



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931**

June 7, 2000

Mr. George Grabryan, Director
Florence/Lauderdale County EMA
City Hall Basement Room B-25
Florence AL 35630

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Grabryan:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

We recently completed a six-month pilot test of the revised Reactor Oversight Program at selected facilities and concluded that the revised process is more objective, timely, and focuses us on the areas of greatest risk and safety significance. In April, the NRC applied this process to all 103 operating nuclear power reactors. We believe this process will allow the NRC to maintain safety, improve efficiency, and reduce unnecessary regulatory burdens on the reactor operator. These improvements will also ensure that the nuclear industry continues to operate in a way that best protects public health and safety.

An important feature of the new process is that you, through the NRC Web site, will have access to the same information we use in our quarterly assessment of plant performance. This information may be accessed at <http://www.nrc.gov/NRR/OVERSIGHT>.

Enclosed is a news release to be issued announcing a public meeting in your area to discuss the new program. It will be held at 7:00 p.m. on June 27, 2000, at the Athens/Limestone Emergency Management Agency Building, 1011 West Market St., Athens, AL. I am inviting you, and other members of your organization, to attend if you are interested in learning more about the revised process. We have also enclosed additional information describing the new reactor inspection and oversight program.

Also on June 27, we will present the results of the Plant Performance Review for Browns Ferry for the period of February 1, 1999, through January 31, 2000. This presentation will be held at the Browns Ferry Nuclear Plant Training Center, Corner of Shaw and Nuclear Plant Roads, Athens, AL at 2:00 p.m. and is open to the public.

Grabryan

2

Should you have any questions, please call me at (404) 562-4530.

Sincerely,

/RA by Peter A. Taylor acting for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Docket Nos. 50-259, 50-260, 50-296
License No. DPR-33, DPR-52, DPR-68

Enclosure: As stated

Grabryan

2

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Docket Nos. 50-259, 50-260, 50-296
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Enclosure: As stated

OFFICE	RII:DRP						
SIGNATURE	PAT						
NAME	PTaylor alt						
DATE	06/07/2000	6/ /2000	6/ /2000	6/ /2000	6/ /2000	6/ /2000	6/ /2000
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY

DOCUMENT NAME: C:\BF pub official ltr.wpd

Mr. Spencer Black, Director
Athens/Limestone County EMA
1011 W. Market Street
Athens AL 35611

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Black:

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. Brett A. Garrison, Director
Huntsville/Madison County EMA
PO Box 308
Huntsville AL 35804

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Garrison:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. H. Eddie Hicks, Director
Morgan County EMA
PO Box 668
Decatur AL 35602

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Hicks:

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Ms. Ann Coffey, Coordinator
Lawrence County EMA
555 Walnut Street SE
Moulton AL 35650

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Ms. Coffey:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

The Honorable Eddie Frost
Mayor, City of Florence
PO Box 98
Florence AL 35631

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Frost:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Should you have any questions, please call me at (404) 562-4530.

Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

The Honorable Barbara Coffey
Mayor, City of Moulton
202 West Court Street
Moulton AL 35650

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Ms. Coffey:

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

The Honorable Dan Williams
Mayor, City of Athens
PO Box 1089
Athens AL 35611

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Williams:

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

The Honorable Julian Price
Mayor, City of Decatur
PO Box 308
Decatur AL 35602

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Price:

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Should you have any questions, please call me at (404) 562-4530.

Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

The Honorable Loretta Spencer
Mayor, City of Huntsville
PO Box 308
Huntsville AL 35801

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Ms. Spencer:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Honorable Dewey Mitchell
Judge of Probate
PO Box 1059
Florence AL 35631

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Mitchell:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Ms. Joan Lang
Lawrence County Administrator
Lawrence County Annex Building
750 Main Street Suite 1
Moulton AL 35650

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Ms. Lang:

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Sincerely,
/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. Bradley Cross, Chairman
Lawrence County Commission
Lawrence County Annex Building
750 Main Street Suite 1
Moulton AL 35650

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Cross:

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Sincerely,
/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. Stanley Menefee, Chairman
Limestone County Commission
310 W. Washington Street
Limestone County Annex Bldg
Athens AL 35611

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Menefee:

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/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
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Enclosure: As stated

Mr. Mike Gillespie, Chairman
Madison County Commission
Madison County Courthouse
Huntsville AL 35801

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Gillespie:

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We recently completed a six-month pilot test of the revised Reactor Oversight Program at selected facilities and concluded that the revised process is more objective, timely, and focuses us on the areas of greatest risk and safety significance. In April, the NRC applied this process to all 103 operating nuclear power reactors. We believe this process will allow the NRC to maintain safety, improve efficiency, and reduce unnecessary regulatory burdens on the reactor operator. These improvements will also ensure that the nuclear industry continues to operate in a way that best protects public health and safety.

An important feature of the new process is that you, through the NRC Web site, will have access to the same information we use in our quarterly assessment of plant performance. This information may be accessed at <http://www.nrc.gov/NRR/OVERSIGHT>.

Enclosed is a news release to be issued announcing a public meeting in your area to discuss the new program. It will be held at 7:00 p.m. on June 27, 2000, at the Athens/Limestone Emergency Management Agency Building, 1011 West Market St., Athens, AL. I am inviting you, and other members of your organization, to attend if you are interested in learning more about the revised process. We have also enclosed additional information describing the new reactor inspection and oversight program.

Also on June 27, we will present the results of the Plant Performance Review for Browns Ferry for the period of February 1, 1999, through January 31, 2000. This presentation will be held at the Browns Ferry Nuclear Plant Training Center, Corner of Shaw and Nuclear Plant Roads, Athens, AL at 2:00 p.m. and is open to the public.

Should you have any questions, please call me at (404) 562-4530.

Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. C. G. Chaffin
Chairman, EMA Board
153 Little Mountain Road
Huntsville AL 35803

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Chaffin:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. Larry Bennich, Chairman
Morgan County Commission
PO Box 668
Decatur AL 35602

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Bennich:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Should you have any questions, please call me at (404) 562-4530.

Sincerely,

/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. Lee Helms
Acting Assistant Director
Alabama Emergency Management
Agency
PO Drawer 2160
Clanton AL 35045-5160

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Helms:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

We recently completed a six-month pilot test of the revised Reactor Oversight Program at selected facilities and concluded that the revised process is more objective, timely, and focuses us on the areas of greatest risk and safety significance. In April, the NRC applied this process to all 103 operating nuclear power reactors. We believe this process will allow the NRC to maintain safety, improve efficiency, and reduce unnecessary regulatory burdens on the reactor operator. These improvements will also ensure that the nuclear industry continues to operate in a way that best protects public health and safety.

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Sincerely,
/RA by Peter A. Taylor for/
Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. Kirksey E. Whatley, Director
Division of Radiation Control
State Department of Public Health
State Office Building
Montgomery AL 36130-1701

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Whatley:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

We recently completed a six-month pilot test of the revised Reactor Oversight Program at selected facilities and concluded that the revised process is more objective, timely, and focuses us on the areas of greatest risk and safety significance. In April, the NRC applied this process to all 103 operating nuclear power reactors. We believe this process will allow the NRC to maintain safety, improve efficiency, and reduce unnecessary regulatory burdens on the reactor operator. These improvements will also ensure that the nuclear industry continues to operate in a way that best protects public health and safety.

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Should you have any questions, please call me at (404) 562-4530.

Sincerely,
/RA by Peter A. Taylor for/

Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated

Mr. John B. Copenhaver
Regional Director
Federal Emergency Management
Agency Region IV
3003 Chamblee-Tucker Road
Atlanta GA 30341

June 7, 2000

SUBJECT: NRC REACTOR OVERSIGHT PROGRAM MEETING

Dear Mr. Copenhaver:

The U. S. Nuclear Regulatory Commission (NRC) recently revised its reactor inspection and oversight process. These revisions change the manner in which we conduct safety inspections and assess performance at NRC-licensed nuclear power plants, such as the Browns Ferry Nuclear Plant near you.

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Sincerely,
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Paul E. Fredrickson, Chief
Reactor Projects Branch 6
Division of Reactor Projects

Enclosure: As stated



NRC NEWS

**UNITED STATES NUCLEAR REGULATORY COMMISSION
OFFICE OF PUBLIC AFFAIRS, REGION II**

61 Forsyth Street SW, Atlanta GA 30303
Web Site: www.nrc.gov/OPA

CONTACT: Ken Clark (404)562-4416/e-mail: kmc2@nrc.gov
Roger D. Hannah (404)562-4417/e-mail: rdh1@nrc.gov

NRC TO HOLD PUBLIC MEETINGS IN ALABAMA JUNE 27 ON CHANGES IN REACTOR OVERSIGHT AND LATEST ASSESSMENT OF BROWNS FERRY PLANT

The Nuclear Regulatory Commission has scheduled two public meetings for Tuesday, June 27

-- one on the latest performance review results for the Tennessee Valley Authority's Browns Ferry nuclear power plant, and the other meeting on the NRC's reactor oversight process.

The first meeting will be held at 2 p.m. EDT at the Browns Ferry Training Center at the site near Athens, Alabama. During the meeting, NRC staff members will discuss Browns Ferry's latest plant performance review results, and will be available after the meeting to answer any questions from media or members of the public.

The Plant Performance Reviews are used to evaluate safety performance information and identify any changes in plant performance so the NRC can allocate inspection resources appropriately. The text of each PPR letter, including the one on Browns Ferry, is available from the NRC Office of Public Affairs and has been posted on the NRC web site at: www.nrc.gov/OPA/ppr.

Current performance information for the two operating units at the Browns Ferry plant is available on the NRC web site at:

www.nrc.gov/NRR/OVERSIGHT/ASSESS/BF2/bf2_chart.html and
www.nrc.gov/NRR/OVERSIGHT/ASSESS/BF3/bf3_chart.html.

The second meeting will begin at 7 p.m. EDT in the Athens/Limestone Emergency Management Agency Building, 1011 Market Street in Athens, Alabama. NRC officials will describe the new oversight process which began at the Browns Ferry plant in April. Members of the public will have the opportunity to ask questions during the meeting.

The new oversight process uses more objective, timely, and safety-significant criteria in assessing performance, while seeking to more effectively and efficiently regulate the industry. Under the new process, inspections by NRC resident inspectors and regional office-based personnel are supplemented by statistical measures of plant activities called performance indicators. These performance indicators measure important safety aspects of plant operations, including the viability of redundant safety systems, emergency planning, radiation protection, and security measures.

Inspection findings and performance indicators for each nuclear power plant are posted on the NRC's web site every three months. The safety significance of each inspection finding or performance indicator is characterized by a color -- green, white, yellow, or red.

The NRC response to the inspection findings and performance indicators will be based on the significance of the items. A green finding receives normal NRC oversight, while white, yellow, or red assessments receive increasing NRC involvement. Multiple red findings could result in a plant shutdown.

Enclosure

A plain language summary of the reactor oversight process is available from the NRC Office of Public Affairs or on the web site at: www.nrc.gov/OPA/primer.htm.

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NRC Reactor Oversight Process

The Nuclear Regulatory Commission has revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new oversight process uses more objective, timely, and safety-significant criteria in assessing performance, while seeking to more effectively and efficiently regulate the industry. It also takes into account improvements in the performance of the nuclear industry over the past twenty years.

The NRC tested the new program at thirteen reactors at nine sites across the country on a pilot basis in 1999 to identify what things worked well and what improvements were called for before beginning initial implementation at all nuclear power plants. After a year of experience has been gained with the new oversight process at all plants, the Commission anticipates making further improvements based on this wider experience.

The impetus behind this comprehensive change in approach came both from the NRC's own fundamental reviews of its regulatory program as part of the "reinventing government" process and from concerns expressed by public interest groups, the nuclear industry, and Congress.

The NRC's mission is to ensure adequate protection of public health and safety as it relates to the peaceful uses of nuclear materials in the United States. The agency does not operate nuclear power plants. Rather it regulates the safe operation of the 103 commercial nuclear power plants by mandating requirements for the design, construction and operation of such plants. The NRC conducts a rigorous process for licensing plants to allow them to operate, as well as licensing individual plant operators. The agency provides continuous oversight of plant operations to verify they are being conducted in accordance with regulations.

The NRC also establishes plant specific technical specifications which must be followed by the plant operators to ensure that the proper combination of safety related equipment is available to safely shut down the plant in the event of an accident. The NRC has full authority to take whatever action is necessary to protect public health and safety and may demand immediate licensee actions, up to and including a plant shutdown.

The commercial nuclear power industry in the United States is a mature industry. Most of the plants have been operating for more than 10 years, and half of them have operated for more than 20 years. All the evidence suggests that the safety and reliability of the nuclear industry has improved markedly since the late 1980's and early 1990s. The number of automatic shutdowns, the number of significant safety problems, and the number of unplanned outages caused by equipment problems have all decreased. (See Glossary for definitions of terms).

The improvements in plant performance can be attributed both to efforts within the nuclear industry and to successful regulatory oversight. Despite this success, the NRC has noted that previous processes for inspection, assessment, and enforcement were not always focused on the most important safety issues. In some situations, regulatory activities have been redundant or inefficient and, at times, overly subjective. NRC actions were not always sufficiently understandable or predictable.

Enclosure

To address these concerns, the new oversight program calls for:

- Focusing inspections on activities where the potential risks are greater
- Applying greater regulatory attention to nuclear power plants with performance problems, while maintaining a normal level of regulatory attention on facilities that perform well
- Using objective measurements of the performance of nuclear power plants
- Giving both the public and the nuclear industry timely and understandable assessments of plant performance
- Reducing unnecessary regulatory burden on nuclear facilities
- Responding to violations of regulations in a predictable and consistent manner that reflects the potential safety impact of the violations

The key features of the program deal with new methods for inspecting and assessing performance to ensure safe operation. It spells out more clearly what a nuclear plant operator can expect from the NRC with good plant performance and what can be expected from the NRC if performance declines.

Setting the Cornerstones of Safe Operation

The new reactor oversight program is, of course, anchored in the NRC's mission to ensure public health and safety in the operation of commercial power plants. That will always remain the agency's overarching responsibility.

The objective is to monitor performance in three broad areas -- reactor safety (avoiding accidents and reducing the consequences of accidents if they occur); radiation safety for both plant workers and the public during routine operations; and protection of the plant against sabotage or other security threats.

To measure plant performance, the oversight program focuses seven on specific "cornerstones" which support the safety of plant operations in the three broad strategic areas.

Initiating Events - This cornerstone focuses on operations and events at a nuclear plant that could lead to a possible accident, if plant safety systems did not intervene. These events could include equipment failures leading to a plant shutdown, shutdowns with unexpected complications, or large changes in the plant's power output.

Mitigating Systems - This cornerstone measures the function of safety systems designed to prevent an accident or reduce the consequences of a possible accident. The equipment is checked by periodic testing and through actual performance.

Barrier Integrity - There are three important barriers between the highly radioactive materials in fuel within the reactor and the public and the environment outside the plant. These barriers are the sealed rods containing the fuel pellets, the heavy steel reactor vessel and associated piping, and the reinforced concrete containment building surrounding the reactor. The integrity of the fuel rods, the vessel, and the piping is continuously checked for leakage, while the ability of the containment to prevent leakage is measured on a regular basis.

Emergency Preparedness - Each nuclear plant is required to have comprehensive emergency plans to respond to a possible accident. This cornerstone measures the effectiveness of the plant staff in carrying out its emergency plans. Such emergency plans are

tested every two years during emergency exercises involving the plant staff and local, state, and, in some cases, federal agencies.

Occupational Radiation Safety - NRC regulations set a limit on radiation doses received by plant workers, and this cornerstone monitors the effectiveness of the plant's program to control and minimize those doses.

Public Radiation Safety - This cornerstone measures the procedures and systems designed to minimize radioactive releases from a nuclear plant during normal operations and to keep those releases within federal limits.

Physical Protection- Nuclear plants are required to have well-trained security personnel and a variety of protective systems to guard vital plant equipment, as well as programs to assure that employees are constantly fit for duty through drug and alcohol testing. This cornerstone measures the effectiveness of the security and fitness-for-duty programs.

In addition to the cornerstones, the reactor oversight program features three "cross-cutting" elements, so named because they affect and are therefore part of each of the cornerstones:

- **Human performance**
- **Management attention to safety and workers' ability to raise safety issues** (The so-called "safety-conscious work environment")
- **Finding and fixing problems** (The utility's corrective action program)

The review and assessment of these cross-cutting elements have an important role in the new program.

Measuring and inspecting nuclear plant performance

Nuclear plant performance will be measured by a combination of objective performance indicators and by the NRC inspection program. These will be closely focused on those plant activities having the greatest impact on safety and overall risk. In addition, the NRC will conduct both periodic and annual reviews of the effectiveness of each utility's programs to identify and correct problems.

Performance indicators use objective data to monitor performance within each of the "cornerstone" areas. The data which make up the performance indicators will be generated by the utilities and submitted to the NRC on a quarterly basis. Each performance indicator is measured against established thresholds which are related to their effect on safety. While performance indicators can provide insights into plant performance for selected areas, the NRC's inspection program provides a greater depth and breadth of information for consideration by the NRC in assessing plant performance.

The NRC will also monitor plant activities through its inspection program. The inspection program is designed to verify the accuracy of performance indicator information and to assess performance that is not directly measured by the performance indicator data.

Using performance indicators

The performance indicator data will be evaluated and integrated with findings of the NRC inspection program. Each of the performance indicators has criteria for measuring acceptable performance. (As in all industrial activities, nuclear power plants are not error-free or risk-free. Equipment problems and human errors will occur. Each performance indicator is designed to determine acceptable levels of operation within substantial safety margins.) These objective criteria are designed to reflect risk according to established safety margins, as indicated by a color coding system.

A “green” coding indicates performance within an expected performance level in which the related cornerstone objectives are met; “white” indicates performance outside an expected range of nominal utility performance but related cornerstone objectives are still being met; “yellow” indicates related cornerstone objectives are being met, but with a minimal reduction in safety margin; and “red” indicates a significant reduction in safety margin in the area measured by that performance indicator. The performance indicators will be reported to the NRC on a quarterly basis by each utility. Following compilation and review by the NRC staff, the performance indicators will be posted on the NRC’s web site.

Scope of inspections

The revised oversight program continues to utilize a variety of NRC inspectors who monitor plant activities. The program includes baseline inspections common to all nuclear plants. The baseline inspection program, based on the "cornerstone" areas, focuses on activities and systems that are "risk significant," that is, those activities and systems that have a potential to trigger an accident, can mitigate the effects of an accident, or increase the consequences of a possible accident. The inspection program will also review the “cross-cutting issues” of human performance, the “safety-conscious work environment,” and how the utilities find and fix problems. Inspections beyond the baseline will be performed at plants with performance below established thresholds, as assessed through information gained from performance indicators and NRC inspections. Additional inspections may also be performed in response to a specific event or problem which may arise at a plant.

The inspections will be performed by NRC resident inspectors stationed at each nuclear power plant and by inspectors based in one of the four NRC regional offices or in NRC headquarters in Rockville, Maryland. The regional offices are in King of Prussia, Pennsylvania; Atlanta, Georgia; Lisle, Illinois; and Arlington, Texas.

The new inspection program uses a “risk-informed” approach to select areas to inspect within each cornerstone. The inspection areas were chosen because of their importance from the point of view of potential risk, past operational experience, and regulatory requirements.

The baseline inspection program has three parts -- inspection of areas not covered by performance indicators or where a performance indicator does not fully cover the inspection area; inspections to verify the accuracy of a licensee's reports on performance indicators; and a thorough review of the utility's effectiveness in finding and resolving problems on its own.

Inspection reports will be issued for all inspections just as under the previous inspection program. The reports will be available to the public on the NRC’s internet web site and from its Public Document Room at NRC headquarters.

Assessing plant performance

The inspection staff has developed a procedure, called the "Significance Determination Process," to help inspectors determine the safety significance of inspection findings. This process will be used for an initial screening review to identify those inspection findings that would not result in a significant increase in risk and thus need not be analyzed further (a "green" finding) . Remaining inspection findings -- which may have an effect on plant risk -- will then be subject to a more thorough risk assessment, using the next phase of the Significance Determination Process. This more detailed assessment may involve NRC risk experts from the appropriate regional office and further review by the utility's plant staff. The final outcome of the review -- evaluating whether the finding is green, white, yellow, or red -- will be used to determine what further NRC action may be called for.

Each calendar quarter, the resident inspectors and the inspection staff in the regional office will review the performance of all nuclear power plants in that region, as measured by the performance indicators and by inspection findings. Every six months, this review will be expanded to include planning of inspections for the following 12-month period.

Each year, the final quarterly review will involve a more detailed assessment of plant performance over the previous 12 months and preparation of a performance report, as well as the inspection plan for the following year. This review will include NRC headquarters staff members, the regional staff, and the resident inspectors.

These annual performance reports will be available to the public on the agency's web site, and the NRC staff will hold public meetings with utilities to discuss the previous year's performance at each plant.

In addition, NRC senior management will review the adequacy of agency actions for plants with significant performance problems. The managers will also take a wider view both of the overall industry performance and of the performance of the agency's regulatory programs. The performance of plants requiring heightened agency scrutiny will then be discussed during a public meeting with the NRC Commissioners at the agency's Rockville, Maryland, headquarters.

NRC Response Plan or “Action Matrix”

Assessment of Plant Performance (in order of increasing safety significance)	NRC Response
<p>I. All performance indicators and cornerstone inspection findings GREEN</p> <ul style="list-style-type: none"> • Cornerstone objectives fully met. 	<ul style="list-style-type: none"> • Routine inspector and staff interaction • Baseline inspection program • Annual assessment public meeting
<p>II. No more than two WHITE inputs in different cornerstones</p> <ul style="list-style-type: none"> • Cornerstone objectives fully met. 	<p>Response at Regional level</p> <ul style="list-style-type: none"> • Staff to hold public meeting with utility management • Utility corrective action to address WHITE inputs • NRC inspection followup on WHITE inputs and corrective action
<p>III. One degraded cornerstone (two WHITE inputs or one YELLOW input or three WHITE inputs in any strategic area)</p> <ul style="list-style-type: none"> • Cornerstone objectives met with minimal reduction in safety margin 	<p>Response at Regional level</p> <ul style="list-style-type: none"> • Senior regional management to hold public meeting with utility management • Utility to conduct self-assessment with NRC oversight • Additional inspections focused on cause of degraded performance
<p>IV. Repetitive degraded cornerstone, multiple degraded cornerstones, or multiple YELLOW inputs, or one RED input</p> <ul style="list-style-type: none"> • Cornerstone objectives met with longstanding issues or significant reduction in safety margin 	<p>Response at Agency level</p> <ul style="list-style-type: none"> • Executive Director for Operations to hold public meeting with senior utility management • Utility develops performance improvement plan with NRC oversight • NRC team inspection focused on cause of degraded performance • Demand for Information, Confirmatory Action Letter, or Order
<p>V. Unacceptable Performance</p> <ul style="list-style-type: none"> • Unacceptable reduction in safety margin 	<p>Response at Agency level</p> <ul style="list-style-type: none"> • Plant not permitted to operate • Commission meeting with senior utility management • Order to modify, suspend, or revoke license

How the NRC will respond to plant performance

The quarterly reviews of plant performance, using both the performance indicators and inspection findings, will determine what additional action, if any, the NRC will take if there are signs of declining performance. This approach to enforcement is intended to be more predictable than previous practices by linking regulatory actions to performance criteria. The new process utilizes four levels of regulatory response with NRC regulatory review increasing as plant performance declines. The first two levels of heightened regulatory review are managed by the appropriate regional office. The next two levels call for an agency response, involving senior management attention from both headquarters and regional offices.

The oversight program retains the same tools used in the past for dealing with declining plant performance and violations. These tools, however, are used in a more predictable manner that is commensurate with the decreased safety performance. In the past, the NRC tended to use fines as a prime indicator of agency concern and as a motivator to affect licensee corrective actions. Under the new approach, there is a system of specified agency actions if performance declines. Fines will generally be reserved for such things as discriminating against workers raising safety concerns, or willful misreporting of required information.

The NRC's actions for performance below the "green" level may include meetings with the utility, additional inspections, and required reviews and response by the utility. Further declines in performance would warrant stronger action by the NRC, including a civil order or even the suspension of the utility's operating license.

Violations of NRC requirements

Each violation of NRC requirements found during NRC inspections will be evaluated to determine its effect on plant safety and risk. If the violation is of very low safety significance, it will be discussed in the inspection report with no formal enforcement action. The utility is expected to deal with the violation through its corrective action program, correcting the violation and taking steps to prevent a recurrence. The issue may also be reviewed during future NRC inspections.

If the NRC risk evaluation finds that the violation has higher safety significance, a Notice of Violation will be issued. A Notice of Violation may also be issued if the utility fails to correct a violation of low safety significance in a reasonable period of time or if a violation is found to be willful.

The Notice of Violation requires the utility to respond formally to the NRC with its actions to correct the violation and what steps it will take to prevent the violation from occurring again. The agency will then review the utility's actions in a later inspection.

Normally, these violations will not be the subject of a fine. However, there may be violations that warrant a fine because of their unusual significance. These violations are likely to be uncommon. Possible examples include exceeding a safety limit specified in a reactor license or the inadvertent startup of a reactor.

In addition, some violations will call for the traditional enforcement approach, including the possible issuance of fines. Examples include:

- Discrimination against workers for raising safety issues or other willful violations.
- Actions that may adversely affect the NRC's ability to monitor utility activities, including failure to report required information, failure to obtain NRC approval for plant changes, failure to maintain accurate records, or failure to provide the NRC with complete and accurate information.
- Incidents with actual safety consequences, including radiation exposures above NRC limits, releases of radioactive material above NRC limits, or failure to notify government agencies when emergency response is required.

Making performance information available to the public

The revised oversight process will provide more information on plant performance than in the past, and the information will be available on a more frequent basis. This information will be placed on the NRC's internet web site as well as in its Public Document Room at NRC headquarters.

A utility will submit to the NRC the quarterly performance indicator data for each nuclear power plant it operates. The NRC staff will review the data for completeness and accuracy. The staff will also evaluate inspection findings for that quarter to determine their safety significance. This review uses the agency's "Significance Determination Process," which is keyed to how plant safety systems and procedures contribute to the risk of a potential accident.

The performance indicators and the assessment of inspection findings will be placed on the NRC web site using the color notation of their significance -- green, white, yellow, or red. The statistics and inspection findings which underlie the color notation will also be posted on the web site.

The revised oversight program is intended to fulfill the following four goals established by the Commission:

1. To maintain safety by establishing a regulatory oversight framework that provides assurance that plants continue to be operated safely by plant operators. Maintaining safety is the NRC's overarching mission.
2. To enhance public confidence in the NRC's regulatory program by increasing the predictability, consistency, objectivity and transparency of the oversight process so that all parties will be well served by the changes taking place.
3. To improve the effectiveness, efficiency, and realism of the oversight process by focusing both agency resources and utility resources on those issues with the most safety-significance.
4. To reduce unnecessary regulatory burden as the process becomes more efficient and effective.

Glossary

Baseline Inspection Program - The normal inspection program performed at all nuclear power plants. The program will focus on plant activities that are not adequately measured by performance indicators, on the corrective action program, and on verifying the accuracy of the performance indicators.

Corrective Action Program - The system by which a utility finds and fixes problems at the nuclear plant. It includes a process for evaluating the safety significance of the problems, setting priorities in correcting the problems, and tracking them until they have been corrected.

Cross-cutting Area - Nuclear plant activity that affects most or all safety cornerstones. These include the plant's cornerstone action program, human performance, and "safety-conscious work environment."

Inspection Reports - Reports are issued periodically to document inspection findings. These may cover a specific time period for the baseline inspection or a particular event or problem examined in a reactive inspection. All inspection reports are public documents and, when issued, are posted to the NRC's internet web site.

Performance Indicator - Objective data which records performance in a specific cornerstone of safety at a nuclear power plant.

Reactive Inspection - An inspection to examine the circumstances surrounding an operational problem or event occurring at a nuclear plant.

Regulatory Conference - A meeting between the NRC staff and a utility to discuss potential safety issues or to discuss a change in performance as indicated by a declining performance indicator or inspection finding. These meetings are open to public observation unless they cover security issues, NRC investigation findings, or similar sensitive topics.

Resident Inspector - An NRC inspector assigned to a nuclear plant on a full-time basis. Each site has at least two resident inspectors.

Risk-informed - Incorporating an assessment of safety significance or relative risk in NRC regulatory actions

Cornerstone of Safety - Nuclear plant activities that are essential for the safe operation of the facility. These cornerstones are grouped under the categories of reactor safety, radiation safety, and safeguards.

Safety Conscious Work Environment - A working environment in which employees are encouraged to report safety concerns without fear of criticism or retaliation from their supervisors because they raised the issue.

Significance Determination Process - The process used by the NRC staff to evaluate inspection findings to determine their safety significance. This involves assessing how the inspection findings affect the risk of a nuclear plant accident, either as a cause of the accident or the ability of plant safety systems or personnel to respond to the accident.

[Note: The following section is a "sidebar" -- although it can be incorporated into the rest of the text if appropriate]

How this new oversight process differs from the previous approach

The previous oversight process evolved over a period of time when the nuclear power industry was less mature and there was much less operational experience on which to base rules and regulations. Very conservative judgments governed the rules and regulations. Significant plant operating events occurred with some frequency, therefore the oversight process tended to be reactive and prescriptive, closely observing plant performance for adherence to the regulations and responding to operational problems as they occurred.

But we now have the benefit of four decades of operational experience and, generally speaking, steadily improving plant performance, particularly over the last decade or so. The new program focuses more of the agency's resources on the relatively small number of plants which evidence performance problems. The baseline inspection program is considered the minimum inspection effort needed to assure that plants meet the "safety cornerstone" objectives. The baseline inspection program is performed at all reactor sites by NRC resident inspectors and inspectors from the regional offices.

Plants which do not meet the "safety cornerstone" objectives, measured by performance indicators and inspection findings, will receive increased inspection, focusing on areas of declining performance. There will also be inspections beyond the baseline program, even at plants performing well, if there are operational problems or events the NRC believes require greater scrutiny. Generic problems, affecting some or all plants, may also require additional inspections.

The previous oversight program relied more heavily on fines when violations occurred, while the new program will make broader use of other enforcement tools such as orders and other formal regulatory actions. When fines were imposed previously, they were often issued long after the violations occurred and their impact was substantially less than the cost of repairs or the costs associated with a shutdown to correct the violations. The new process is intended to be more effective in correcting performance or equipment problems because the agency's response will be both more timely and more predictable.

The new assessment program is substantially different from the previous process. It makes greater use of objective performance indicators. Together, the indicators and inspection findings provide the information needed to support reviews of plant performance, to be conducted on a quarterly basis, with the results posted on the NRC's internet site.

The new assessment process also features expanded reviews on a semi-annual basis to include inspection planning and a performance report, all of which will also be posted on the NRC's web site.

The performance assessment process previously involved three processes:

- **Plant Performance Review** - Conducted every six months to assess events, inspection findings, and other data. This review was done to plan future inspections and to identify those plants with declining performance that required further NRC action.

- **Senior management meetings** - Twice a year, NRC senior managers reviewed information assessing plant performance to discuss what regulatory action was needed at plants with declining performance. The managers designated those plants warranting heightened NRC monitoring as being on a "watch list." These "watch list" plants were then discussed at a public meeting with the Commission.
- **Systematic Assessment of Licensee Performance (SALP)** - Every 12 to 24 months, the NRC staff performed a separate review of the performance of each plant, preparing a Systematic Assessment of Licensee Performance report. This report included a numerical rating of the plant in four categories -- plant operations, maintenance, engineering, and plant support -- as well as a narrative discussion of performance in each area.

[Note: The following table is a “sidebar” to be included where the text discusses performance indicators]

Measuring nuclear plant performance

Performance indicators are reported quarterly by operators of nuclear plants, reviewed by the NRC staff, and posted on the NRC’s web site.

Safety Cornerstone	Performance Indicator
Initiating events	Unplanned reactor shutdowns (automatic and manual)
	Loss of normal reactor cooling system following unplanned shutdown
	Unplanned events that result in significant changes in reactor power
Mitigating Systems	Safety System not available <ul style="list-style-type: none"> • Specific Emergency Core Cooling Systems • Emergency Electric Power Systems
	Safety System Failures
Integrity of barriers to release of radioactivity	Fuel Cladding (measured by radioactivity in reactor cooling system)
	Reactor cooling system leak rate
Emergency Preparedness	Emergency response organization drill performance
	Readiness of emergency response organization
	Availability of notification system for area residents

Occupational Radiation Safety	Compliance with regulations for controlling access to radiation areas in plant
	Uncontrolled radiation exposures to workers greater than 10 percent of regulatory limit
Public Radiation Safety	Effluent releases requiring reporting under NRC regulations and license conditions
Physical Protection	Security system equipment availability
	Personnel screening program performance
	Employee fitness-for-duty program effectiveness