

Gary R. Peterson Vice President

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May 25, 2000

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555

Subject: Duke Energy Corporation

Catawba Nuclear Station, Units 1 and 2

Docket Numbers 50-413 and 50-414

Proposed Technical Specifications Amendment Technical Specification 3.5.2, Emergency Core Cooling System, 3.6.6, Containment Spray System, 3.6.17, Containment Valve Injection Water System,

3.7.5, Auxiliary Feedwater System, 3.7.7,

Component Cooling Water System, 3.7.8, Nuclear Service Water System, 3.7.10, Control Room Area Ventilation System, 3.7.12, Auxiliary Building Filtered Ventilation Exhaust System, & 3.8.1, AC

Sources - Operating

Pursuant to 10 CFR 50.90, Duke Energy Corporation is requesting an amendment to the Catawba Nuclear Station Facility Operating License and Technical Specifications This request is for temporary changes to TS 3.5.2, Emergency Core Cooling System (ECCS), 3.6.6, Containment Spray System (CSS), TS 3.6.17, Containment Valve Injection Water System (CVIWS), TS 3.7.5, Auxiliary Feedwater (AFW) System, TS 3.7.7, Component Cooling Water (CCW) System, TS 3.7.8, Nuclear Service Water System (NSWS), TS 3.7.10 Control Room Area Ventilation System (CRAVS), 3.7.12, Auxiliary Building Filtered Ventilation Exhaust System (ABFVES), and 3.8.1 AC Sources - Operating for Catawba Nuclear Station Unit 2. The proposed TS changes will allow the "A" and "B" Nuclear Service Water System (NSWS) headers to be sequentially taken out of service for 12 days each for cleaning and pipe replacement. This cleaning and pipe replacement is scheduled to occur when Unit 1 is in refueling outage 1 EOC 12 and Unit 2 is at power operation.

A001

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The attached justification supports these proposed changes.

The contents of this amendment request package are as follows:

Attachment 1 provides a marked copy of the affected TS pages for Catawba, showing the proposed changes. Attachment 2 contains reprinted pages of the affected TS pages. Attachment 3 provides a description of the proposed changes and technical justification. Pursuant to 10 CFR 50.92, Attachment 4 documents the determination that the amendment contains No Significant Hazards Considerations.

Pursuant to 10 CFR 51.22(c)(9), Attachment 5 provides the basis for the categorical exclusion from performing an Environmental Assessment/Impact Statement. Attachment 6 includes simplified drawings of the NSWS depicting the pipe sections to be cleaned.

Implementation of this amendment to the Catawba Facility Operating License and TS will not impact the Catawba Updated Final Safety Analysis Report (UFSAR).

Catawba Nuclear Station has used probabilistic risk analysis (PRA) to determine the risk associated with taking a loop of NSWS out of service for this project. Catawba Nuclear Station has taken a proactive approach to reducing its level of core damage risk. Based upon investigations performed by PRA personnel, it was determined that the core damage frequency (CDF) would be significantly reduced by the installation of redundant cooling to one train of its Centrifugal Charging Pumps (high head safety injection). This modification has been completed on both units.

Currently additional periodic testing is performed on the NSWS and AFW systems due to system degradation to ensure proper system operation. These tests result in additional unavailability of these systems. The proposed cleaning and modifications will increase available system margin and reduce the system unavailability due to increased system testing. Catawba Nuclear Station strongly believes that the short term increase in risk associated with this temporary TS change is acceptable because of the increased margin gained and reduced system unavailability.

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Duke is requesting NRC review and approval of this proposed amendment by October 1, 2000, so that it may be implemented in conjunction with the Catawba Unit 1 End-of-Cycle 12 Refueling Outage.

This proposed license amendment was modeled after similar license amendments previously granted by the NRC. These amendments were granted for the North Anna Power Station in support of their service water system refurbishment. The NRC granted separate SERs for Amendments Nos. 194 and 172 on October 11, 1995, for Amendments 205 and 186 on July 17, 1997.

In accordance with Duke administrative procedures and the Quality Assurance Program Topical Report, this proposed amendment has been previously reviewed and approved by the Catawba Plant Operations Review Committee and the Duke Corporate Nuclear Safety Review Board.

Pursuant to 10 CFR 50.91, a copy of this proposed amendment is being sent to the appropriate State of South Carolina official.

Inquiries on this matter should be directed to R. D. Hart at (803) 831-3622.

Very truly yours,

Gary R. Peterson

RDH/s

Attachments

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Gary R. Peterson, being duly sworn, states that he is Site Vice President of Duke Energy Corporation; that he is authorized on the part of said corporation to sign and file with the Nuclear Regulatory Commission this amendment to the Catawba Nuclear Station Facility Operating Licenses Numbers NPF-35 and NPF-52 and Technical Specifications; and that all statements and matters set forth herein are true and correct to the best of his knowledge.

Gary R. Peterson, Site Vice President

Subscribed and sworn to me:

5-25-00

Date

Notary Public

My commission expires:

6-26-2002

Data

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xc (with attachments):

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ATTACHMENT 1

MARKED-UP TECHNICAL SPECIFICATIONS PAGES FOR CATAWBA

3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)

3.5.2 ECCS — Operating

LCO 3.5.2

Two ECCS trains shall be OPERABLE.*

APPLICABILITY:

MODES 1, 2, and 3.

In MODE 3, both safety injection (SI) pump flow paths may be isolated by closing the isolation valves for up to 2 hours to perform pressure isolation

valve testing per SR 3.4.14.1.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	One or more trains inoperable.	A.1	Restore train(s) to OPERABLE status.	72 hours ★
	<u>ÁND</u>			
	At least 100% of the ECCS flow equivalent to a single OPERABLE ECCS train available.			
B.	Required Action and associated Completion Time not met.	B.1 <u>AND</u>	Be in MODE 3.	6 hours
		B.2	Be in MODE 4.	12 hours

* Insert A

Insert A

*For each ECCS train on Unit 2, the Completion Time that one ECCS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.6 CONTAINMENT SYSTEMS

3.6.6 Containment Spray System

LCO 3.6.6

Two containment spray trains shall be OPERABLE.*

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One containment spray train inoperable.	A.1	Restore containment spray train to OPERABLE status.	72 hours 🔭
В.	Required Action and associated Completion Time not met.	B.1 <u>AND</u>	Be in MODE 3.	6 hours
		B.2	Be in MODE 5.	84 hours

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.6.6.1	Verify each containment spray manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position.	31 days

(continued)

* Insert B

Insert B

*For each CSS train on Unit 2, the Completion Time that one CSS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.6 CONTAINMENT SYSTEMS

3.6.17 Containment Valve Injection Water System (CVIWS)

LCO 3.6.17

Two CVIWS trains shall be OPERABLE.

APPLICABILITY:

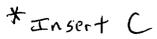
MODES 1, 2, 3, and 4.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One CVIWS train inoperable.	A.1	Restore CVIWS train to OPERABLE status.	7 days 🗡
В.	Required Action and associated Completion Time not met.	B.1	Be in MODE 3.	6 hours
		B.2	Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.6.17.1	Verify system surge tanks pressure is ≥ 36.4 psig.	31 days
SR 3.6.17.2	Verify valve injection flow rate is < 1.29 gpm (Unit 1) < 1.21 gpm (Unit 2) for Train A and < 1.16 gpm for Train B with a surge tank pressure ≥ 36.4 psig.	18 months
SR 3.6.17.3	Verify each automatic valve actuates to its correct position on an actual or simulated actuation signal.	18 months



Insert C

*For each Containment Valve Injection Water System (CVIWS) train on Unit 2, the Completion Time that one CVIWS train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.5 Auxiliary Feedwater (AFW) System

LCO 3.7.5	Three AFW trains shall be OPERABLE.
	Only one AFW train, which includes a motor driven pump, is required to be OPERABLE in MODE 4.

APPLICABILITY:

MODES 1, 2, and 3,

MODE 4 when steam generator is relied upon for heat removal.

ACTIONS

AOTA		T		T
	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	One steam supply to turbine driven AFW pump inoperable.	A.1	Restore steam supply to OPERABLE status.	7 days AND 10 days from discovery of failure to meet the LCO
В.	One AFW train inoperable in MODE 1, 2 or 3 for reasons other than Condition A.	B.1	Restore AFW train to OPERABLE status.	72 hours *AND 10 days from discovery of failure to meet the LCO

(continued)

* Insert D

Insert D

*For each AFW train on Unit 2, the Completion Time that one AFW train can be inoperable as specified by Required Action B.1 may be extended beyond the "72 hours and 10 days from discovery of failure to meet the LCO" up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.7 Component Cooling Water (CCW) System

LCO 3.7.7

Two CCW trains shall be OPERABLE.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One CCW train inoperable.	A.1	Enter applicable Conditions and Required Actions of LCO 3.4.6, "RCS Loops—MODE 4," for residual heat removal loops made inoperable by CCW. Restore CCW train to OPERABLE status.	72 hours 🕇
В.	Required Action and associated Completion Time of Condition A not met.	B.1 <u>AND</u> B.2	Be in MODE 3. Be in MODE 5.	6 hours 36 hours

X Insert E

Insert E

*For each CCW train on Unit 2, the Completion Time that one CCW train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.8 Nuclear Service Water System (NSWS)

LCO 3.7.8

Two NSWS trains shall be OPERABLE.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS

CONDITION		REQUIRED ACTION	COMPLETION TIME
e NSWS train perable.	A.1	residual heat removal loops made inoperable by NSWS	72 hours
		e NSWS train A.1	e NSWS train perable. 1. Enter applicable Conditions and Required Actions of LCO 3.8.1, "AC Sources— Operating," for emergency diesel generator made inoperable by NSWS. 2. Enter applicable Conditions and Required Actions of LCO 3.4.6, "RCS Loops—MODE 4," for residual heat removal loops made inoperable by NSWS.

(continued)

X Insert F

Insert F

*For each NSWS train on Unit 2, the Completion Time that one NSWS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.10 Control Room Area Ventilation System (CRAVS)

LCO 3.7.10

Two CRAVS trains shall be OPERABLE.

APPLICABILITY:

MODES 1, 2, 3, 4, 5, and 6,

During movement of irradiated fuel assemblies,

During CORE ALTERATIONS.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One CRAVS train inoperable.	A.1	Restore CRAVS train to OPERABLE status.	7 days 🗡
В.	Required Action and associated Completion Time of Condition A not met in MODE 1, 2, 3, or 4.	B.1 <u>AND</u> B.2	Be in MODE 3. Be in MODE 5.	6 hours 36 hours
C.	Required Action and associated Completion Time of Condition A not met in MODE 5 or 6, or during movement of irradiated fuel assemblies, or during CORE ALTERATIONS.	C.1	Place in high chlorine protection mode if automatic transfer to high chlorine protection mode is inoperable. Place OPERABLE CRAVS train in operation.	Immediately
				(continued)

* Insert G

Insert G

*For each CRAVS train, the Completion Time that one CRAVS train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.12 Auxiliary Building Filtered Ventilation Exhaust System (ABFVES)

LCO 3.7.12

Two ABFVES trains shall be OPERABLE.*

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	One ABFVES train inoperable.	A.1	Restore ABFVES train to OPERABLE status.	7 days X
B.	Required Action and associated Completion Time not met.	B.1 <u>AND</u>	Be in MODE 3.	6 hours
		B.2	Be in MODE 5.	36 hours
C.	One or more ABFVES train(s) heater inoperable.	C.1	Restore ABFVES train(s) heater to OPERABLE status.	7 days
		<u>OR</u>		
		C.2	Initiate action in accordance with Specification 5.6.6.	7 days

X Insert H

Insert H

*For each ABFVES train on Unit 2, the Completion Time that one ABFVES train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.8 ELECTRICAL POWER SYSTEMS

3.8.1 AC Sources—Operating

LCO 3.8.1

The following AC electrical sources shall be OPERABLE.

- a. Two qualified circuits between the offsite transmission network and the Onsite Essential Auxiliary Power System; and
- b. Two diesel generators (DGs) capable of supplying the Onsite Essential Auxiliary Power Systems;

AND

The automatic load sequencers for Train A and Train B shall be OPERABLE.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTI	ACTIONS					
	CONDITION		REQUIRED ACTION	COMPLETION TIME		
Α.	One offsite circuit inoperable.	A.1	Perform SR 3.8.1.1 for OPERABLE offsite circuit.	1 hour AND		
		AND		Once per 8 hours thereafter		
		A.2	Declare required feature(s) with no offsite power available inoperable when its redundant required feature(s) is inoperable.	24 hours from discovery of no offsite power to one train concurrent with inoperability of redundant required feature(s)		
		<u>AND</u>				
				(continued)		

Insert I

*For each EDG on Unit 2, the Completion Time that one EDG can be inoperable as specified by Required Action B.4 may be extended beyond the "72 hours and 6 days from discovery of failure to meet the LCO" up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
В.	(continued)	B.4	Restore DG to OPERABLE status.	72 hours ** AND 6 days from discovery of failure to meet LCO
C.	Two offsite circuits inoperable.	C.1	Declare required feature(s) inoperable when its redundant required feature(s) is inoperable.	12 hours from discovery of Condition C concurrent with inoperability of redundant required features
		AND C.2	Restore one offsite circuit to OPERABLE status.	24 hours

(continued)

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ATTACHMENT 2

REPRINTED TECHNICAL SPECIFICATIONS PAGES FOR CATAWBA

3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)

3.5.2 ECCS - Operating

LCO 3.5.2

Two ECCS trains shall be OPERABLE*.

APPLICABILITY:

MODES 1, 2, and 3.

------NOTE------

In MODE 3, both safety injection (SI) pump flow paths may be isolated by closing the isolation valves for up to 2 hours to perform pressure isolation

valve testing per SR 3.4.14.1.

	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	One or more trains inoperable.	A.1	Restore train(s) to OPERABLE status.	72 hours*
	AND			
	At least 100% of the ECCS flow equivalent to a single OPERABLE ECCS train available.			
В.	Required Action and associated Completion Time not met.	B.1 <u>AND</u>	Be in MODE 3.	6 hours
		B.2	Be in MODE 4.	12 hours

^{*}For each ECCS train on Unit 2, the Completion Time that one ECCS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.6 CONTAINMENT SYSTEMS

3.6.6 Containment Spray System

LCO 3.6.6

Two containment spray trains shall be OPERABLE*.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One containment spray train inoperable.	A.1	Restore containment spray train to OPERABLE status.	72 hours*
В.	Required Action and associated Completion Time not met.	B.1 AND	Be in MODE 3.	6 hours
		B.2	Be in MODE 5.	84 hours

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.6.6.1	Verify each containment spray manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position.	31 days

(continued)

^{*}For each CSS train on Unit 2, the Completion Time that one CSS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.6 CONTAINMENT SYSTEMS

3.6.17 Containment Valve Injection Water System (CVIWS)

LCO 3.6.17

Two CVIWS trains shall be OPERABLE*.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	One CVIWS train inoperable.	A.1	Restore CVIWS train to OPERABLE status.	7 days*
В.	Required Action and associated Completion Time not met.	B.1 AND	Be in MODE 3.	6 hours
		B.2	Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.17.1 Verify system surge tanks pressure is ≥ 36.4 psig.	31 days
SR 3.6.17.2 Verify valve injection flow rate is < 1.29 gpm (Unit 1) < 1.21 gpm (Unit 2) for Train A and < 1.16 gpm for Train B with a surge tank pressure ≥ 36.4 psig.	18 months
SR 3.6.17.3 Verify each automatic valve actuates to its correct position on an actual or simulated actuation signal.	18 months

^{*}For each CVIWS train on Unit 2, the Completion Time that one CVIWS train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.5 Auxiliary Feedwater (AFW) System

LCO 3.7.5	Three AFW trains shall be OPERABLE*.
	Only one AFW train, which includes a motor driven pump, is required to be OPERABLE in MODE 4.

APPLICABILITY:

MODES 1, 2, and 3,

MODE 4 when steam generator is relied upon for heat removal.

ACTIONS

ACTI	3110			
	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One steam supply to turbine driven AFW pump inoperable.	A.1	Restore steam supply to OPERABLE status.	7 days AND 10 days from discovery of failure to meet the LCO
B.	One AFW train inoperable in MODE 1, 2 or 3 for reasons other than Condition A.	B.1	Restore AFW train to OPERABLE status.	72 hours* AND 10 days* from discovery of failure to meet the LCO

(continued)

^{*}For each AFW train on Unit 2, the Completion Time that one AFW train can be inoperable as specified by Required Action A.1 may be extended beyond the "72 hours and 10 days from discovery of failure to meet the LCO" up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.7 Component Cooling Water (CCW) System

LCO 3.7.7

Two CCW trains shall be OPERABLE*.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIV	JNO			
	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One CCW train inoperable.	A.1	Enter applicable Conditions and Required Actions of LCO 3.4.6, "RCS Loops—MODE 4," for residual heat removal loops made inoperable by CCW	72 hours*
 В.	Required Action and	B.1	Be in MODE 3.	6 hours
υ.	associated Completion Time of Condition A not met.	AND		
	mot.	B.2	Be in MODE 5.	36 hours

^{*}For each CCW train on Unit 2, the Completion Time that one CCW train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.8 Nuclear Service Water System (NSWS)

LCO 3.7.8

Two NSWS trains shall be OPERABLE*.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS

ACTI	JNS	т			
CONDITION			REQUIRED ACTION		COMPLETION TIME
A.	One NSWS train inoperable.	A.1	1. Enter applicate Conditions an Required Acti LCO 3.8.1, "A Sources—Operating," for emergency di generator mainoperable by 2. Enter applicate Conditions an Required Acti LCO 3.4.6, "F Loops—MOD residual heat loops made inoperable by Restore NSWS trait OPERABLE status.	ole ons of or esel de NSWS. ole ons of RCS oE 4," for removal	72 hours*

(continued)

^{*}For each NSWS train on Unit 2, the Completion Time that one NSWS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.10 Control Room Area Ventilation System (CRAVS)

LCO 3.7.10

Two CRAVS trains shall be OPERABLE*.

APPLICABILITY:

MODES 1, 2, 3, 4, 5, and 6,

During movement of irradiated fuel assemblies,

During CORE ALTERATIONS.

	<u> </u>			
	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α	One CRAVS train inoperable.	A.1	Restore CRAVS train to OPERABLE status.	7 days*
associated Co Time of Condi	Required Action and associated Completion Time of Condition A not	B.1	Be in MODE 3.	6 hours
	met in MODE 1, 2, 3, or 4.	B.2	Be in MODE 5.	36 hours
C.	Required Action and associated Completion Time of Condition A not met in MODE 5 or 6, or during movement of irradiated fuel assemblies, or during CORE ALTERATIONS.	C.1	Place OPERABLE CRAVS	Immediately
		<u>OR</u>	train in operation.	
				(continued)

^{*}For each CRAVS train, the Completion Time that one CRAVS train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.7.12 Auxiliary Building Filtered Ventilation Exhaust System (ABFVES)

LCO 3.7.12

Two ABFVES trains shall be OPERABLE*.

APPLICABILITY:

MODES 1, 2, 3, and 4.

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One ABFVES train inoperable.	A.1	Restore ABFVES train to OPERABLE status.	7 days*
В.	Required Action and associated Completion Time not met.	B.1 <u>AND</u>	Be in MODE 3.	6 hours
		B.2	Be in MODE 5.	36 hours
C.	One or more ABFVES train(s) heater inoperable.	C.1	Restore ABFVES train(s) heater to OPERABLE status.	7 days
		<u>OR</u>		
		C.2	Initiate action in accordance with Specification 5.6.6.	7 days

^{*}For each ABFVES train on Unit 2, the Completion Time that one ABFVES train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

3.8 ELECTRICAL POWER SYSTEMS

3.8.1 AC Sources—Operating

LCO 3.8.1 The following AC electrical sources shall be OPERABLE*:

- a. Two qualified circuits between the offsite transmission network and the Onsite Essential Auxiliary Power System; and
- b. Two diesel generators (DGs) capable of supplying the Onsite Essential Auxiliary Power Systems;

AND

The automatic load sequencers for Train A and Train B shall be OPERABLE.

APPLICABILITY:

MODES 1, 2, 3, and 4.

ACTIONS						
CONDITION		REQUIRED ACTION		COMPLETION TIME		
Α.	One offsite circuit inoperable.	A.1	Perform SR 3.8.1.1 for OPERABLE offsite circuit.	1 hour		
		AND		Once per 8 hours thereafter		
		A.2	Declare required feature(s) with no offsite power available inoperable when its redundant required feature(s) is inoperable.	24 hours from discovery of no offsite power to one train concurrent with inoperability of redundant required feature(s)		
		AND				
				(continued)		

^{*}For each EDG on Unit 2, the Completion Time that one EDG can be inoperable as specified by Required Action A.1 may be extended beyond the "72 hours and 6 days from failure to meet the LCO" up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
B.	(continued)	B.4	Restore DG to OPERABLE status.	72 hours* AND 6 days* from discovery of failure to meet LCO
C.	Two offsite circuits inoperable.	C.1	Declare required feature(s) inoperable when its redundant required feature(s) is inoperable.	12 hours from discovery of Condition C concurrent with inoperability of redundant required features
		AND C.2	Restore one offsite circuit to OPERABLE status.	24 hours

(continued)

^{*}For each EDG on Unit 2, the Completion Time that one EDG can be inoperable as specified by Required Action A.1 may be extended beyond the "72 hours and 6 days from discovery of failure to meet the LCO" up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with the NSWS piping, valves, and branch lines, necessary repairs and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

ATTACHMENT 3 DESCRIPTION OF PROPOSED CHANGES AND TECHNICAL JUSTIFICATION

Introduction

Pursuant to 10 CFR 50.90, Duke Energy requests temporary changes to Technical Specification (TS) 3.5.2, Emergency Core Cooling System (ECCS) - Operating, 3.6.6, Containment Spray System (CSS), 3.6.17, Containment Valve Injection Water System (CVIWS), 3.7.5, Auxiliary Feedwater (AFW) System, 3.7.7, Component Cooling Water (CCW) System, 3.7.8, Nuclear Service Water System (NSWS), 3.7.10, Control Room Area Ventilation System (CRAVS), 3.7.12, Auxiliary Building Filtered Ventilation Exhaust System (ABFVES), and 3.8.1 AC Sources - Operating for Catawba Nuclear Station Unit 2. proposed TS changes will allow the "A" and "B" Nuclear Service Water System (NSWS) headers to be sequentially taken out of service for 12 days each for cleaning and pipe replacement. This cleaning and pipe replacement is scheduled to occur when Unit 1 is in refueling outage 1EOC12 and Unit 2 is at power operation.

Recent operating history has indicated a declining performance trend for the NSWS. System flow balance testing has been performed which revealed an adverse trend in system pressure and flow coefficient. Inspections were done of available piping in June 1999. Inspections revealed corrosion product buildup on the piping walls. Based on this data, Duke Energy has decided to perform a detailed cleaning of the NSWS piping. A project team was formed to manage this pipe cleaning effort.

The project will include both "A" and "B" redundant trains of the system and will be sequenced independently into the 1EOC12 outage schedule. The cleaning boundary will extend from the discharge of the 1A, 2A, 1B & 2B Pump Discharge Strainers to the inlet of safety related heat exchangers, assured sources, and components in the auxiliary building and diesel rooms. Approximately 8,000 feet of pipe will be cleaned during this project. During this evolution, instrumentation lines, branch connections, inline equipment / components within the boundary of this project will be inspected, cleaned, repaired and or replaced. The basic steps necessary to accomplish this project include system draining / disassembly for access, inspection, installation of temporary cleaning equipment, piping / component cleaning, repairs/replacements, system re-assembly / restoration, flow balance / heat exchanger testing and return to service.

During the NSWS pipe cleaning project, Nuclear Station Modifications (NSMs) will be implemented on both units to

replace piping and move system boundary valves in the branch lines off the NSWS system to the auxiliary feedwater (AFW) system. This modification is being implemented because of the continual degradation of the NSWS to AFW piping. degradation is occurring because of exposure to stagnant raw water in that portion of the piping. Recent testing has determined that the piping has continued its increase in roughness and is continuing to corrode which reduces the flow margin in the piping. The piping is periodically flow tested and currently passes the operability requirements. Continued exposure to raw water will increase the corrosion layer and eventually the piping will fall below operability limits. This piping replacement will return margin to the flow requirements and inhibit future potential for corrosion. Moving the system boundary valves will help to maintain flow margin.

The proposed changes to TS requirements provide operational flexibility needed to perform necessary cleaning, repairs and replacement of NSWS piping.

This project is currently scheduled to be implemented during the Unit 1 refueling outage 1EOC12. The first loop will be taken out of service after Unit 1 has filled the refueling cavity to a level of 23 feet above the reactor vessel flange. In this condition TS 3.9.4 requires only one residual heat removal (RHR) loop operable and in operation. The NSWS project will continue during the time that the core is off loaded into the spent fuel pit. Therefore, the NSWS headers will be removed from service on Unit 1 during the time period when they are not required to be operable. During the time period that one NSWS header is inoperable the opposite NSWS header and support systems will remain operable.

Project Discussion

The purpose of this project is to remove mud, silt, and corrosion byproducts safely from the internal pipe surfaces of the supply side piping of the Nuclear Service Water System. The process used for this cleaning will insure an uninterrupted flow of service water to one train of safety related components and assured sources within the system. This activity is based on recommendations from Engineering and is based on the concern for heat exchanger / component fouling due to the corrosion deposits and debris within the system and reduction in flow.

Additionally, during this cleaning operation, modifications will be made to the four loops of the AFW Supply. These

modifications consist of relocating isolation valves off the main NSWS header(s) so the normally stagnant NSWS to AFW piping can be backfilled with condensate grade water. This minimizes corrosion and fouling concerns associated with raw water and will help maintain the existing margin. The general corrosion rate will be greatly reduced, the pitting associated with raw water corrosion will be eliminated, and the macro fouling will be minimized. In addition, portions of the NSWS to AFW piping will be replaced with a larger diameter pipe to enhance the current flow margin.

The project will include both "A" and "B" redundant NSWS loops and will be sequenced independently into the 1EOC12 outage schedule. The cleaning boundary will extend from the discharge of the 1A, 2A, 1B & 2B Pump Discharge Strainers to the inlet of all safety related heat exchangers, assured sources, and components in the auxiliary building and diesel rooms. Approximately 8,000 feet of pipe will be cleaned during this project. During this evolution instrumentation lines, branch connections, inline equipment / components within the boundary of this project will be inspected, cleaned, repaired and or replaced. The basic steps necessary to accomplish this project include, system draining / disassembly for access, inspection, installation of temporary cleaning equipment, piping / component cleaning, repairs/replacements, system re-assembly / restoration, flow balance / heat exchanger testing and return to service.

The large scope of this maintenance activity requires direct upper management involvement. The Outage Work Activity Risk Management Process (Catawba Nuclear Station (CNS) Site Directive 3.0.23) is the process to be used. This structured approach ensures appropriate level management attention throughout the project. It assures proper review, representation, and planning from appropriate on-site groups prior to execution of work. This process also provides step by step directions for the execution and completion of the project. Under the guidelines of this directive this project is considered a "Critical Maintenance Process" and will follow that format. The controlling document for the project is called the "Critical Maintenance Process Plan". The site will incorporate this "Plan" into the outage schedule.

The primary method for this cleaning will be high-pressure water. Due to piping configuration and access concerns, the supply lines to the Diesel Jacket Water lines will be cleaned using the "Pigging" process, which consists of injecting abrasive objects into the line.

Personnel experienced in cleaning piping systems will be used for this project. These resources will clean the pipe, remove and dispose of the debris. A rotating spin nozzle operating at 8,000-10,000 PSI with a flow rate designed to remove debris will be the primary method used to dislodge the corrosion deposits from the large pipe. Access to the pipe will be gained through the flange openings by removing valves/equipment and manways. Some sections of the system will require access by selectively cutting the piping.

The refueling outage schedule will determine which loop is to be cleaned first. Current schedule is to start with A loop while B loop remains in service. To better manage the cleaning process each loop will be divided into five sections respectively. Each section will be assigned an execution team and a coordinator. The five sections will be scheduled and worked in parallel during the 12 day Required Action time. Attachment 6 contains simplified figures that depict the sections of NSWS piping that are scheduled to be cleaned as a part of this project and the sections of piping from the NSWS to AFW system that are to be replaced. Figure 1 is a general overview of the NSWS with the 5 sections Figures 2 - 6 provide details of the 5 sections Each figure shows both the A and B loop, but to be cleaned. only one loop will be cleaned at a time. Some portions of the NSWS system may be cleaned during a different time period. These portions are those that can be cleaned within the existing TS Required Action time frame. Figures 7 - 10 show the sections of NSWS piping to the AFW system that are to be modified.

Plans are to clean the five sections and install the NSWS to AFW modifications concurrently. Presently it is estimated that this work, including taking the system out of service and draining the affected portions, will take between 7 - 8 Following the cleaning and modifications, 2 days will be required to fill, perform the NSWS / AFW flow test, NSWS heat exchanger DP testing and NSWS flow balance. Therefore, the total time should run from 9-10 days. This project is being carefully scheduled to minimize the outage time. However, this is the first time that Catawba has undergone this type of project for the NSWS. Catawba is requesting a TS extension for 12 days. Additional time is being requested to allow for any unforeseen circumstances that may arise which could lengthen the project.

Description of Proposed Changes

Duke Energy proposes to temporarily change TS 3.5.2, ECCS - Operating, 3.6.6, Containment Spray System, 3.6.17, Containment Valve Injection Water System, 3.7.5, Auxiliary Feedwater (AFW) System, TS 3.7.7, Component Cooling Water System, 3.7.8, Nuclear Service Water System, TS 3.7.10, Control Room Area Ventilation System (CRAVS), TS 3.7.12, Auxiliary Building Filtered Ventilation Exhaust System (ABFVES), and 3.8.1 AC Sources - Operating to allow operation of the NSWS with one train inoperable on both units for one time period of 12 days.

An evaluation of the impact of these proposed temporary TS changes on other safety systems was performed. The effect of modified operation of the ECCS, CSS, CVIWS, NSWS, AFW, CCW, CRAVS, ABFVES, and EDG systems due to the NSWS activities on equipment required by other TS as well as effect of other TS on the operation of the ECCS, CSS, CVIWS, NSWS, AFW, CCW, CRAVS, ABFVES, and EDG systems during the two 12-day periods was evaluated. The proposed temporary TS changes discussed below address the conclusions of this evaluation.

These proposed changes apply to Unit 2 (except for CRAVS) because the NSWS system work is scheduled during a Unit 1 refueling outage when the Unit 1 NSWS system TS requirements are reduced.

The NSWS TS 3.7.8 only requires additional entry into TS 3.8.1 for the associated EDG and TS 3.4.6 for the associated RHR loop made inoperable by the inoperable NSWS train. No other TS are required by TS 3.7.8 to be directly entered. Since the inoperability of NSWS results in the inoperability of it's associated DG, TS that specifically rely on DG operability will have to be entered. The TS bases for CRAVS and ABFVES both state that because these are shared systems loss of normal or emergency power requires entry into the TS LCO for each unit in the modes of applicability. This results in entry into the TS LCO for TS 3.7.10, CRAVS, and TS 3.7.12, ABFVES during the time in the project when a NSWS loop is inoperable.

The CSS relies on NSWS flow through CSS heat exchangers during the recirculation phase of a LOCA. Therefore, during each NSWS loop outage, NSWS flow will be isolated to its respective CSS heat exchanger. In this condition the CSS train with its NSWS supply isolated will be considered inoperable. This results in entry into the TS LCO for TS

3.6.6 for CSS during the time in the project when a NSWS loop is inoperable.

NSWS is the safety related assured source for make up water supply to the CVIWS during a postulated accident. During each NSWS loop outage, NSWS flow will be isolated to its respective CVIWS train. In this condition the CVIWS train with its NSWS supply isolated will be considered inoperable. This results in entry into the TS LCO for TS 3.6.17 for CVIWS during the time in the project when a NSWS loop is inoperable.

During each NSWS loop outage, NSWS flow will be isolated to its respective CCW heat exchanger. During this alignment, Operations will rack out the respective CCW pump motor breakers. Also the loads on the CCW trains will be in a cross tie alignment. In this condition the CCW train with its NSWS supply isolated will be considered inoperable. This results in entry into the TS LCO for TS 3.7.7 for CCW during the time in the project when a NSWS loop is inoperable.

During Unit power operations however, the Catawba operating procedures are written to maintain availability of essential heat loads associated with the CCW train made unavailable when the CCW system is in a cross train alignment except for the heat exchangers associated with the RHR and CCW trains.

The Residual Heat Removal Heat Exchanger associated with the inoperable CCW train would not be aligned to the on-line CCW train. The RHR Heat Exchanger isolation valve associated with the inoperable train is secured by closing the valve and opening its breaker. This causes entry into TS 3.5.2, ECCS - Operating for Unit 2 during the time in the project when the NSWS loop is inoperable.

Other systems covered by TS are addressed by TS 3.0.6. TS 3.0.6 states that when a supported system LCO is not met solely due to a support system LCO not being met, the Conditions and Required Actions associated with this supported system are not required to be entered. Only the support system LCO ACTIONS are required to be entered. This is an exception to LCO 3.0.2 for the supported system. In this event, additional evaluations and limitations may be required in accordance with TS 5.5.15, "Safety Function Determination Program (SFDP)." If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists is required to be entered.

The AFW system is an exception to TS 3.0.6 because of the wording in the Bases section for the LCO. The NSWS is the safety-related source of water supply to the AFW system. During the NSWS project, this source will be taken out of service for 12 days. This will affect one motor driven AFW pump. The other motor driven AFW pump and the turbine AFW pump will still have a safety-related source of water supply.

TS 3.5.2, "ECCS - Operating"

The following footnote will be added for the ECCS system to temporarily allow one train of ECCS to be inoperable for 12 days:

*For each ECCS train on Unit 2, the Completion Time that one ECCS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.6.6, "Containment Spray System"

The following footnote will be added for the Containment Spray System to temporarily allow one train of CSS to be inoperable for 12 days:

*For each CSS train on Unit 2, the Completion Time that one CSS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.6.17 "Containment Valve Injection Water System (CVIWS)"

The following footnote will be added for the CVIWS to temporarily allow one train of CVIWS to be inoperable for 12 days:

*For each Containment Valve Injection Water System (CVIWS) train on Unit 2, the Completion Time that one CVIWS train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.7.5 "Auxiliary Feedwater (AFW) System"

The following footnote will be added for the AFW system to temporarily allow one train of AFW to be inoperable for 12 days:

*For each AFW train on Unit 2, the Completion Time that one AFW train can be inoperable as specified by Required Action B.1 may be extended beyond the "72 hours and 10 days from discovery of failure to meet the LCO" up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.7.7 "Component Cooling Water (CCW) System

The following footnote will be added for the CCW system to temporarily allow one train of CCW to be inoperable for 10 days:

*For each CCW train on Unit 2, the Completion Time that one CCW train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours

up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.7.8 "Nuclear Service Water System"

The following footnote will be added for the NSWS to temporarily allow one train of NSWS to be inoperable for 12 days:

*For each NSWS train on Unit 2, the Completion Time that one NSWS train can be inoperable as specified by Required Action A.1 may be extended beyond the 72 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.7.10 "Control Room Area Ventilation System"

The following footnote will be added for the CRAVS to temporarily allow one train of CRAVS to be inoperable for 12 days:

*For each CRAVS train, the Completion Time that one CRAVS train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.7.12 "Auxiliary Building Filtered Ventilation Exhaust System"

The following footnote will be added for the ABFVES to temporarily allow one train of ABFVES to be inoperable for 12 days:

*For each ABFVES train on Unit 2, the Completion Time that one ABFVES train can be inoperable as specified by Required Action A.1 may be extended beyond the 168 hours up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

TS 3.8.1 "AC Sources - Operating

The following footnote will be added for the EDGs to temporarily allow one train of NSWS to be inoperable for 12 days:

*For each EDG on Unit 2, the Completion Time that one EDG can be inoperable as specified by Required Action B.4 may be extended beyond the "72 hours and 6 days from discovery of failure to meet the LCO" up to 288 hours as part of the NSWS system upgrades. System upgrades include maintenance and modification activities associated with cleaning of NSWS piping, valves and branch lines, necessary repairs and/or replacement, valve repair and/or replacement, and replacement of portions of the NSWS piping to the AFW system. Upon completion of the cleaning, upgrades, and system restoration in refueling outage 1 EOC12, this footnote is no longer applicable.

Technical Justification

The NSWS cleaning and repair project and the proposed temporary changes to TS 3.5.2, 3.6.6, 3.6.17, 3.7.5, 3.7.7, 3.7.8, 3.7.10, 3.7.12, and 3.8.1 have been evaluated to assess their impact on the normal operation of the affected systems and to ensure that the design basis of these functions are preserved.

NSWS System

The NSWS provides a heat sink for the removal of process and operating heat from safety related components during a design basis accident. During normal operation and during normal plant shutdowns, the NSWS also provides this function for various safety related and non-safety-related components.

The NSWS consists of two independent loops (designated A and B) of essential equipment, each of which is shared between the two units. Each loop contains two NSWS pumps, each of which is provided backup emergency power from a separate emergency diesel generator (EDG). Each set of two pumps supplies two trains (1A and 2A, or 1B and 2B) of essential equipment through common discharge piping. While the pumps are unit designated (i.e., 1A, 1B, 2A, 2B), all pumps receive automatic start signals from a safety injection or blackout signal from either unit. Therefore, a pump designated to one unit will supply post-accident cooling to equipment in that loop on both units, provided its associated EDG is available. The NSWS also provides a safety-related source of water for the Auxiliary Feedwater (AFW) system.

During this time period the operable NSWS loop will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable loop will still respond as designed during design basis events.

Emergency Core Cooling System

The ECCS consists of three separate subsystems: centrifugal charging (high head), safety injection (SI) (intermediate head), and residual heat removal (RHR) (low head). subsystem consists of two redundant, 100% capacity trains. The ECCS flow paths consist of piping, valves, heat exchangers, and pumps such that water from the RWST can be injected into the RCS following the accidents described in this LCO. The major components of each subsystem are the centrifugal charging pumps, the RHR pumps, heat exchangers, and the SI pumps. Each of the three subsystems consists of two 100% capacity trains that are interconnected and redundant such that either train is capable of supplying 100% of the flow required to mitigate the accident consequences. This interconnecting and redundant subsystem design provides the operators with the ability to utilize components from opposite trains to achieve the required 100% flow to the core.

During the time when a NSWS loop is out of service, the respective ECCS equipment on the CCW train without NSWS cooling will be supplied from the opposite CCW train via a cross train alignment. A calculation has been performed which shows that the CCW train inservice can support the loads during the cross train alignment.

This one time extension of the Completion Time from 72 hours to 288 hours is reasonable, based on the redundant capabilities afforded by the operable train, and the low probability of a DBA occurring during this period.

During this time period the operable ECCS train will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable train will still respond as designed during design basis events.

Containment Spray System

The Containment Spray System provides containment atmosphere cooling to limit post accident pressure and temperature in containment to less than the design values. Reduction of containment pressure and the iodine removal capability of the spray reduce the release of fission product radioactivity from containment to the environment, in the event of a Design Basis Accident (DBA).

The Containment Spray System consists of two separate trains of equal capacity, each capable of meeting the system design basis spray coverage. Each train includes a containment spray pump, one containment spray heat exchanger, spray headers, nozzles, valves, and piping. Each train is powered from a separate Engineered Safety Feature (ESF) bus. The refueling water storage tank (RWST) supplies borated water to the Containment Spray System during the injection phase of operation. In the recirculation mode of operation, containment spray pump suction is transferred from the RWST to the containment recirculation sump(s).

When the CSS suction is from the containment recirculation sump, its associated heat exchanger receives NSWS flow for cooling. During the NSWS system upgrades this flow will not be available. However this does not affect the initial injection flow provided. This one time modification of the operability requirements is reasonable, based on the redundant capabilities afforded by the operable train, and the low probability of a DBA occurring during this period.

During this time period the CSS train will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable train will still respond as designed during design basis events.

Containment Valve Injection Water System

The CVIWS is designed to inject water between the two seating surfaces of double disc gate valves used for Containment isolation. The injection pressure is higher than Containment design peak pressure during a LOCA. This will prevent leakage of the Containment atmosphere through the gate valves, thereby reducing potential offsite dose below the values specified by 10 CFR 100 limits following the postulated accident.

The system consists of two independent, redundant trains; one supplying gate valves that are powered by the A train diesel and the other supplying gate valves powered by the B train diesel. This separation of trains prevents the possibility of both containment isolation valves not sealing due to a single failure.

Each train consists of a surge chamber, which is filled with water and pressurized with nitrogen. One main header exits the chamber and splits into several headers. A solenoid valve is located in the main header before any of the branch headers, which will open after a 60 second, delay on a Phase A isolation signal. Each of the headers supply injection water to containment isolation valves located in the same general location, and close on the same engineered safety signal. A solenoid valve is located in each header, which supplies seal water to valves closing on a Containment Pressure - High-High signal. These solenoid valves open after a 60 second delay on a Containment Pressure - High-High signal. Since a Phase A isolation signal occurs before a Containment Pressure - High-High signal, the solenoid valve located in the main header will already be injecting water to Containment isolation valves closing on a Phase A isolation signal. This leaves an open path to the headers supplying injection water on a Containment Pressure - High-High signal. The delay for the solenoid valves opening is to allow adequate time for the slowest gate valve to close, before water is injected into the valve seat.

Makeup water is provided from the Demineralized Water Storage Tank for testing and adding water to the surge chamber during normal plant operation. Assured water is provided from the essential header of the Nuclear Service Water System (NSWS). This supply is assured for at least 30 days following a postulated accident. If the water level in the surge chamber drops below the low-low level or if the surge chamber nitrogen pressure drops below the low-low pressure after a Phase A isolation signal, a solenoid valve in the supply line from the NSWS will automatically open and remains open, assuring makeup to the CVIWS at a pressure greater than 110% of peak Containment accident pressure.

During the NSWS system upgrades this assured makeup flow would not be available during the time frame that each NSWS loop is out of service. However this does not affect the operation of the system during the initial phase of a postulated accident. This one time modification of the operability requirements is reasonable, based on the redundant capabilities afforded by the operable train, and the low probability of a DBA occurring during this period.

During this time period the operable CVIWS train will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable train will still respond as designed during design basis events.

Auxiliary Feedwater System

The AFW System is configured into three trains. System is considered OPERABLE when the components and flow paths required to provide redundant AFW flow to the steam generators are OPERABLE. This requires that the two motor driven AFW pumps be OPERABLE in two diverse paths, each supplying AFW to separate steam generators. The turbine driven AFW pump is required to be OPERABLE with redundant steam supplies from two main steam lines upstream of the Main Steam Isolation Valves (MSIV), and shall be capable of supplying AFW to any of the steam generators. The piping, valves, instrumentation, and controls in the required flow paths also are required to be OPERABLE. The NSWS assured source of water supply is configured into two trains. turbine driven AFW pump receives NSWS from both trains of NSWS, therefore, the loss of one train of assured source renders only one AFW train inoperable. The remaining NSWS train provides an OPERABLE assured source to the other motor driven pump and the turbine driven pump.

There are several sources of water available to the AFW pumps. The preferred sources are non-safety grade condensate quality, located in the Turbine and Service Buildings. These are called the CSS. The CSS is formed from the Upper Surge Tanks (two 42,500 gallon tanks per

unit) and the Condenser Hotwell (normal operating level of 170,000 gallons). The CSS supplies the AFW requirements during normal system operating modes; but, since the CSS is not safety related, its availability is not assured. The assured source of supply to the AFW pumps is provided by the safety related portion of the Nuclear Service Water System. An additional source of supply is available from the Condenser Circulating Water System for safe shutdown events.

TS 3.7.6 requires the CSS to be operable in modes 1,2,3 and mode 4 when steam generator is relied upon for heat removal. The CSS contains sufficient cooling water to remove decay heat for 2 hours following a reactor trip from 100% Rated Thermal Power (RTP), and then to cool down the reactor coolant system (RCS) to RHR entry conditions, assuming a natural circulation cooldown. In doing this, it retains sufficient water to ensure adequate net positive suction head for the AFW pumps during cooldown, as well as account for any losses from the steam driven AFW pump turbine, or before isolating AFW to a broken line.

For emergency events, when none of the condensate grade sources are available, two redundant and separate trains of nuclear service water are available. The water supplied by the two nuclear service water sources is of lower quality, however, safety considerations override those of steam generator cleanliness.

The Standby Nuclear Service Water Pond serves as the ultimate long-term safety related source of water for the AFW System. The automatic detection and transfer controls of the AFW System will detect and transfer the pump suctions to nuclear service water upon detection of the postulated failures of the condensate supplies.

During this time period the operable AFW trains will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable trains will still respond as designed during design basis events.

Component Cooling Water System

The CCW System provides a heat sink for the removal of process and operating heat from safety related components during a Design Basis Accident (DBA) or transient. During normal operation, the CCW System also provides this function for various nonessential components, as well as the spent fuel storage pool. The CCW System serves as a barrier to the release of radioactive byproducts between potentially

radioactive systems and the Nuclear Service Water System (NSWS), and thus to the environment. The CCW System is arranged as two independent, full capacity cooling loops, and has isolatable non-safety related components. Each safety related train includes two 50% capacity pumps, surge tank, heat exchanger, piping, valves, and instrumentation. Each safety related train is powered from a separate bus.

The CCW trains are independent of each other to the degree that each has separate controls and power supplies and the operation of one does not depend on the other. In the event of a DBA, one CCW train is required to provide the minimum heat removal capability assumed in the safety analysis for the systems to which it supplies cooling water. In this Condition, the remaining operable CCW train is adequate to perform the heat removal function. This one time extension of the Completion Time from 72 hours to 288 hours is reasonable, based on the redundant capabilities afforded by the operable train, and the low probability of a DBA occurring during this period.

During this time period the operable CCW train will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable train will still respond as designed during design basis events.

Control Room Area Ventilation System

The CRAVS provides a protected environment from which operators can control the unit following an uncontrolled release of radioactivity or high chlorine gas. The CRAVS consists of two independent, redundant trains that recirculate and filter the control room area air. train consists of a prefilter, a high efficiency particulate air (HEPA) filter, an activated carbon adsorber section for removal of gaseous activity (principally iodines), and a Ductwork, valves or dampers, and instrumentation also form part of the system, as well as prefilters to remove water droplets from the air stream. A second bank of HEPA filters follows the adsorber section to collect carbon fines and provide backup in case of failure of the main HEPA The CRAVS is shared between the two units. system must be operable for each unit when that unit is in the mode of applicability. Additionally, both normal and emergency power must also be operable because the system is If a CRAVS component becomes inoperable, or normal or emergency power to a CRAVS component becomes inoperable, then the Required Actions of this LCO must be entered

independently for each unit that is in the mode of applicability of the LCO.

During the NSWS system upgrades, a train of NSWS will be inoperable for 12 days. This results in the DGs on both units associated with the NSWS train being declared inoperable. Therefore the associated CRAVS train will also be inoperable during the 12-day period. TS LCO 3.7.10 requires two CRAVS trains operable in modes 1,2,3,4,5, and 6, during movement of irradiated assemblies, and during core alterations. Condition A for this LCO states that with one CRAVS train inoperable, the CRAVS train must be restored to operable status within 7 days. This one time request is to extend the time frame from 7 days to 12 days.

During this time period the operable CRAVS train will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable train will still respond as designed during design basis events.

Auxiliary Building Filtered Ventilation Exhaust System

The ABFVES normally filters air exhausted from potentially contaminated areas of the auxiliary building, which includes the Emergency Core Cooling System (ECCS) area and non-safety portions of the auxiliary building. The ABFVES, in conjunction with other normally operating systems, also provides ventilation for these areas of the auxiliary The ABFVES consists of two independent and redundant trains. Each train consists of a heater demister section and a filter unit section. The heater demister section consists of a prefilter/moisture separator (to remove entrained water droplets and to prevent excessive loading of the carbon adsorber) and an electric heater (to reduce the relative humidity of air entering the filter The filter unit section consists of a prefilter, an upstream HEPA filter, an activated carbon adsorber (for the removal of gaseous activity, principally iodines), a downstream HEPA, and a fan.

Upon receipt of the actuating Engineered Safety Feature Actuation System signal(s), the ABFVES exhausts air from the ECCS pump rooms while remaining portions of the system are isolated. This exhaust air goes through the pump room heater demister. The pump room heater demister removes both large particles within the air and entrained water droplets present in the air. The heater demister also preheats air and reduces the relative humidity of the air prior to entry into the filter unit. The pump room heater demister

prevents excessive loading of the HEPA filters and carbon adsorbers within the filter unit.

The ABFVES fans power supply is provided by electrical buses, which are shared between the two units. If normal or emergency power to the ABFVES becomes inoperable, then the Required Actions of this LCO must be entered independently for each unit that is in the mode of applicability of the LCO.

During the NSWS system upgrades, a train of NSWS will be inoperable for 12 days. This results in the DGs on both units associated with the NSWS train being declared inoperable. This results in the electrical bus supplying the associated ABFVES train not being supplied by an operable DG. Therefore the associated ABFVES train will be inoperable during the 12-day period.

During this time period the operable ABFVES train will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable train will still respond as designed during design basis events.

Emergency Diesel Generators

Each train of the 4.16 kV Essential Auxiliary Power System is provided with a separate and independent emergency diesel generator (EDG) to supply the Class 1E loads required to safely shut down the unit following a design basis accident. Additionally, each EDG is capable of supplying its associated 4.16 kV blackout switchgear through a connection with the 4.16 kV essential switchgear.

Each EDG must be capable of starting, accelerating to rated speed and voltage, and connecting to its respective ESF bus on detection of bus undervoltage. Each EDG must also be capable of accepting required loads within the assumed loading sequence intervals, and continue to operate until offsite power can be restored to the ESF buses. These capabilities are required to be met from a variety of initial conditions.

The Diesel Generator Engine Cooling Water System for each diesel includes a jacket water-intercooler water heat exchanger located within the Diesel Room, which is supplied with cooling water from the Nuclear Service Water System. The Diesel Generator Engine Cooling Water System is designed to maintain the temperature of the diesel generator engine within an optimum operating range during standby and during

full-load operation in order to assure its fast starting and load-accepting capability and to reduce thermal stresses. The system is also designed to supply cooling water to the engine lube oil cooler, the combustion air aftercoolers, and the governor lube oil cooler.

During the NSWS project, the NSWS supply to one EDG will be inoperable. A temporary station modification will be implemented for the Unit 2 EDGs to supply an alternate, non-safety related, source of cooling to the EDG with the inoperable NSWS supply. The EDG will still be considered inoperable, but it will be technically capable of being manually started to perform its intended function.

During this time period the operable EDG will be protected to the extent practical by minimizing any maintenance on the system for either unit. In this configuration, the operable train will still respond as designed during design basis events.

Contingency Measures

The proposed work activities to be performed to accomplish the NSWS project were evaluated. As a result, specific contingency measures were developed to provide added assurance of the safe operation of the facility during the project. Listed below is a summary of the contingency measures that will be implemented as a part of the project. These actions will mainly be applied to Unit 2 and to Unit 1 as necessary unless otherwise specified.

- 1. During the two 12-day periods when operating with only one operable NSWS header, no major maintenance or testing shall be planned on the remaining operable NSWS header. In addition, during the two 12-day periods, no major maintenance or testing shall be planned on the operable equipment that relies upon NSWS as a support system. To the maximum extent practicable, routine tests (e.g. quarterly pump tests) and preventive maintenance work (e.g. motor checks) will be scheduled prior to or following the 12-day periods. Certain tests may have to be performed during the 12-day periods.
- 2. Diesel Generator Jacket Water Heat Exchanger A
 Temporary Station Modification will be installed on Unit
 2 to maintain the technically inoperable EDG capable of
 being manually started while the normal NSWS supply
 piping is being cleaned. This will be accomplished by
 using water from the fire protection system.

- 3. Diesel Generator Starting Air A Temporary Station Modification will be installed on Unit 2 to maintain the cooling water to the diesel generator starting air system aftercoolers while the normal NSWS supply piping is being cleaned. This will be accomplished by using drinking water to supply the aftercooler. This cooling water flow rate is adequate to maintain the non safety-related function of the starting air compressors.
- 4. No major maintenance or testing shall be planned on the operable offsite power sources during the NSWS system upgrades. Switchyard activities will be coordinated to ensure that the operable offsite power supply and main transformer on Unit 2 are protected to the maximum extent practicable.
- 5. Appropriate training will be provided to Operations personnel on this TS change and contingency measures to be implemented during this project.
- 6. During the two 12-day periods, no major maintenance or testing shall be planned on the Standby Shutdown Facility (SSF). To the maximum extent practicable, routine tests and preventive maintenance work for the SSF will be scheduled prior to or following the 12-day periods.
- 7. During the two 12-day periods, no major maintenance or testing shall be planned on the operable trains of ECCS, CSS, CVIWS, AFW, CCW, CRAVS, ABFVES, and EDG. To the maximum extent practicable, routine tests and preventive maintenance work for these systems will be scheduled prior to or following the 12-day periods.
- 8. During the two 12-day periods that a NSWS header is out of service, the operable trains remaining in service will be considered protected trains. Operations will increase their routine monitoring of these trains to help ensure their operability.
- 9. Plant procedures will be used to cross tie selected CCW system loads during the time period a CCW heat exchanger will be out of service during the NSWS cleaning project.

Additional Plant Systems

A separate plant subsystem has been incorporated into the Catawba design to allow a means of limited plant shutdown, independent from the control room and auxiliary shutdown panels. This system, known as the Standby Shutdown System, provides an alternate means to achieve and maintain a hot

shutdown condition following postulated fire and sabotage events. This system is in addition to the normal shutdown capabilities available. The Standby Shutdown System (except for interfaces to existing safety-related systems) is designed in accordance with accepted fire protection and security requirements and is not designed as a safety related system. The Standby Shutdown System utilizes the turbine driven AFW pump to provide adequate secondary side makeup independent from all AC power and normal sources of During this mode of operation, the turbine driven AFW pump operates remotely controlled from the Standby Shutdown Facility (SSF). If the turbine has not started automatically prior to the event, it may be started manually and receive suction water from condensate sources. condensate sources are depleted or lost, the turbine will automatically transfer suction to an independent source initiated by the SSF related train of the condensate source loss detection logic and battery-powered motor-operated valves. The independent source of water is the buried piping of the Condenser Circulating Water System, which contains sufficient water in the embedded pipe to maintain the plant at hot standby for at least 3 days. In this manner, sufficient AFW flow may be maintained even if all normal and emergency AC power is lost, and all condensate and safety-grade water sources are lost.

In order to improve the total core damage frequency, backup cooling was provided to Centrifugal Charging Pump (CCP) 1A (2A). The backup cooling water to CCP 1A (2A) is supplied by a non-safety related four-inch drinking water system header in the Auxiliary Building. The drinking water system supply ties into the Component Cooling Water (CCW) System Supply piping to the CCP 1A Motor Coolers and Pump Bearing and Speed Reducer Oil Coolers. On the CCW System return side of these coolers, drain lines are routed from the return lines to the containment spray/residual heat removal sump in the Auxiliary Building. The backup cooling water can be aligned to either the 1A or 2A CCP but not to both pumps at the same The backup cooling supplied by the drinking water system is not safety-related and is not relied upon to mitigate any design basis accidents or events. Operability of the "A" CCPs is not dependent on the backup cooling.

Probabilistic Risk Analysis

Catawba Nuclear Station has used probabilistic risk analysis (PRA) to determine the risk associated with taking a loop of NSWS out of service for up to 12 days (9 days beyond its current TS limit of 72 hrs).

It should be noted that Catawba Nuclear Station has taken a proactive approach to reducing its level of core damage risk. Based upon investigations performed by its PRA personnel, it was determined that the core damage frequency (CDF) could be significantly reduced by the installation of redundant cooling to one train of its CCP pumps. Using the current Catawba PRA model, it was determined that such a modification would reduce the annual CDF (excluding seismic) from 6.4 E-05 to approximately 3 E-05. This modification has been installed on the 'A' train CCP for both units. The modification will allow only one of the "A" CCP to receive the backup cooling at a time.

The PRA models were also used to perform the risk evaluation for taking a train of NSWS out of service beyond its TS limit. It was determined that taking a train of NSWS out of service increases the CDF (excluding seismic) from approximately 3 E-05 to roughly 3.8E-04 (assuming no other risk significant maintenance occurs on the operating unit while the NSWS train is unavailable). This converts to an hourly increase of about 4.5 E-08 / hr. Thus, for a 9-day outage extension, the associated increase would be 9.6 E-06. Therefore, two 9-day NSWS train outage extensions will increase one unit's annual non-seismic CDF by 2 x 9.6 E-06, or 1.9 E-05.

The impact to the seismic CDF was also considered. The Catawba PRA seismic CDF is approximately 8.5 E-06 / yr. Taking a train of NSWS out of service increases the seismic CDF to approximately 1 E-05 / year, an increase of 1.5 E-06 on a yearly basis. The increase on a daily basis is small, and the accumulated increase over the allowed outage extension time period is negligible compared to the increase as a result of the non-seismic initiators.

For the year in which the NSWS outage occurs, the CDF of the affected unit is projected to be approximately 5.9 E-05 / year. This is lower than the 7.2 E-05 CDF estimate for the base case PRA calculation of the unmodified plant. Although the plant configuration associated with the NSWS pipe cleaning will result in a temporary risk increase, the annual core damage probability for the one-year period following implementation of the redundant cooling modification should be a decrease of approximately 1.5 E-05.

It is recognized that when taking a NSWS loop out of service consideration will be given to the protection of the available EDGs, component cooling system, SSF, and the remaining NSWS train during this time. Also, the risk analysis assumes the redundant cooling to the "A" CCP pump

is available; thus, no maintenance activities involving this pump are to be performed during the NSWS outage windows.

The Large Early Release Frequency (LERF) for Catawba is dominated by the inter-facing systems LOCA (ISLOCA) and some seismic events which result in a large containment isolation failure. The other internal events do not contribute significantly to the LERF. The requested NSWS outage extension does not create any core damage sequences not currently evaluated by the existing PRA model. frequency of some previously analyzed sequences do increase due to the longer maintenance unavailability of a NSWS loop. Sequences involving containment isolation or containment bypass (potential LERF contributors) have been evaluated to be 1 E-10 in the base case PRA. No sequences involving a loss of NSWS were found to contribute to the LERF. Sequences that were originally evaluated to be less than 1E-10 in the base case PRA are unlikely to increase sufficiently as a result of the NSWS outage extension to significantly change the Catawba LERF which was evaluated to be approximately 4.3 E-07/yr. It is concluded that the LERF implications of the proposed NSWS outage extension are insignificant.

Catawba has taken a proactive approach towards proper risk management as demonstrated by the implementation of the modification to provide backup cooling to the "A" CCP pump. The risk increase associated with the proposed NSWS outage extension should be considered in combination with the risk reductions already achieved at Catawba. The core damage frequency contribution from the proposed outage extension is judged to be acceptable for a one-time, or rare, evolution. Considering the change in CDF associated with the outage extension in the framework of an average over a five-year period, the average annual contribution is in the range of 3.8 E-06, a low-to-moderate increase in the CDF for consideration of permanent changes to the licensing basis.

Precedent Licensing Actions

This proposed license amendment was modeled after similar license amendments previously granted by the NRC. These amendments were granted for the North Anna Power Station in support of their service water system refurbishment. The NRC granted separate SERs for Amendments Nos. 194 and 172 on October 11, 1995, for Amendments 205 and 186 on July 17, 1997.

ATTACHMENT 4

NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

No Significant Hazards Consideration Determination

The following discussion is a summary of the evaluation of the changes contained in this proposed amendment against the 10 CFR 50.92(c) requirements to demonstrate that all three standards are satisfied. A no significant hazards consideration is indicated if operation of the facility in accordance with the proposed amendment would not:

- 1. Involve a significant increase in the probability or consequences of an accident previously evaluated, or
- 2. Create the possibility of a new or different kind of accident from any accident previously evaluated, or
- 3. Involve a significant reduction in a margin of safety.

First Standard

The increased NSWS train unavailability that results from the implementation of this amendment does involve a one time increase in the probability or consequences of an accident previously evaluated. Because this is a temporary and not a permanent change, the time averaged risk increase is acceptable. The consequences of the accident remain unaffected. The proposed temporary changes to the ECCS, CSS, CVIWS, NSWS, AFW, CCW, CRAVS, ABFVES and EDG TS have been evaluated to assess their impact on the normal operation of the systems and ensure that the design basis safety functions are preserved.

The NSWS system is designed such that one NSWS loop containing two operable NSWS pumps has sufficient capacity to supply post LOCA loads on one unit and shutdown and cooldown loads on the other unit. Additionally, one NSWS loop containing one operable NSWS pump has sufficient capacity to maintain one unit indefinitely in mode 5 (commencing 36 hours following a trip from rated thermal power) while supplying the post LOCA loads on the other Thus, after a unit has been placed in mode 5 only one NSWS pump and its associated EDG are required to be operable on each loop, in order for the system to be capable of performing its intended function, including single failure considerations. During this time period, both units will maintain one NSWS pump and its associated EDG operable. capability of the remaining NSWS train along with the low probability of a design basis accident occurring during this period is such that the extension of the required action time from 72 hours to 288 hours will not have an adverse impact on any accident consequences.

During the NSWS project at least one ECCS train will be operable and capable of performing its intended function. The other ECCS train will have its CCW flow requirements met by a cross train alignment for the respective pumps. In this condition, the remaining operable ECCS train is adequate to perform the DBA heat removal function. The capability of the remaining ECCS train along with the low probability of a design basis accident occurring during this period is such that the extension of the required action time from 72 hours to 288 hours will not have an adverse impact on any accident consequences.

During the NSWS project one CSS train will be fully operable and the other train will be able to provide initial injection flow to containment as designed. This is sufficient to ensure that the UFSAR accident analysis for containment parameters. The capability of the remaining CSS train along with the low probability of a design basis accident occurring during this period is such that the extension of the required action time from 72 hours to 288 hours will not have an adverse impact on any accident consequences.

During the NSWS project at least one CVIWS train will be operable and capable of performing its intended function. A single train is capable of meeting its design function for containment isolation. This is sufficient to ensure that UFSAR accident analyses results remain within applicable limits. The capability of the remaining CVIWS train along with the low probability of a design basis accident occurring during this period is such that the extension of the required action time from 168 hours to 288 hours will not have an adverse impact on any accident consequences.

The AFW system on Unit 2 will have two trains of AFW operable (one motor driven pump and one turbine driven The third train will be capable of starting and supplying feedwater from the CSS. It will not have its safety-related supply from the NSWS. Each of the motor driven pumps supply 100% of the flow requirements to two steam generators, although each pump has the capability to be realigned to feed other steam generators. The turbine driven pump provides 200% of the flow requirements and supplies water to the four steam generators. The diversity of the system and feedwater sources is such that the extension of the required action time from 72 hours to 288 hours will not have an adverse impact on any accident consequences.

During the NSWS project at least one CCW train will be operable and capable of performing its intended function. In this condition, the remaining operable CCW train is adequate to perform the DBA heat removal function. The capability of the remaining CCW train along with the low probability of a design basis accident occurring during this period is such that the extension of the required action time from 72 hours to 288 hours will not have an adverse impact on any accident consequences.

During the NSWS project at least one CRAVS train will be operable and capable of performing its intended function. A single train is capable of pressurizing the control room to greater than or equal to 0.125 inches water gauge. This is sufficient to ensure that UFSAR accident analyses results for the control room operators remain within applicable limits. The capability of the remaining CRAVS train along with the low probability of a design basis accident occurring during this period is such that the extension of the required action time from 168 hours to 288 hours will not have an adverse impact on any accident consequences.

During the NSWS project at least one ABFVES train will be operable and capable of performing its intended function. A single train is capable of meeting its design functions to minimize the release of radioisotopes from the ECCS pump rooms. This is sufficient to ensure that UFSAR accident analyses results remain within applicable limits. The capability of the remaining ABFVES train along with the low probability of a design basis accident occurring during this period is such that the extension of the required action time from 168 hours to 288 hours will not have an adverse impact on any accident consequences.

During the NSWS project at least one EDG per unit will be operable and capable of performing its intended function at all times. The EDG with the inoperable NSWS train will still be capable of being manually started if required. A backup cooling system will be installed to provide the cooling water that is normally supplied by NSWS. In this configuration, the EDG will be available to complete its intended function, even though it will not meet the TS definition of operability. The installation of the backup cooling system will be evaluated per the requirements of 10 CFR 50.59. Therefore, the extension of the required action time from 72 hours to 288 hours will not have an adverse impact on any accident consequences.

An evaluation was performed utilizing PRA for extending the NSWS TS time limit from 72 hours to 288 hours. It was

determined that taking a loop of NSWS out of service increases the CDF (excluding seismic) from approximately 3E-05 / year to roughly 3.8E-04 / year (assuming no other maintenance occurs on the operating unit while the NSWS loop is unavailable). This converts to an hourly increase of about 4.5E-08 / hour. Thus, for a 9-day outage extension, the associated increase would be 9.6E-06. Therefore, two 9-day NSWS loop outage extensions will increase one unit's annual non-seismic CDF by 2 x 9.6E-06, or 1.9E-05 / year. The impact to the seismic CDF was also considered and found to be negligible compared to the increase as a result of the non-seismic initiators. The core damage frequency contribution from the proposed outage extensions is judged to be acceptable for a one-time or rare evolution.

Therefore, there will be no impact on any accident consequences.

Second Standard

Implementation of this amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated. The proposed temporary TS changes do not affect the basic operation of the ECCS, CSS, CVIWS, NSWS, AFW, CCW, CRAVS, ABFVES, or EDG systems. The only change is increasing the required action time frame from 72 hours to 288 hours (ECCS, CSS, NSWS, AFW, CCW, and EDG) or from 168 hours to 288 hours (CVIWS, CRAVS and ABFVES). During the project, contingency measures will be in place to provide additional assurance that the affected systems will be able to complete their design functions.

No new accident causal mechanisms are created as a result of NRC approval of this amendment request. No changes are being made to the plant, which will introduce any new accident causal mechanisms.

Third Standard

Implementation of this amendment would not involve a significant reduction in a margin of safety. Margin of safety is related to the confidence in the ability of the fission product barriers to perform their design functions during and following an accident situation. These barriers include the fuel cladding, the reactor coolant system, and the containment system. The performance of these fission product barriers will not be impacted by implementation of this proposed temporary TS amendment. As stated in item (1) above, the ECCS, CSS, CVIWS, NSWS, AFW, CCW, CRAVS, ABFVES and EDG systems will still be capable of performing their

required functions during the course of this NSWS project. No safety margins will be impacted.

In addition, the probabilistic risk analysis conducted for this proposed amendment demonstrated that there is no appreciable increase in overall plant risk incurred by its implementation.

Based upon the preceding discussion, Duke Energy has concluded that the proposed amendment for a temporary one time TS change does not involve a significant hazards consideration.

ATTACHMENT 5 ENVIRONMENTAL ANALYSIS

Environmental Analysis

Pursuant to 10 CFR 51.22(b), an evaluation of this license amendment request has been performed to determine whether or not it meets the criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) of the regulations.

Implementation of this amendment will have no adverse impact upon the Catawba units; neither will it contribute to any additional quantity or type of effluent being available for adverse environmental impact or personnel exposure.

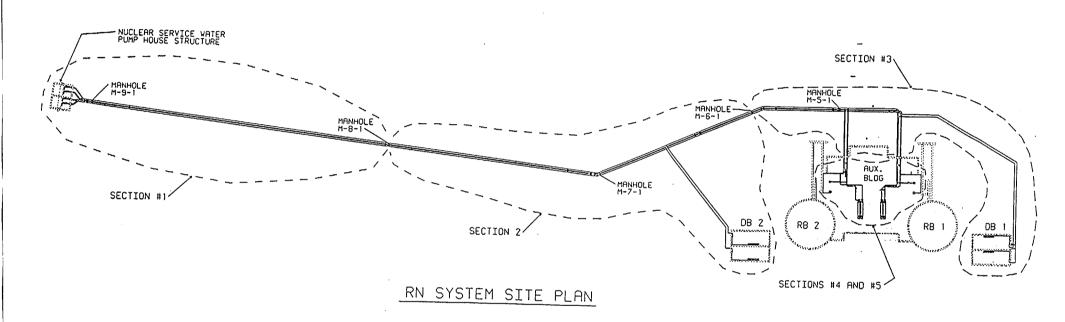
It has been determined there is:

- 1. No significant hazards consideration,
- 2. No significant change in the types, or significant increase in the amounts, of any effluents that may be released offsite, and
- 3. No significant increase in individual or cumulative occupational radiation exposures involved.

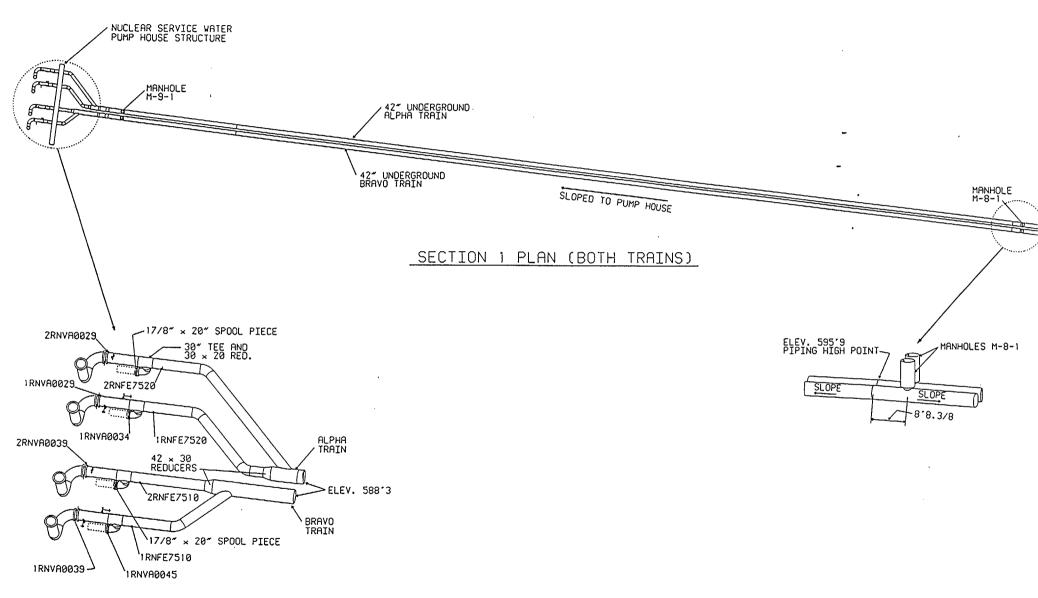
Therefore, this amendment to the Catawba TS meets the criteria of 10 CFR 51.22(c)(9) for categorical exclusion from an environmental impact statement.

ATTACHMENT 6 NSWS CLEANING PROJECT FIGURES

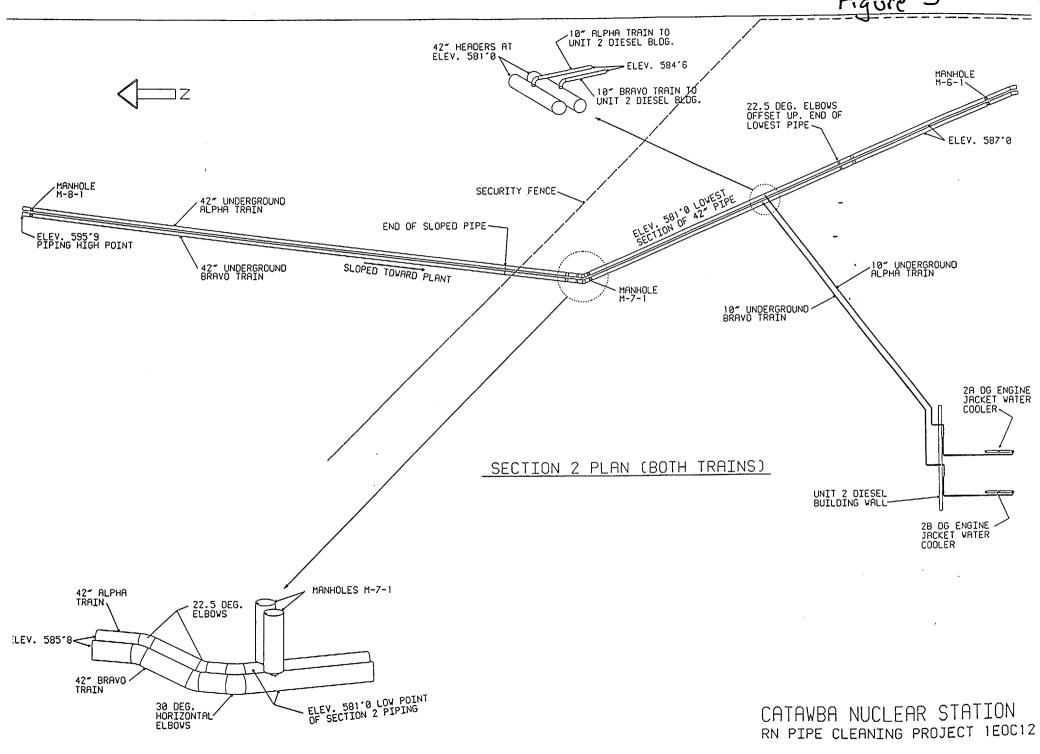








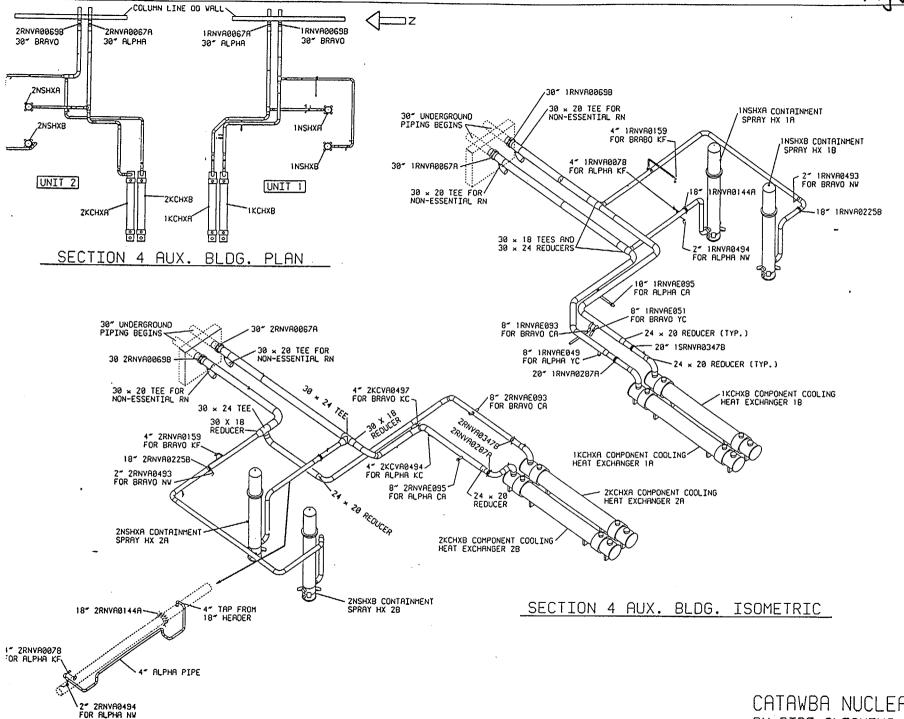
CATAWBA NUCLEAR STATION RN PIPE CLEANING PROJECT 1EOC12



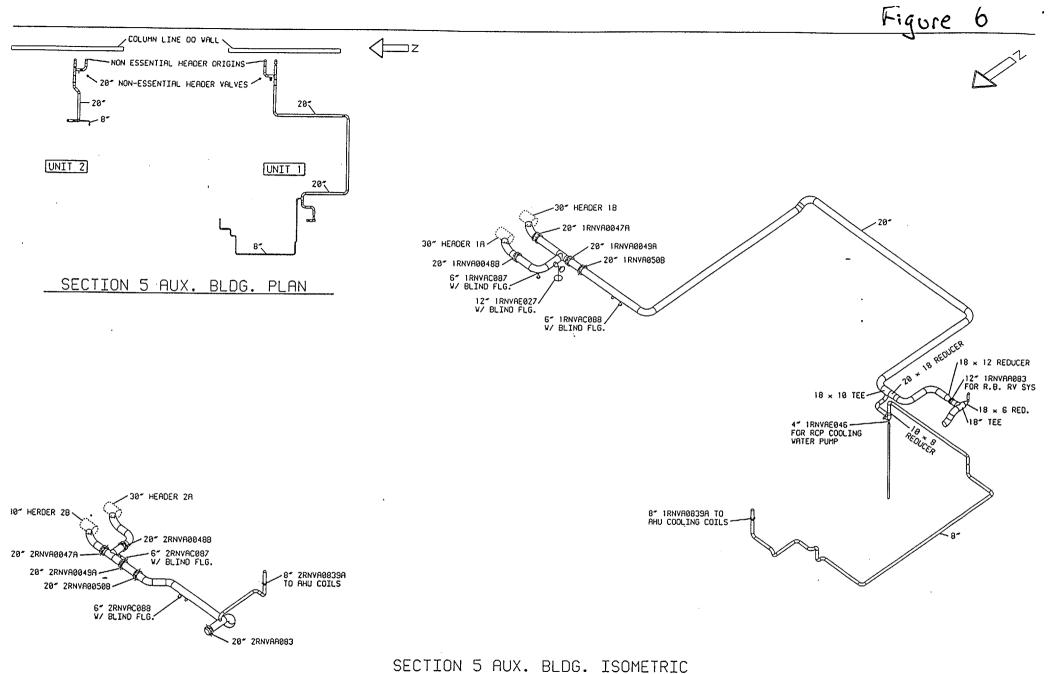
.42" ALPHA MAIN ELEV. 580"3 LOVEST PIPE TO 42 × 30 REO. 22.5 DEG. ELBOW OFFSET UP. END OF-LOWEST PIPE MANHOLE 42° BRAVO MAIN ELEV. 585°9 ELEV. 585'3 42 × 30 REDS ,10° BRAVO TRAIN TO UNIT 1 DIESEL BLDO MANHOLE M-6-1 TO UNIT I AUX. BLDG. 42″ UNDERGROUND BRAVO TRAIN 30" ALPHA TO UNIT 1 AUXILIARY BLDG.-30″ BRAYO TO UNIT I AUXILIARY BLOG. 42″ UNDERGROUND ALPHA TRAIN 10° ALPHA TRAIN TO 30" ALPHA TO UNIT 2 AUXILIARY BLDG. UNIT 1 DIESEL BLOG. 30" BRAVO TO UNIT 2 AUXILIARY BLDG. /10″ UNDERGROUN BRAVO TRAIN AUXILIARY BLDG. WALLS AT COL. LINE 0-0 10" UNDERGROUND ALPHA TRAIN 1RNVA00678 2RNVA00698 `ZRNVA0067A 1RNVA0069B ELEV. 580'3 LOW POINT, ALPHA TRAIN MANHOLES M-5-1-42" ALPHA TRAIN 30° LINES TO AUX. BLDG. UNIT 2 AT ELEV. 586°3 22.5 DEG. ELBOVS BRAVO TRAIN IA DG ENGINE JACKET WATER 42" BRAVO ELEV. 585134 COOLER TRAIN SECTION 3 PLAN (BOTH TRAINS) 18 DG ENGINE JACKET WATER-COOLER

CATAWBA NUCLEAR STATION RN PIPE CLEANING PROJECT 1EOC12

UNIT 1 DIESEL BUILDING VALL

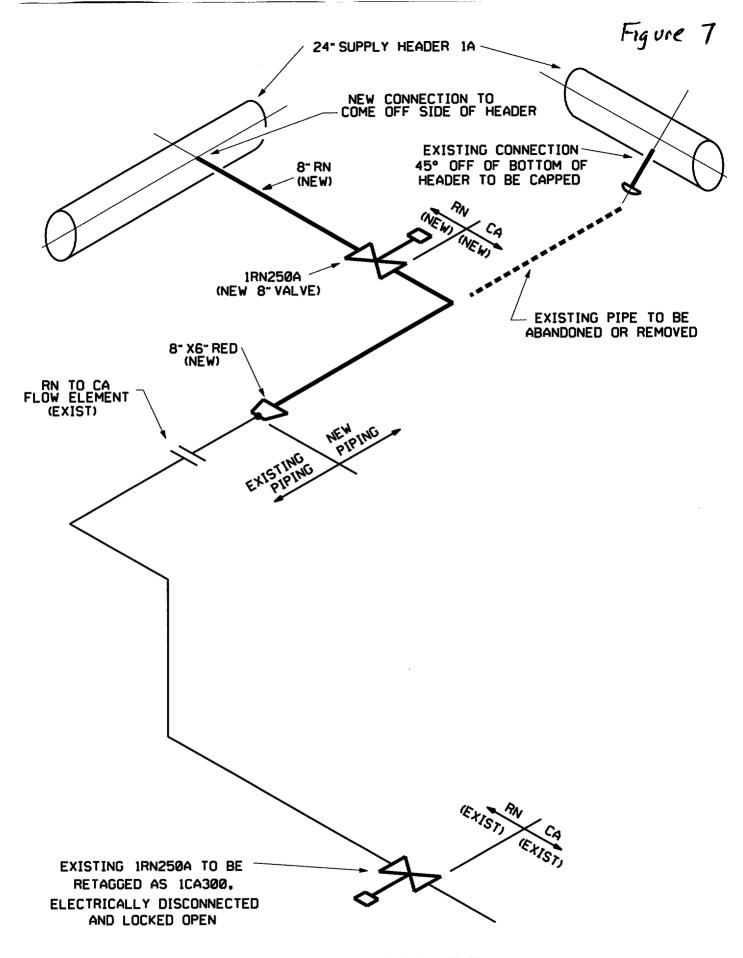


CATAWBA NUCLEAR STATION RN PIPE CLEANING PROJECT 1EOC12

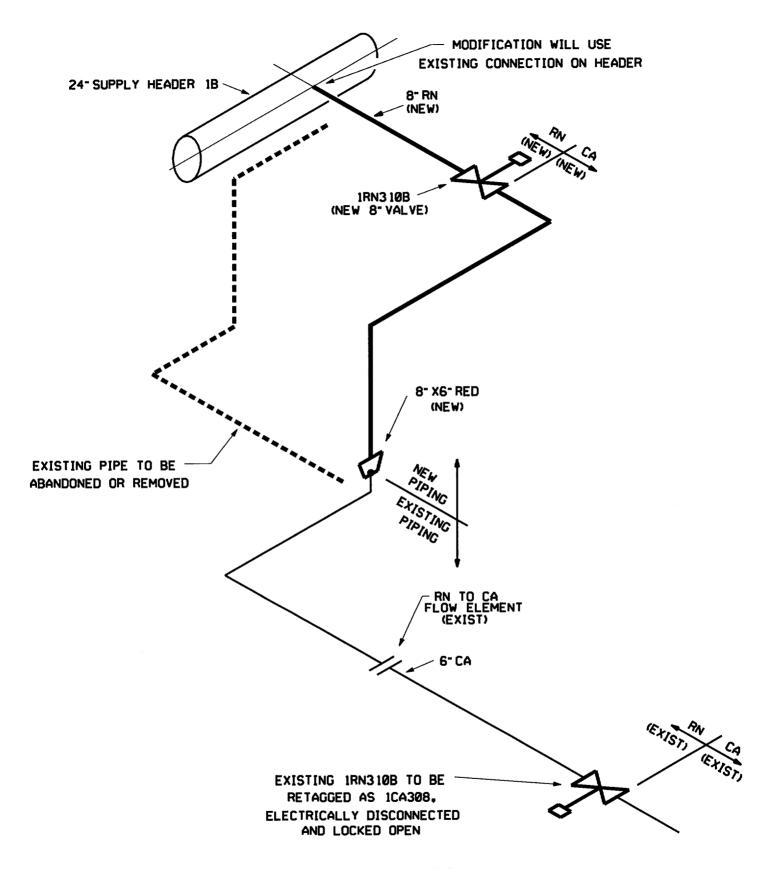


SECTION S HOX. BEDG. ISOMETRIC

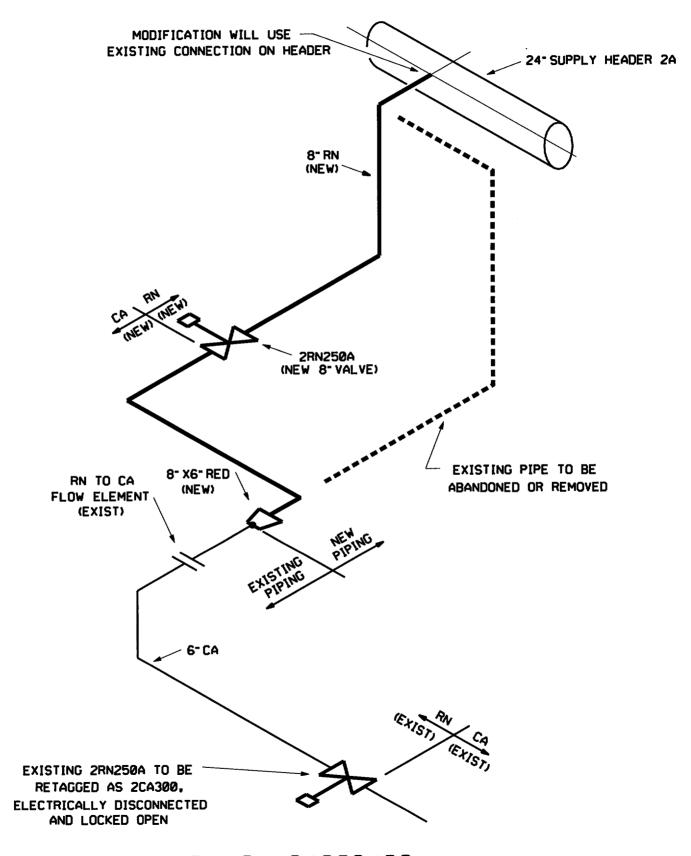
CATAWBA NUCLEAR STATION RN PIPE CLEANING PROJECT 1EOC12



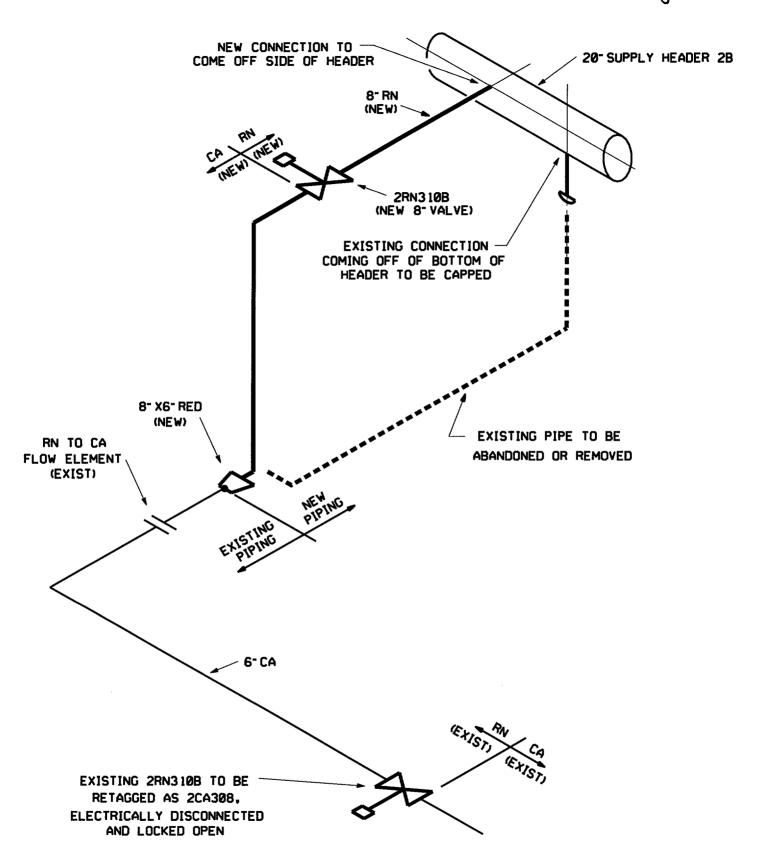
NSM CN-11396/00 RN TO CA MODIFICATION "A" TRAIN



NSM CN-11396/00 RN TO CA MODIFICATION "B" TRAIN



NSM CN-21396/00 RN TO CA MODIFICATION "A" TRAIN



NSM CN-21396/00 RN TO CA MODIFICATION "B" TRAIN