

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 22, 2000

LICENSEE:

Public Service Electric and Gas Company

FACILITY:

Salem Nuclear Generating Station, Unit Nos. 1 and 2

SUBJECT:

SUMMARY OF MEETING BETWEEN THE NUCLEAR REGULATORY

COMMISSION (NRC) STAFF AND PUBLIC SERVICE ELECTRIC AND GAS

COMPANY (PSE&G) ON MAY 25, 2000

On Wednesday, May 25, 2000, at 9:30 a.m. a meeting was held between members of the NRC staff and representatives of PSE&G, the licensee for the Salem Nuclear Generating Station, Unit Nos. 1 and 2 (Salem). The purpose of the meeting was to discuss issues related to Salem's license amendment application dated November 24, 1999, and was a follow-up to the public meeting held on May 10, 2000. The licensee had requested changes to its technical specifications (TSs) in response to Generic Letter (GL) 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal." The meeting was open to interested members of the public, petitioners, intervenors, or other parties to attend as observers pursuant to the "Commission Policy Statement on Staff Meetings Open to the Public" (see 59 FR 48340, published September 20, 1994). The discussions lasted approximately 4 hours. A list of attendees is provided as Enclosure 1 to the meeting summary.

The Salem licensee opened the meeting with a short presentation. Copies of handouts for the discussion are provided as Enclosure 2. Following the meeting on May 10, 2000, the licensee determined that its originally proposed TS Surveillance Requirement (SR), specifying a 90% efficiency for the 1" filter serving the Auxiliary Building Ventilation (ABV) system, should be changed. PSE&G was concerned that it would be difficult to consistently meet this value, based upon the results of recent tests. As a result, the licensee is requesting additional changes to its original November 24, 1999, proposal. PSE&G stated that the changes were needed in order to comply with the GL 99-02 guidance allowing a minimum 2:1 factor of safety for filter efficiency for the ABV system and the Control Room Emergency Air Conditioning System (CREACS), and to keep post-accident control room doses (thyroid) below Salem's current licensing basis. A summary of the revised changes is provided in the table below. The most recent changes are highlighted in bold.

1" Auxiliary Building Filter	Analysis	TS SR Testing
- Current Licensing Basis	90%	90%
- Revised Proposed TS Change	70%	85%
2" CREACS Filter	Analysis	TS SR Testing
- Current Licensing Basis	95%	99%
- Revised Proposed TS Change	95%	97.5%

The discussion that followed the PSE&G presentation focused on the proposed changes involving the 1" ABV filter. Salem's current licensing basis dose calculations credits a filter efficiency of 90%, and the licensee wishes to apply an efficiency of 70%. PSE&G stated that in 1998, the engineering calculation apportioning the amount of engineered safety feature (ESF) leakage to the ABV charcoal filter, was revised. According to the revised calculation, approximately 77% of the ESF leakage is processed through the charcoal filter bed, and the remaining 23% is not filtered. As a result, PSE&G believes that the current licensing basis dose calculation, which apportions 50% of the ESF leakage to the charcoal filter, was overly conservative. Therefore, the licensee revised its dose calculation input assumptions as follows:

Input Assumption/Parameter	Previous Calculation	Revised Calculation
Credited efficiency 1-inch ABV charcoal filter	90%	70%
Amount of ESF leakage passing through ABV charcoal filter	50%	65%
Equivalent overall iodine removal efficiency	45%	45% (Actual = 45.5%)

Based upon the revised input parameters, PSE&G stated that the resulting calculated thyroid dose to control room workers would remain to be 28.4 rem.

Since PSE&G had made these changes, without prior NRC approval, pursuant to Title 10 of the *Code of Federal Regulations*, Section 50.59 (10 CFR 50.59), the licensee provided a brief presentation on how it arrived at the conclusion that the change did not constitute an unreviewed safety question according to the rule. The NRC staff acknowledged that the licensee is entitled to make changes to its facility under 10 CFR 50.59, and that any change to the facility could be subject to audit in the future.

The meeting concluded at approximately 2:00 p.m., with the understanding that the licensee would be submitting a revised proposed license amendment request as early as June 2, 2000.

Robert J. Fretz, Project Manager, Section 2

Project Directorate I

Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures: 1. List of Meeting Attendees

2. PSE&G Slide Presentation

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/RA/

Robert J. Fretz, Project Manager, Section 2 Project Directorate I Division of Licensing Project Management Office of Nuclear Reactor Regulation

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cc w/encls: See next page

Distribution: See next page

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DATE	06/06/00	6/9/00	6/14/0	

Official Record Copy

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cc:

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PUBLIC SERVICE ELECTRIC AND GAS

MEETING, MAY 10, 2000

LIST OF ATTENDEES

NRC

James W. Clifford	NRR/DLPM
Robert Fretz	NRR/DLPM
Michelle Hart	NRR/DSSA
Jack Hayes	NRR/DSSA
Eileen McKenna	NRR/DRIP
J. H. Raval	NRR/DSSA
John Segala	NRR/DSSA
Harold Walker	NRR/DSSA
Eric Weiss	NRR/DSSA

PSE&G

Gabe Salamon Steve Mannon Fran Sullivan Brian Thomas Ray Runowski John Duffy Kenneth Hutko	Manager, Licensing Manager, System Engineering Director, Engineering Licensing Engineering Engineering Engineering Engineering
Douglas Groves	Engineering

Public Participants

Christine Cave McGraw-Hill



Salem Generic Letter 99-02 Implementation

May 25, 2000



- Introduction
- Feedback From May 10 Meeting
- Revised Technical Approach
- Implementation

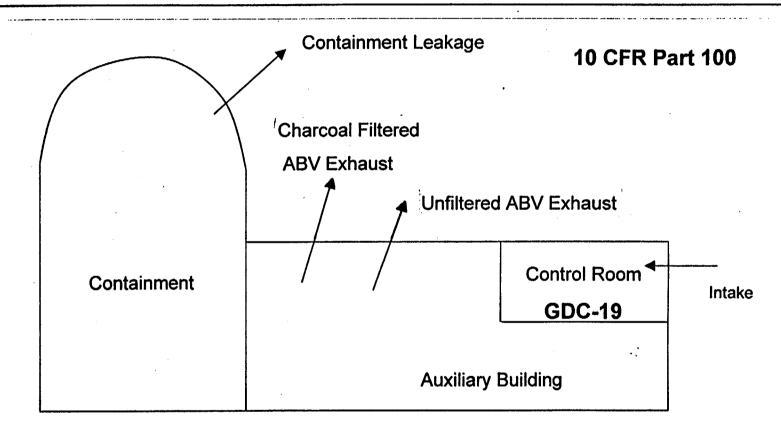


Where We Have Been Since May 10

- Further Testing of 1" Charcoal Filter
- Reviewed Input Assumptions
- Conservative Input Assumptions for Airborne ESF Leakage



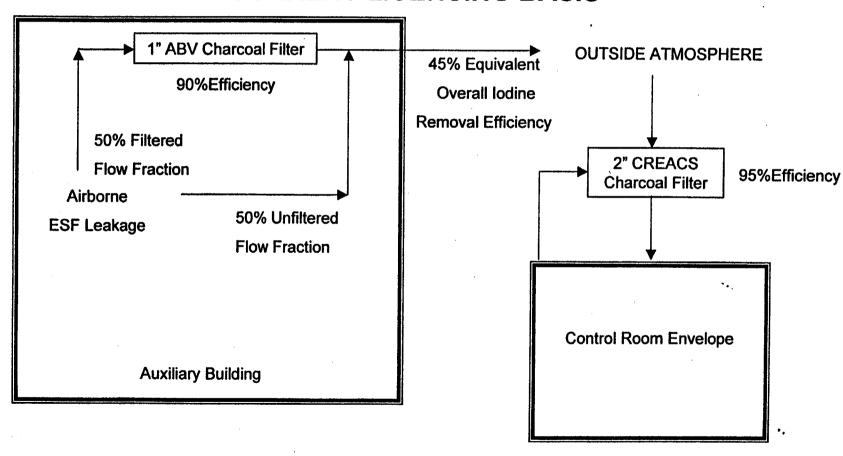
Salem GL 99-02 Implementation





Salem GL 99-02 Implementation

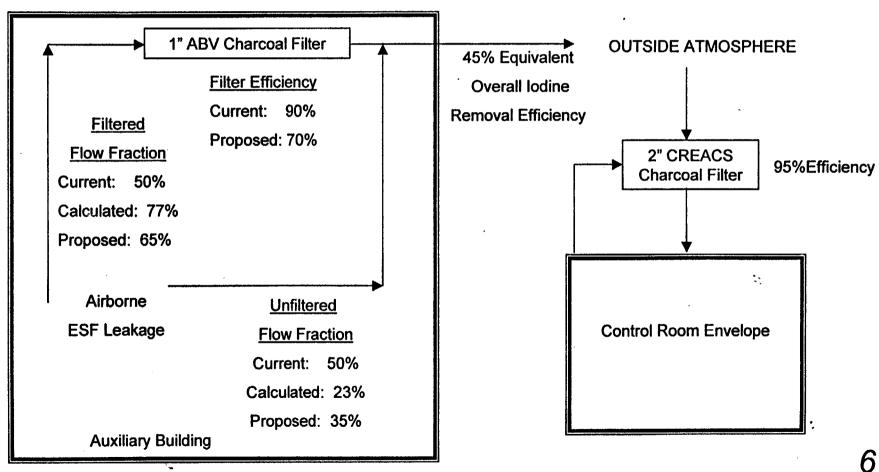
CURRENT LICENSING BASIS





Salem GL 99-02 Implementation

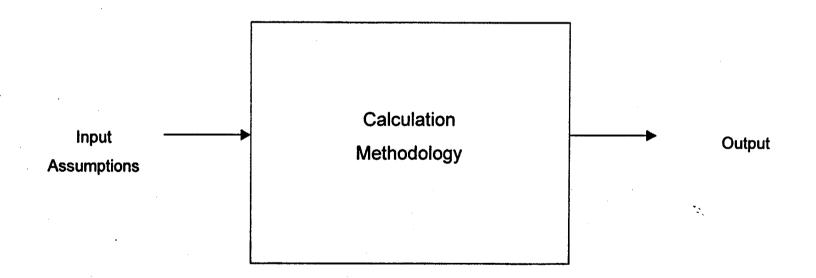
PROPOSED CHANGE





Salem GL 99-02 Implementation

TYPICAL ANALYSIS BREAKDOWN





Salem GL 99-02 Implementation

INPUT ASSUMPTIONS

Current Licensing Basis Proposed Unfiltered Unfiltered 50% ESF Leakage 35% ESF Leakage 0% Filter Efficiency **0% Filter Efficiency** 100% ESF Leakage **45% Filter Efficiency Filtered Filtered** 50% ESF Leakage 65% ESF Leakage 90% Filter Efficiency 70% Filter Efficiency





10CFR50.59 Review

■ Does the change create the possibility of a new or different type of accident from those evaluated in the SAR



10CFR50.59 Review

■ Does the change reduce the margin of safety as defined in the basis of any Technical Specification



10CFR50.59 Review

■ Does the change increase the probability of occurrence or the consequences of accident or malfunction of equipment important to safety previously evaluated in the SAR





Summary

- LOCA Input Parameter Change Does Not Result in an Unreviewed Safety Question
- Technical Specification Changes are being Submitted to Meet GL 99-02
- Current Testing Will Be Performed IAW ASTM D3803-1989 Using the Enforcement Discretion of GL 99-02

HARD COPY

PUBLIC PD I-2 Reading RFretz OGC ACRS

cc: Licensee & Service List (with all enclosures)

E-MAIL

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TClark

MHart

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