



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 20, 2000

Mr. Merrill Atkins
Regulatory Affairs Manager
Yankee Atomic Electric Company
Midstate Office Park Suite 200/210
Auburn, MA 01501

SUBJECT: YANKEE NUCLEAR POWER STATION - ISSUANCE OF AMENDMENT TO
REVISE TECHNICAL SPECIFICATIONS RELATED TO THE QUALITY
ASSURANCE PROGRAM (TAC NO. MA5032)

Dear Mr. Atkins:

The Commission has issued the enclosed Amendment No. 154 to the Yankee Atomic Electric Company (YAEC) Possession Only License (POL) No. DPR-3 for the Yankee Nuclear Power Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated March 17, 1999, as supplemented April 23, July 21, November 2, 1999, and March 6, 2000.

The amendment revises TS Section 6.0, Administrative Controls, by consolidating management positions and modifying review and audit functions.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

Phillip M. Ray, Project Manager
Decommissioning Section
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-29

Enclosures: 1. Amendment No. 154 to DPR-3
2. Safety Evaluation

cc w/encls: See next page

DF01

Yankee Nuclear Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

YANKEE ATOMIC ELECTRIC COMPANY

DOCKET NO. 50-29

YANKEE NUCLEAR POWER STATION

AMENDMENT TO POSSESSION ONLY LICENSE

Amendment No. 154
License No. DPR-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Yankee Atomic Electric Company (the licensee) dated March 17, 1999, as supplemented April 23, July 21, November 2, 1999, and March 6, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will be maintained in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Possession Only License No. DPR-3 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 154, are hereby incorporated in the license. The licensee shall possess and maintain the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "R. F. Dudley, Jr.", is written over the typed name of Michael T. Masnik.

Michael T. Masnik, Chief
Decommissioning Section
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: June 20, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 154

POSSESSION ONLY LICENSE NO. DPR-3

DOCKET NO. 50-29

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

6-1
6-5 through 6-12
6-13
6-14
6-19
6-21
6-22

INSERT

6-1
6-5 through 6-10
6-13
6-14
6-19
6-21
6-22

6.0 ADMINISTRATIVE CONTROLS

Administrative controls are the written rules, orders, instructions, procedures, policies, practices, and the designation of authorities and responsibilities by the management to obtain assurance of safety and quality of maintenance of a nuclear facility. These controls shall be adhered to.

6.1 RESPONSIBILITY

- 6.1.1 The Decommissioning Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.
- 6.1.2 In all matters relating to the operation of the plant and to these Technical Specifications, the Decommissioning Manager shall report to and be directly responsible to the Vice President of Yankee Atomic Electric Company.

6.2 ORGANIZATION

- 6.2.1 An on-site and an off-site organization shall be established for plant operation and corporate management. The on-site and off-site organization shall include the positions for activities affecting the safety of the facility.
 - a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organizational charts. These organizational charts shall be documented in the FSAR.
 - b. A single corporate officer shall have overall responsibility for plant nuclear safety. This individual shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining and providing technical support in the plant so that continued nuclear safety is assured.
 - c. There shall be an individual management position in the on-site organization having overall responsibility for safe operation of the plant; he/she shall have control over those on-site resources necessary for safe operation and maintenance of the plant.

6.0 ADMINISTRATIVE CONTROLS (Continued)

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility management/supervisory staff shall meet or exceed the minimum qualifications of ANSI 18.1-1971 for comparable positions, except for the Radiation Protection Manager who shall also meet the minimum qualifications of Regulatory Guide 1.8, Revision 1.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility Certified Fuel Handlers shall be conducted in accordance with an NRC approved training program. A training program for the unit staff shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971.

6.5 REVIEW AND AUDIT

6.5.1 Independent Safety Review

An Independent Safety Review shall be a thorough review conducted by one or more qualified Independent Safety Reviewers. Persons performing these reviews shall be knowledgeable in the subject area being reviewed. Independent Safety Reviews must be completed prior to implementation of proposed activities.

- a. Independent Safety Reviewers shall be individuals without direct responsibility for the performance of the activities under review; these reviewers may be from the same functionally cognizant organization as the individual or group performing the original work.
- b. Independent Safety Reviewers shall have at least 5 years of professional experience and either a Bachelor's Degree in Engineering or the Physical Sciences or shall have equivalent qualifications in accordance with ANSI 18.1-1971. The Decommissioning Manager (or a designee) shall document the appointment of Independent Safety Reviewers.
- c. The following subjects shall be independently reviewed by a qualified Independent Safety Reviewer:
 1. safety evaluations for changes in the facility as described in the Final Safety Analysis Report (FSAR), changes in procedures as described in the FSAR, and tests or experiments not described in the FSAR to verify that such actions do not involve a change to the Technical

6.0 ADMINISTRATIVE CONTROLS (Continued)

Specifications or will not involve an unreviewed safety question as defined in 10CFR50.59;

2. proposed changes to the programs required by Technical Specification 6.7. to verify that such changes do not involve a change to the Technical Specifications and will not involve an unreviewed safety question as defined in 10CFR50.59; and
3. proposed changes to the Technical Specification Bases.

6.5.2 Independent Review and Audit Committee (IRAC)

The IRAC is responsible for reviewing, auditing, and advising the President of Yankee Atomic Electric Company (or a designee) on matters related to the safe storage of irradiated fuel. This review and audit function is independent of line organization responsibilities.

- a. The IRAC shall include a minimum of five members. Alternates may be substituted for regular members. The licensee shall designate in writing the chairman, the members, and alternates for the IRAC. The chairman shall not have management responsibilities for, or report to, the line organizations responsible for operation or maintenance of the fuel storage facility.
- b. The IRAC shall collectively have experience and knowledge in the following functional areas:
 1. fuel handling and storage (including the potential for criticality).
 2. chemistry and radiochemistry.
 3. engineering.
 4. radiation protection, and
 5. quality assurance.

If necessary, individuals with knowledge and experience in other functional areas may be utilized to provide advice to the IRAC.

- c. The IRAC shall hold at least one meeting per quarter.
- d. A quorum shall consist of three regular members or their duly appointed alternates. Those members representing the line organizations responsible for the operation and maintenance of the facility shall not constitute a majority of the

6.0 ADMINISTRATIVE CONTROLS (Continued)

quorum. At least one member of the quorum shall be the chairman or the chairman's designated alternate.

- e. As a minimum, the IRAC shall perform the following functions:
 - 1. advise the Decommissioning Manager (or a designee) on all matters related to safe storage of irradiated fuel;
 - 2. advise the management of the audited organization and the Decommissioning Manager (or a designee) of audit results as they relate to safe storage of irradiated fuel;
 - 3. recommend to the management of the audited organization, and its management, any corrective action to improve the safe storage of irradiated fuel; and
 - 4. notify the President of Yankee Atomic Electric Company (or a designee) of any safety significant disagreement between the IRAC and the Decommissioning Manager within 24 hours.

- f. The IRAC shall be responsible for reviewing:
 - 1. the safety evaluations for procedures, and changes thereto, completed under the provisions of 10 CFR 50.59 to verify that such actions do not involve an unreviewed safety question as defined in 10 CFR 50.59. This review may be completed after implementation of the affected procedure;
 - 2. changes to structures, systems, or components important to the safe storage of irradiated fuel to verify that such changes do not involve an unreviewed safety question as defined in 10 CFR 50.59. This review may be completed after implementation of the change;
 - 3. tests or experiments involving the safe storage of irradiated fuel to verify that such tests or experiments do not involve an unreviewed safety question as defined in 10 CFR 50.59. This review may be completed after performance of the test or experiment;
 - 4. proposed changes to the YNPS Technical Specifications or the license;
 - 5. violations of codes, regulations, orders, license requirements, or internal procedures/instructions having nuclear safety significance;

6.0 ADMINISTRATIVE CONTROLS (Continued)

6. indications of unanticipated deficiencies in any aspect of design or operation of structures, systems, or components that could affect safe storage of irradiated fuel;
7. significant accidental, unplanned, or uncontrolled radioactive releases, including corrective action(s) to prevent recurrence;
8. significant operating abnormalities or deviations from normal and expected performance of equipment that affect safe storage of irradiated fuel;
9. the performance of the corrective action system; and
10. internal and external experience information related to the safe storage of irradiated fuel that may indicate areas for improving facility safety.

Reports or records of these reviews shall be forwarded to the Decommissioning Manager within 30 days after completion of the review.

- g. The IRAC's audit responsibilities shall encompass:
 1. conformance of irradiated fuel storage to provisions contained within the YNPS Technical Specifications and applicable license conditions at least once per 12 months;
 2. the training and qualifications of facility staff at least once per 12 months;
 3. implementation of all programs required by YNPS Technical Specification 6.7 at least once per 24 months;
 4. actions taken to correct deficiencies occurring in structures, systems, components, or methods of operation that affect safe storage of irradiated fuel at least once per 6 months;
 5. facility operations, modifications, maintenance, and Surveillance related to the safe storage of irradiated fuel to verify independently that these activities are performed safely and correctly at least once per 24 months; and

6.0 ADMINISTRATIVE CONTROLS (Continued)

6. other activities and documents as requested by the Decommissioning Manager (or a designee).

Reports of records of these audits, including any recommendations for improving the safe storage of irradiated fuel, shall be forwarded to the Decommissioning Manager (or a designee) within 30 days after completion of the audit.

6.5.3 Records

Written records of reviews and audits shall be maintained. As a minimum, these records shall include:

- a. Results of the activities conducted under the provisions of Specifications 6.5.1 and 6.5.2;
- b. Recommendations to the management of the audited organization;
- c. An assessment of the safety significance of review or audit findings;
- d. Documentation of reviews conducted under Specification 6.5.1.c; and
- e. Determination of whether each item considered under Specifications 6.5.2.f.1 through 6.5.2.f.3 involves an unreviewed safety question as defined in 10CFR50.59.

6.0 ADMINISTRATIVE CONTROLS (Continued)

Pages 6-10 through 6-12 have been deleted.

6.0 ADMINISTRATIVE CONTROLS (Continued)

6.6 REPORTABLE EVENT ACTION

- 6.6.1 The following actions shall be taken for REPORTABLE EVENTS:
- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.73, and
 - b. Each REPORTABLE EVENT shall be reviewed by an Independent Safety Reviewer and the results of this review shall be submitted to the Independent Review and Audit Committee (IRAC) and the Decommissioning Manager.

6.7 PROCEDURES AND PROGRAMS

- 6.7.1 Written procedures shall be established, implemented, and maintained that meet or exceed the requirements and recommendations of Sections 5.2 through 5.2.9 and 5.3 of ANSI N18.7-1972 and Appendix "A" of Regulatory Guide 1.33, Revision 2, except as provided in 6.7.2 and 6.7.3 below. The written procedures shall also cover the activities relating to:
- a. Fire Protection Program implementation.
 - b. PROCESS CONTROL PROGRAM implementation.
 - c. OFF-SITE DOSE CALCULATION MANUAL implementation.
 - d. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.
- 6.7.2 Each procedure and administrative policy of 6.7.1 above, and changes thereto, shall be reviewed by an Independent Safety Reviewer and approved by the Decommissioning Manager prior to implementation and reviewed periodically as set forth in administrative procedures.
- 6.7.3 Deleted.

6.0 ADMINISTRATIVE CONTROLS (Continued)

- 6.7.4 Temporary changes to procedures of 6.7.1 above may be made provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant management staff, at least one of whom is a Certified Fuel Handler.
 - c. The change is documented and approved by the Decommissioning Manager within 14 days of implementation.
- 6.7.5 The following programs shall be established, implemented, and maintained:
- a. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

 - 1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation, including surveillance tests and setpoint determination in accordance with the methodology in the ODCM;
 - 2) Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 CFR, Part 20, Appendix B, Table II, Column 2;
 - 3) Monitoring, sampling, and analysis of radioactive liquid, and gaseous effluents in accordance with 10 CFR 20.106 and with the methodology and parameters in the ODCM;
 - 4) Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released

6.0 ADMINISTRATIVE CONTROLS (Continued)

6.9.2 (continued)

- f. Records of transient or operational cycles for the Reactor Pressure Vessel.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of inservice inspections performed pursuant to Technical Specifications.
- i. Records of Quality Assurance activities required by the QA manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of Independent Safety Reviews and the IRAC meetings, and Records of the Plant Operational Review Committee (PORC) and the Nuclear Safety Audit and Review Committee (NSARC), the review and audit functions which preceded the Independent Safety Review function and IRAC.
- l. Records for Environmental Qualification.
- m. Records of analysis required by the Radiological Environmental Monitoring Program.
- n. Records of the service lives of all snubbers, including the date at which the service life commences and associated installation and maintenance records.
- o. Records of reviews performed for changes made to the OFF-SITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

6.10 RADIATION PROTECTION PROGRAM

- 6.10.1 Procedures for personnel radiation protection shall be prepared consistent with requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposures.

6.0 ADMINISTRATIVE CONTROLS (Continued)

6.12 PROCESS CONTROL PROGRAM (PCP)

6.12.1 Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.9.2.o. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluation justifying the change(s), and
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of federal, state, or other applicable regulations.
- b. Shall become effective after review and acceptance by an Independent Safety Reviewer and the approval of the Decommissioning Manager.

6.0 ADMINISTRATIVE CONTROLS (Continued)

6.13 OFF-SITE DOSE CALCULATION MANUAL (ODCM)

6.13.1 Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.9.2.o. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluation justifying the change(s), and
 - 2) A determination that the change will maintain the level of the radioactive effluent control required by 10 CFR 20.106, 40 CFR 190, 10 CFR 50.36a, and Appendix I to 10 CFR 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
- b. Shall become effective after review and acceptance by an Independent Safety Reviewer and the approval of the Decommissioning Manager.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 154 TO POSSESSION ONLY LICENSE NO. DPR-3
YANKEE ATOMIC ELECTRIC COMPANY
YANKEE NUCLEAR POWER STATION
DOCKET NO. 50-29

1.0 INTRODUCTION

By letter dated March 17, 1999, as supplemented by letters dated April 23, 1999, July 21, 1999, November 2, 1999, and March 6, 2000, Yankee Atomic Electric Company (YAEC, the licensee) submitted a request for a license amendment to revise the administrative controls (Section 6) of the Yankee Nuclear Power Station Technical Specifications (TS). The revision eliminates the positions of Manager of Operations and Plant Superintendent and assigns their responsibilities to the Decommissioning Manager. Additionally, the revision eliminates the Plant Operations Review Committee (PORC) and the Nuclear Safety Audit Review Committee (NSARC) and replaces them with an Independent Safety Reviewer and an Independent Review and Audit Committee (IRAC).

The April 23, July 21, and November 2, 1999, and March 6, 2000, letters provided additional clarifying information and updated TS pages. This information was within the scope of the original application and *Federal Register* notice and did not change the staff's initial proposed no significant hazards consideration determination.

2.0 EVALUATION

2.1 General

There are no accidents or other events in the Final Safety Analysis Report, as updated on June 28, 1999, that would result in an immediate threat to the public or the plant staff, or result in offsite doses in excess of the Environmental Protection Agency Protective Action Guides.

2.2 Review and Audit

The licensee proposed that TS Section 6.5 be revised to:

1. Eliminate the positions of Manager of Operations and Plant Superintendent and assign their responsibilities to the Decommissioning Manager.
2. Eliminate the Plant Operations Review Committee (PORC) and the Nuclear Safety Audit Review Committee (NSARC) and replace them with an Independent Safety Reviewer and an Independent Review and Audit Committee (IRAC).

Additionally, the licensee stated that other sections of the TS would be revised editorially to be in accordance with the changes made to Section 6.5. These editorial changes included Sections 6.1, 6.2, 6.6, 6.7, 6.9, 6.12, and 6.13.

Section 13.4, "Operational Review," of NUREG-0800, the "Standard Review Plan" (SRP), provides the acceptance criteria used by the staff to evaluate TS provisions related to the plant staff review of operational activities performed by licensee organizational units fulfilling the review and audit function. This acceptance criterion is based on meeting the relevant requirements of 10 CFR 50.40(b) as it relates to the licensee being technically qualified to engage in licensed activities, and of Appendix B to 10 CFR Part 50 as it relates to the review and audit functions required by the licensee's quality assurance program. Therefore, TS provisions associated with the review and audit function satisfies the criteria in both Section 50.36(c)(6), and Appendix B to 10 CFR Part 50.

The licensee stated that, due to the permanent cessation of power operation and the advanced state of decommissioning, it proposed eliminating the positions of Manager of Operations and Plant Superintendent and assigning the responsibilities to the Decommissioning Manager. The licensee stated that the proposed change would not eliminate any of the duties and responsibilities of the Manager of Operations and Plant Superintendent. The licensee noted that the Decommissioning Manager was the current acting Manager of Operations.

The licensee also proposed to replace the functions of the onsite committee, the PORC, with an Independent Safety Review performed by independent safety reviewers. The licensee's proposal included definition of the qualifications and responsibilities of the independent safety reviewers. Additionally, the licensee proposed to replace the functions of the offsite committee, the NSARC, with an IRAC. The licensee stated the changes were justified based on the reduced scope and complexity of operations at the station and the reduced staff. The licensee stated these reductions made the continued operation of PORC and NSARC impracticable and unnecessary. The licensee also stated that the responsibilities of the new review organizations would encompass the majority of the functions currently performed by the PORC and NSARC and be consistent with the scope of activities at a permanently defueled facility in an advanced stage of decommissioning.

In answer to NRC questions asked in telephone calls on June 21, 1999, and September 29, 1999, the licensee provided additional information and clarifications in letters dated July 21, 1999 (BYR 99-051), November 2, 1999 (BYR 99-072), and March 6, 2000 (BYR 2000-015).

Regarding independence, the licensee clarification stated that YAEC would ensure sufficient independence of the independent reviewers through explicit administrative controls and that the controls would be comparable to the controls in place for PORC and NSARC. The licensee stated that the Decommissioning Manager would be responsible for the assignment of personnel to perform the independent review function. Further, the licensee stated that the Decommissioning Manager would report to the Vice President of YAEC.

Regarding audits, the licensee clarification stated that YAEC was committed to ANSI 18.7 (1976) which included all 18 criteria of 10 CFR Part 50, Appendix B. The licensee also clarified that, although it appeared to have eliminated the requirement for audits of the Emergency Plan and

the Security Plan, it fully intended to perform audits of the Emergency Plan, the Security Plan, and the Security Training and Qualification Plan. The licensee noted that 10 CFR 50.54(p)(3), Appendix C to Part 73, and 10 CFR 50.54(t) require a review and audit of these programs at least every 12 months. Therefore, the licensee stated that maintaining a separate requirement for these audits in the TS was redundant and unnecessary.

Regarding responsibilities of IRAC, the licensee clarification stated that IRAC is responsible for advising both the Decommissioning Manager (a contractor) and the President of YAEC, or designee.

Regarding the IRAC's five expert disciplines versus ten disciplines in ANSI 18.7, the licensee stated that the five disciplines were all that was needed for foreseeable work. The licensee further stated that other expert disciplines would be made available to IRAC if some future need arose. The licensee added a clarification to TS 6.5.2.b. which stated that other individuals with knowledge and experience in other functional areas would be used to provide advice to the IRAC if necessary.

Regarding the standards for determining the equivalent experience for a Bachelor of Science Degree, the licensee clarification stated that ANSI 18.1- 1981 would be used.

2.3 Conclusion

Based on the advanced stage of decommissioning of Yankee Nuclear Power Station, reductions in the review and audit commitments can be accomplished consistent with the requirements of Appendix B to 10 CFR Part 50. The licensee has proposed appropriate controls to ensure the qualification and independence of the IRAC. The assigned review responsibilities are consistent with the advanced stage of facility decommissioning. The staff concludes that the modified organization, including the revised review and audit functions, provides reasonable assurance of compliance with the requirements of Appendix B to 10 CFR Part 50 for activities performed under the Yankee Decommissioning Quality Assurance Program. Therefore, the proposed amendment is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Commonwealth of Massachusetts State official was notified of the proposed issuance of the amendment. The Massachusetts State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes recordkeeping, reporting, and administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Paul Narbut

Date: June 20, 2000

June 20, 2000

Mr. Merrill Atkins
Regulatory Affairs Manager
Yankee Atomic Electric Company
Midstate Office Park Suite 200/210
Auburn, MA 01501

SUBJECT: YANKEE NUCLEAR POWER STATION - ISSUANCE OF AMENDMENT TO
REVISE TECHNICAL SPECIFICATIONS RELATED TO THE QUALITY
ASSURANCE PROGRAM (TAC NO. MA5032)

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Sincerely,

/RA/

Phillip M. Ray, Project Manager
Decommissioning Section
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-29

Enclosures: 1. Amendment No. 154 to DPR-3
2. Safety Evaluation

cc w/encls: See next page

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