June 22, 2000

Mr. James Knubel
Chief Nuclear Officer
Power Authority of the State of
New York
123 Main Street
White Plains, NY 10601

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT - PHONE CALL OF MAY 25, 2000

Dear Mr. Knubel:

The May 25, 2000, conference call between the NRC staff and your staff resulted in clarification of the enclosed questions that were e-mailed to your staff on May 23, 2000. At the conclusion of the call, your staff agreed to provide a partial response to our questions that could be answered with little difficulty within 2 weeks and a complete response to the remaining questions within 60 days. The enclosure provides the questions.

Sincerely,

/RA/

Guy S. Vissing, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure: As stated

cc w/encl: See next page

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JAMES A. FITZPATRICK NUCLEAR POWER PLANT (JAF)

(1) NEDC-24361P, “JAF suppression pool temperature”.

(2) Power Authority of the State of New York (PASNY) calculations performed in response to NEDC-24361P.

(3) Was an evaluation of the suppression pool temperature done during power uprate and increase of the lake water temperature? If an evaluation was performed, provide the evaluation.

(4) NYPA calculations used for determining Heat Capacity Temperature Limit (HCTL) used in Emergency Operating Procedures (EOPs).

(5) EOPs used for Suppression Pool Temperature control (Primary Containment Control). Also submit the plant operating procedures for the operation of Residual Heat Removal (RHR) shutdown cooling system (SDC).

(6) PASNY response and follow-up actions to service Information letter (SIL)-615. General Electric (GE) suggested that owners of GE Boiling Water Reactors (BWRs) evaluate the following if the Automatic Depression System (ADS) is not single failure proof:

(a) Demonstrate that the RHR system has the capacity to maintain the maximum suppression pool temperature below 170 degrees F during events for which High Pressure Coolant Injection (HPCI) is relied upon for long-term operation and reactor depressurization, and that the HPCI pump has adequate Net Positive Suction Head (NPSH) at the maximum calculated temperature, or

(b) Implement modifications to assure that the electrical power, controls and cabling for at least one ADS valve are physically separated from the electric power, controls and cabling for the remainder of the ADS valves.

Submit only the information developed in response to the above items in the SIL. **(We are not requesting any modifications or any new evaluation)**

(7) HPCI flow diagram, Piping and Instrumentations Drawings (P&ID) and turbine exhaust sparger drawing.

(8) RHR flow diagram (P& ID).

(9) Did PASNY perform an analysis or calculation to verify that the alternate shutdown cooling method using Safety Relief Valves (SRVs) and Suppression pool cooling is feasible in JAF?

(10) The following issues relate to those brought before the BWR Owners Group meeting on March 8-9, 2000. JAF had previously raised them with GE. GE responded that they needed to be reviewed by each utility as plant-specific issues:

(a) At JAF loss of a single division of suppression cooling may also disable shutdown cooling (SDC). What remedial steps would be taken by operating staff in accordance with the EOPs if this failure were to occur during accident mitigation?

Are the power supply to the inboard and outboard isolation valves in the RHR common shutdown cooling suction line single failure proof? Are they supplied only from one electric division or both? Can a single failure disable both shutdown cooling systems? Can a loss of one DC power supply result in loss of SDC and one half of RHR in JAF?

(b) At JAF drywell pressure must be decreased below 2.7 psi before shutdown cooling can be brought into service. Following potential accident conditions which cause drywell pressurization, what provisions are available at JAF to reduce and maintain drywell pressure below 2.7 psi so that SDC can be initiated?

(c) At JAF the SRVs can depressurize the reactor to about 65 psia. The SDC isolation set point is 90 psia. Following a postulated accident can SDC at JAF be brought into service before it isolates on high pressure?

(d) The SDC inboard isolation valve may not be Environmentally Qualified (EQ'd) to open after many hours of exposure to accident conditions. How does the environmental qualification of the SDC inboard isolation valve at JAF compare to expected conditions following postulated accidents? Provide the information to review the environmental qualification of the RHR shutdown cooling valve 10MOV-18. Describe how the valve is qualified to operate as assumed in NEDC-24361P.

(11) Provide a sketch or a drawing to show the relative positions of thermocouples used for suppression pool temperature monitoring. Describe the suppression pool temperature monitoring system and the operator actions during postulated accidents that would be initiated based on indicated suppression pool temperature. How is the suppression pool temperature (bulk or average or local) indicated in the control room calculated?