

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

**Nuclear Business Unit** 

JUN 0 5 2008

LRN-00-0211

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555

Gentlemen:

LICENSEE EVENT REPORT 354/00-004-00 HOPE CREEK GENERATING STATION FACILITY OPERATING LICENSE NO NPF-57 DOCKET NO. 50-354

This Licensee Event Report entitled "Reactor Scram with Reactor Defueled Due to Scram Discharge Volume High Level" is being submitted in accordance with the requirements of 10CFR50.73(a)(2)(iv) as "an event or condition that resulted in a manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS),...."

Sincerely,

Mark Bezilla

**Vice President - Operations** 

Attachment

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Distribution:

LER File 3.7

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On M	ay	6, 200	0, a S	CRAM si	gnal	was q	ener	ated a	as a	a re:	sult of	hiah	level	l in the	
Scra	m D	ischar	ae Vol	ume (SD	V).	At the	e ti	me of	th	e eve	ent, the	reac	tor	vae in s	
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Scram Discharge Volume (SDV). At the time of the event, the reactor was in a refueling outage with the core completely offloaded to the fuel pool, all control rods were inserted and the control rod drive hydraulic system was isolated. Work was being performed that required isolating air to the scram discharge volume vent and drain valves, resulting in these valves being closed. With the plant in this configuration Control Rod Drive leakage is directed to the scram discharge volume. Water level continued to rise in the discharge volume until the high scram discharge volume level scram setpoint was reached. This event resulted from an inadequate partial release of a tagout and untimely action to drain the SDV. Corrective action is to perform a review of tagouts in order to identify enhancements that will prevent recurrence of this type of event. This event is being reported pursuant to 10CFR50.73(a)(2)(iv) as an "event or condition that resulted in a manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS),...."

NRC FORM 366A	U.S. NUCLEAR RE	GULATORY	COMMISSION
(6-1998)			

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)	L	PAGE (3)			
Hope Creek Generating Station	05000354	YEAR	SEQUENTIAL REVISION NUMBER NUMBER	8	OF	3
		00	- 004 - 00			

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor (BWR/4)
Control Rod Drive System - EIIS Identifier {AA/ISV}\*
Reactor Protection System - EIIS Identifier {JC/LS}

\* Energy Industry Identification System (EIIS) codes and component function identifier codes appear as {SS/CC}

## CONDITIONS PRIOR TO OCCURRENCE

The plant was in the Undefined Operational Condition with all fuel removed from the reactor vessel. The control rods were fully inserted and the control rod drive system was removed from service. No other structures, systems, or components were inoperable at the time of the occurrence that contributed to the event.

### DESCRIPTION OF OCCURRENCE

On May 6, 2000, at 2324, a full Reactor Protection System (RPS) SCRAM occurred due to high level in the scram discharge volume (SDV). Prior to the SCRAM, Operators had performed a standard tagout which completely deenergizes the SCRAM related components in their fail safe positions to support activities on the alternate rod insertion valves thus eliminating the potential for a SCRAM. On May 5, in order to support the performance of a surveillance, operators performed a partial release on a standard tagout associated with the scram discharge volume. The partial release reenergized the SCRAM logic. Later on May 5, Operators filled the CRD system and placed a CRD pump in service to support re-coupling of a control rod and other items. After the CRD pump was placed in service, minor water inleakage was observed into the SDV. This is not unexpected. Rising level resulted in a Rod Block alarm and operations drained the SDV early on May Late on May 6, the SDV high level once again occurred and operators were taking actions to manually open the SDV valves in order to drain the water out of the SDV when the SCRAM occurred.

# APPARENT CAUSE OF OCCURRENCE

The partial release prepared by the Licensed Operators did not adequately evaluate and address the effects of the revised configuration. This led to the conditions which permitted the SCRAM to occur.

NRC FORM 366A	U.S. NUCLEAR REGULATORY COMMISSION	
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# SAFETY SIGNIFICANCE AND IMPLICATIONS

There were no actual safety consequences associated with this condition. The reactor was shut down with all fuel off loaded to the fuel pool. This event did not affect the health and safety of the public.

## PREVIOUS OCCURRENCES

A review has been conducted of Licensee Event Reports and Inspection reports for 1998, 1999 and 2000 in order to identify similar events. No similar ESF actuations resulting from tagging/releases were identified.

## CORRECTIVE ACTIONS

The tagouts which were used are being reviewed and tagout revisions for this tagging evolution are being considered in order to provide additional guidance to minimize challenges to plant operations.

#### COMMITMENTS

The corrective action cited in this LER is a voluntary enhancement and does not constitute a commitment.

NC.LR-AS.ZZ-0001(Z) Attachment 3 Rev 1, LER PROCESS AND PERFORMANCE MONITORING

LICE	NSEE EVENT REPORT PR	OCESS	PERFO	RMAN	CE INDIC	ATORS	
EVALUATION MANAGER	LICENSING STAFF	OTHER ORGANIZATIONS	Process Indicators and Measurement	Targets	Criteria	hecking	
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Provides detailed input. Description, root cause and	LE Develops Draft LER	Provide additional information as required	Q1 Adequate Input received	75%	Sufficient info to draft LER.	Quarterly	Regiona Lead
corrective actions. Q1, T1	Distribute for Comments	required	T1 Input timely T2 - % of time draft to internal reviewers on time	75% 70%	Day 14 Copy to reviewer by day 16.	Quarterly	Regiona Lead Regiona Lead
Review / Concur Comments	Incorporate Comments Prepare Final Draft	Review / Concur Q2 Concur	Q2 - % of time with no valid comments from reviewers.	80%	Other than phrasing	Quarterly	Regions lead
T3 Sign Traveler	Prepare SORC Review Copy	Sign Traveler T3	T3 % of time Traveler signed on time.	80%	Prior to SORC meeting.	Quarterly	Regiona Lead
	Provide Copy to SORC Secy	NO SORC Approval	T4 - % of time draft to SORC Secy before meeting	80%	Electronic copy by the week Prior	Quarterly	Region Lead
	Assemble Validation Pkg	Q3 YES	Q3 % of Time LER SORC Approved	90%	Multiple SOR0 mtg reqd	Quarterly	Regiona Lead
	E Q4 Transmit to NRC	VP Review and Signature	Q4 - % of initial requiring no supplement.	95%	Form 366 Box checked	Quarterly	Regiona Lead
		T5	T5 LERs submitted to VP on time.	100%	1 day prior To Due date	Quarterly	Regiona Lead