

June 12, 2000

Mr. Craig G. Anderson
Vice President, Operations ANO
Entergy Operations, Inc.
1448 S. R. 333
Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT 2 - REQUEST FOR ADDITIONAL
INFORMATION ON PROPOSED RISK-INFORMED LICENSE CHANGE
REGARDING STEAM GENERATOR TUBING (TAC NO. MA8418)

Dear Mr. Anderson:

During a public meeting on June 8, 2000, we discussed your proposed risk-informed license change regarding steam generator tubing for the remainder of Cycle 14. At the conclusion of our meeting, we indicated that additional information would be needed to continue our review. Accordingly, enclosed is a request for additional information (RAI). As we are still working on some additional questions, there will be another RAI in the near future.

The contents of this RAI have been discussed with Mr. Dale James of your staff, and a response by June 19, 2000, was agreed to. The staff appreciates your efforts in regard to this matter.

If for any reason this date becomes unreasonable, please contact me at your earliest opportunity.

Sincerely,

/RA/

Thomas W. Alexion, Project Manager
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-368

cc: See next page

Enclosure: RAI

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Accession No.:ML003722701

*06/09/00 e-mail for Q12

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Request for Additional Information

1. Provide a description of the plant changes required to support the new depressurization procedure.
2. Based on a description of each of the dominant scenarios that contribute to the high/dry portion of your core damage frequency, provide a description of the timing, and in particular the timing of cues relative to plant state. Similarly, provide a description of the cues, and what instrumentation is needed to provide those cues.
3. Provide a copy of the procedures leading to and including the depressurization action, indicating the entry conditions.
4. Provide a discussion of the training on the new depressurization procedure.
5. Provide a description of the actions necessary to perform the task, including an identification of who performs the actions, and where.
6. Provide a rough estimate of the time required to perform the task.
7. Provide an estimate of the interval of time required, after occurrence of the depressurization procedure initiation cues, in order to achieve a probability of 0.25 or less that the human actions needed for depressurization are not yet completed.
8. Provide the correlation for the Larson-Miller creep damage parameter, used for the stainless steel surge line in your thermal-hydraulic analyses with the Modular Accident Analysis Program (MAAP) computer code.
9. Provide the fraction of tubes currently plugged in each steam generator. If sleeves are currently installed, include their effect on net flow rate as its equivalent in number of plugged tubes.
10. Provide any other parameter changes from the conditions specified in your previously submitted document titled Calc No.: 99-E-0019-02 "ANO-2 MAAP and PROBFIL Calculations."
11. Using the most recently provided estimates of flaw growth rates and the probability of detection as a function of flaw size during your most recent inspection (2P99), provide the probability distributions for the stress magnification factors for partial through-wall cracks (m_p) and through-wall cracks (m) for each of these three points in time during your current operating interval: 1) start-up the fall of 1999, 2) June 15, 2000, without inspection, and 3) September 15, 2000, without inspection.
12. During the meeting between Entergy Operations, Inc. (Entergy) and the Nuclear Regulatory Commission on June 8, 2000, Entergy staff stated that an improved eddy current testing calibration standard had been used for inspections conducted during the 2P99 outage. They also indicated that they had performed a study in which the eddy current testing data taken with the new and improved calibration standard was compared

to data taken with the previously used calibration standard for 30 some indications. Please provide the referenced eddy current data for review by the staff. Also, provide bobbin probe eddy current data for a random sample of 12 tubes covering the full length of the tubes and the rotating pancake coil eddy current data for a random sample of 12 tube/tube support plate intersections. These latter data should be those taken with the calibration standard of record, i.e., the new and improved standard, for the examinations.

[More questions, including specifications for a MAAP analysis, will be provided based on the results of these questions.]

Arkansas Nuclear One

cc:

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& Chief Operating Officer
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Russellville, AR 72801