meet the criteria requiring

establishment of an LCO. Relocating this TS to the SSTR will permit the licensee to control future changes pursuant to 10 CFR 50.59. Based on the review, the staff finds relocation of TS 3/4.9.5 to the licensee's SSTR to be acceptable.

3.2 TS 3/4.9.6 Refueling Machine

TS 3/4.9.6 defines the minimum capacity and overload cutoff limits for the refueling machine and for the auxiliary hoist, to be considered operable during the movement of drive rods or fuel assemblies. The TS Bases state that the operability requirements ensure that the refueling machine will be used for movement of drive rods and fuel assemblies and that each hoist has sufficient capacity to lift a drive rod or fuel assembly. It also ensures that the core internals and reactor vessel are protected from excessive lifting force in the event they are inadvertently engaged during lifting operations.

The relocation of the LCO requirements related to the refueling machine and auxiliary hoists' operability is acceptable as it is not an LCO that satisfies the criteria of 10 CFR 50.36. Further, there is no accident analysis based on the minimum capacity and overload cutoff limits of the refueling machine or auxiliary hoist. Based on the above, the staff's review finds relocation of TS 3/4.9.6 to the licensee's SSTR to be acceptable.

3.3 TS 3/4.9.7 Crane Travel - Spent Fuel Storage Areas

TS 3/4.9.7 LCO prohibits loads in excess of 2100 pounds from travel over fuel assemblies in the spent fuel pool. This restriction ensures that no more than the contents of one fuel assembly will rupture in the event of a fuel handling accident. The bounding accident analyses demonstrate acceptable radiological consequences even if a fuel assembly were dropped into the spent fuel pool, based on the capabilities of the Fuel Storage Building Emergency Air Cleaning System. Consequently, the travel and loading restrictions of the crane are not necessary to prevent or mitigate fuel handling accidents and therefore, do not satisfy the TS LCO criteria of 10 CFR 50.36. Based on the above, the staff finds relocation of TS 3/4.9.7 to the licensee's SSTR to be acceptable.

3.4 Index Page ix

With the acceptability of the relocation of TS Sections 3/4.9.5, 3/4.9.6, and 3/4.9.7 as stated above the deletion of 1) "Communications" from 3/4.9.5, 2) "Refueling Machine" from 3/4.9.6, and 3) "Crane Travel - Spent Fuel Storage Areas and the insertion of "THIS SPECIFICATION NUMBER IS NOT USED" in these TS sections is acceptable.

3.5 Bases Sections 3/4.9.5, 3/4.9.6, and 3/4.9.7

With the acceptability of the relocation of TS Sections 3/4.9.5, 3/4.9.6, and 3/4.9.7 as stated above the staff has no objection to the deletion of the text of these sections and replacing the section title with "THIS SPECIFICATION NUMBER IS NOT USED."

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Hampshire and Massachusetts State officials were notified of the proposed issuance of the amendment. The State officials had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 31358). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Pulsifer

Date: June 23, 2000

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