June 2, 2000

Dr. Robert C. Mecredy
Vice President, Ginna Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, New York 14649

SUBJECT: NRC's R. E. GINNA INSPECTION REPORT 05000244/2000-002

Dear Dr. Mecredy:

On May 13, 2000, the NRC completed an inspection of your R. E. Ginna facility. The enclosed report presents the results of that inspection. Preliminary results were discussed with Mr. R. Popp and other members of your staff on May 19, 2000.

NRC resident inspectors examined numerous activities as they related to reactor safety and compliance with the Commission's rules and regulations, and with the conditions of your operating license. The inspection consisted of a selected examination of procedures and records, observations of activities, and interviews with personnel. There were no findings identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be placed in the NRC Public Document Room and will be available on the NRC Public Electronic Reading Room (PERR) link at the NRC home page, http://www/nrc/gov/NRC/ADAMS/index.html.

Sincerely,

/RA/

Michele G. Evans, Chief
Projects Branch 1
Division of Reactor Projects

Docket No. 05000244
License No. DPR-18

Enclosure: Inspection Report 05000244/2000-002

cc w/encl:
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U.S. NUCLEAR REGULATORY COMMISSION

REGION I

Docket No:	05000244
License No:	DPR-18
Report No:	05000244/2000-002
Licensee:	Rochester Gas and Electric Corporation (RG&E)
Facility:	R. E. Ginna Nuclear Power Plant
Location:	1503 Lake Road Ontario, New York 14519
Dates:	April 2, 2000 through May 13, 2000
Inspectors:	H. K. Nieh, Senior Resident Inspector C. R. Welch, Resident Inspector
Approved By:	M. G. Evans, Chief Projects Branch 1 Division of Reactor Projects

Report Details

The report covered a six week period of resident inspection conducted per the NRC's Revised Reactor Oversight Process (Attachment 1).

Summary of Plant Status: The Ginna facility operated at full power throughout the inspection period except for a planned power reduction to 75 percent on the weekend of April 8, 2000, to support offsite electrical distribution system maintenance.

1. REACTOR SAFETY
(Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity)

1R04 Equipment Alignment

a. Inspection Scope

The inspectors performed a partial walkdown of the A emergency diesel generator (EDG) during a B EDG outage. This inspection verified that the redundant A EDG was operable and that necessary support systems, such as service water cooling, were properly aligned to support normal and emergency operation. The inspectors referenced technical specifications and associated plant procedures.

b. Issues and Findings

There were no findings identified.

1R05 Fire Protection

a. Inspection Scope

The inspectors toured the following areas to assess the condition of fire detection and suppression equipment, fire barriers, and the control of combustible materials:

- Control room
- A and B emergency diesel generator rooms
- Auxiliary building, safety injection pump area
- Intermediate building, auxiliary feedwater pump area
- Portions of the turbine building

Procedure T-32.1, "Fire Service Water System Valve and Breaker Position Verification," was used as a reference. The inspectors also reviewed associated fire protection documentation to verify that compensatory measures were used when required.

b. Issues and Findings

There were no findings identified.

1R11 Licensed Operator Requalification

a. Inspection Scope

On April 26, 2000, the inspectors observed a simulator training session for an operating crew to assess operator performance and training effectiveness. The inspectors also verified that the simulator's board configuration matched that of the actual control room.

b. Issues and Findings

There were no findings identified.

1R12 Maintenance Rule Implementation

a. Inspection Scope

The inspectors reviewed the implementation of Ginna's maintenance rule program for the following equipment performance problems:

- Residual heat removal system action report (AR) No. 99-0514
- Auxiliary feedwater system AR Nos. 99-0781 and 99-1070
- Safety injection system AR No. 99-0658
- Diesel generator AR No. 99-1010

This inspection also included a review of RG&E's most recent semi-annual system engineering performance report and quarterly maintenance rule status report.

b. Issues and Findings

There were no findings identified.

1R13 Maintenance Risk Assessments and Emergent Work Control

a. Inspection Scope

The inspectors evaluated RG&E's risk management for a planned C safety injection pump breaker outage and for an emergent A main feedwater regulating valve maintenance activity. This inspection verified that appropriate risk assessments were performed and that proper controls were used for emergent activities. The inspectors held discussions with operations, scheduling, and engineering personnel to determine that RG&E's online risk monitor (EOOS - equipment out of service) was appropriately utilized.

b. Issues and Findings

There were no findings identified.

1R15 Operability Evaluations

a. Inspection Scope

The inspectors reviewed the technical adequacy of the following action reports (AR) and the associated operability evaluations:

- AR 2000-0518 C safety injection (SI) pump leakage
- AR 2000-0529 SI system suction piping weld defects
- AR 2000-0448 Control room envelope boundary

Technical specifications, the updated final safety analysis report, and associated design documents were used as references.

b. Issues and Findings

There were no findings identified.

1R19 Post Maintenance Testing

a. Inspection Scope

Through direct observation and review of results, the inspectors verified that the post-maintenance tests for the following work orders (WO) satisfied applicable requirements:

- WO 20001149 A service water pump high vibration readings
- WO 20001389 C safety injection pump switch replacement
- WO 19902428 A emergency diesel generator lube oil and jacket water cooler modification

b. Issues and Findings

There were no findings identified.

1R22 Surveillance Testing

a. Inspection Scope

The inspectors observed selected portions of, reviewed the results of, and verified the adequacy of the following surveillance and inservice testing activities:

- PT-2.7. A service water pump
- PT-3Q A & B containment spray pumps
- PT-32A Reactor trip breaker testing

Minor deficiencies in the reactor trip breaker testing procedure were identified and reported to RG&E personnel for corrective action.

b. Issues and Findings

There were no findings identified.

4. OTHER ACTIVITIES [OA]

4OA1 Performance Indicator Verification

a. Inspection Scope

The inspectors verified the performance indicators (PI) for the initiating events cornerstone:

- Unplanned scrams per 7,000 critical hours
- Scrams with a loss of normal heat removal
- Unplanned power changes per 7,000 critical hours

Monthly operating reports, Licensee Event Reports, and a sample of NRC inspection reports were reviewed for the prior twelve quarters to determine the accuracy and completeness of the first quarter 2000 PI data.

b. Issues and Findings

There were no findings identified.

4OA6 Management Meetings

a. Exit Meeting Summary

On May 19, 2000, the inspectors presented the inspection results to members of RG&E management led by Mr. R. Popp. RG&E management acknowledged the findings presented. No proprietary information was identified.

b. RG&E/NRC Management Meeting

A public meeting was held on May 9, 2000, at NRC headquarters in Rockville, Maryland to discuss RG&E's calculations associated with the main steam non-return check valves (reference NRC Inspection Report 05000244/1999-005). No members of the public were in attendance.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

J. Widay	VP, Plant Manager
G. Graus	I&C/Electrical Maintenance Manager
J. Hotchkiss	Mechanical Maintenance Manager
G. Joss	Results and Test Supervisor
F. Mis	Principal Health Physicist
J. Pacher	Electrical Systems Engineering Manager
R. Ploof	Secondary Systems Engineering Manager
R. Popp	Production Superintendent
J. Smith	Maintenance Superintendent
W. Thomson	Chemistry & Radiological Protection Manager
T. White	Operations Manager
G. Wrobel	Nuclear Safety & Licensing Manager

ITEMS OPENED, CLOSED, AND DISCUSSED

None

LIST OF ACRONYMS USED

AR	Action report
EDG	Emergency diesel generator
EOOS	Equipment out of service
NRC	Nuclear Regulatory Commission
PI	Performance indicator
PT	Performance test
RG&E	Rochester Gas and Electric Corporation
SI	Safety injection
WO	Work order

ATTACHMENT 1

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety	Radiation Safety	Safeguards
● Initiating Events	● Occupational	● Physical Protection
● Mitigating Systems	● Public	
● Barrier Integrity		
● Emergency Preparedness		

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings indicate issues that, while they may not be desirable, represent very low safety significance. WHITE findings represent issues with low to moderate safety significance, which may require additional NRC inspections. YELLOW findings represent issues with substantial safety significance, which would require the NRC to take additional actions. RED findings represent issues with high safety significance and an unacceptable loss of safety margin, which would result in the NRC taking significant actions that could include ordering the plant shut down.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. The color for an indicator corresponds to levels of performance that may result in increased NRC oversight (WHITE), performance that results in definitive, required action by the NRC (YELLOW), and performance that is unacceptable but still provides adequate protection to public health and safety (RED). GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, as described in the matrix. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.