Wisconsin Electric
A WISCONSIN ENERGY COMPANY

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NPL 2000-0221

May 19, 2000 10 CFR 50.90

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U.S. NUCLEAR REGULATORY COMMISSION
Mail Stop P1-137
Washington, DC 20555

Ladies and Gentlemen:

DOCKETS 50-266 AND 50-301
LICENSE AMENDMENT REQUEST 220
DELETION OF LICENSE CONDITION
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Wisconsin Electric Power Company, licensee, hereby requests amendments to Facility Operating Licenses DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2, respectively. The purpose of the proposed amendments is to eliminate one of the license conditions and associated implementation dates from Appendix C to the licenses. The requested amendments are being submitted in response to the NRC letter dated April 7, 2000.

The NRC issued amendments 174 and 178 to DPR-24 and DPR-27, respectively, on July 9, 1997. Included with these amendments was the addition of Appendix C, "Additional Conditions," to the licenses. Appendix C contains two license conditions based on commitments made by WE in response to issues addressed during the NRC staff's review.

One of the license conditions required WE to submit a license amendment application and supporting radiological dose analyses and evaluations demonstrating compliance with GDC 19 dose limits without reliance on potassium iodide and/or supplied air breathing apparatus; and implementation of any required system modifications within two years of approval of the amendment application and supporting analyses. By letter dated August 6, 1997, the NRC staff provided an additional list of expectations for the analyses. Our control room radiological dose analyses and supporting proposed changes to the Technical Specifications were submitted on February 26, 1998 (TSCR 204).

A001

NRR-057

On July 22, 1999, WE staff met with NRC staff to discuss the proposed amendments and requested that WE be allowed to withdraw the proposed amendments and supporting analyses as being inconsistent with the licensing basis for the Point Beach Nuclear Plant (PBNP). By letter dated April 7, 2000, the NRC staff concurred that the use of KI as a prophylaxis measure to reduce operator dose within regulatory limits was not precluded by the PBNP licensing basis; and, that an assumption of a loss of offsite power, an expectation from the August 6, 1997 letter, was also inconsistent with the PBNP licensing basis. Therefore, WE is hereby requesting that the license condition requiring this analysis, be deleted from the PBNP Operating Licenses. We also understand that upon approval of these requested amendments, we may withdraw our February 26, 1998 request (TSCR 204). Attached are a safety evaluation, determination of no significant hazards, and mark-up and clean license pages indicating the proposed changes.

We have determined that the proposed amendments do not involve a significant hazards consideration, authorize a significant change in the types or total amounts of effluent release, or result in any significant increase in individual or cumulative occupational radiation exposure. Therefore, we conclude that the proposed amendments meet the categorical exclusion requirements of 10 CFR 51.22(c)(9) and that an environmental impact appraisal need not be prepared.

The NRC staff's April 7, 2000, letter also communicated the expectation that WE continue to meet certain commitments made in our June 13, 1997 letter (NPL 97-0351). These commitments included periodic inspections and tests of the Emergency Core Cooling System (ECCS) and control room ventilation system, above and beyond those required by Technical Specifications, to provide additional assurance that control room operator dose remain within GDC 19 dose limits. Our February 26, 1998, request for amendments communicated our plan to discontinue these additional tests and inspections upon approval of the requested amendments based on an overall reduction in the allowable containment leakage from 0.4 to 0.2 weight percent/day (La). This overall reduction in allowed leakage, in conjunction with the Technical Specification required surveillances, provides additional assurance that the regulatory requirements for control room operator dose are met.

WE informed the commission by letter dated February 24, 1999, that the reduced containment leakage limit had been implemented within our Containment Leakage Rate Test Program, thus superceding the commitments in our June 13, 1997, letter. The commitments in our June 13, 1997, letter have been discontinued. The leakage limit is one-half that assumed in our analysis of record and provides adequate assurance that operator dose will remain within GDC 19 dose limits. Therefore, the intent of the original commitments continue to be met.

WE understands its obligation as a licensee to ensure that PBNP is operated in accordance with its design and licensing bases. We reaffirm our commitment, included in the staff's April 7, 2000 letter, to revise and submit radiological dose analyses for the control room and a license amendment proposal as necessary to demonstrate continued conformance to the regulatory requirements and the PBNP licensing basis, in accordance with the specified timeline.

We appreciate the constructive and open dialogue we have had with the staff on this issue. If you have any questions or require additional information, please contact us.

Sincerely,

Mark E. Reddemann
Vice President
Point Beach Nuclear Plant

JG/tat

Subscribed and sworn before me on
this 22nd day of May, 2000.

Christine K. Pozorski
Notary Public, State of Wisconsin

My commission expires 8/25/2002.

cc: NRC Resident Inspector
NRC Regional Administrator
NRC Project Manger
PSCW

SAFETY EVALUATION
DOCKETS 50-266 AND 50-301
LICENSE AMENDMENT REQUEST 220
DELETION OF LICENSE CONDITION
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

On July 22, 1999, WE staff met with NRC staff to discuss the radiological dose analysis and related amendments for the control room submitted on February 26, 1998, as required by the Point Beach Nuclear Plant license. The requested amendments were submitted in compliance with a license condition that required in part, "(1) submitting a license amendment application including supporting analyses and evaluation by February 27, 1998, that contains the proposed methods and for compliance with GDC-19 dose limits under accident conditions based on system design and without reliance on the use of potassium iodide and/or supplied air breathing apparatus." At the meeting, WE staff requested that WE be allowed to withdraw the proposed amendments and supporting analyses, as they were inconsistent with the current and historical licensing basis for the Point Beach Nuclear Plant (PBNP). The submitted analysis was to demonstrate compliance with the dose limits of GDC-19 without the use of potassium iodide (KI) to reduce operator thyroid dose. The analysis is based on more restrictive Technical Specification limits for the control room ventilation system and containment leakage, and requires modifications to the control room ventilation system.

By letter dated April 7, 2000, the NRC staff concurred that the use of KI as a prophylaxis measure to reduce operator dose within regulatory limits as assumed in the radiological analysis of record submitted in support of amendments 174 and 178 to the Unit 1 and Unit 2 licenses, respectively, was not precluded by the historical PBNP licensing basis; and, that an assumption of a loss of offsite power and its effects on ventilation system operation be addressed, an expectation of the February 26, 1998, submittal communicated in an NRC letter dated August 6, 1997, was also inconsistent with the PBNP licensing basis. Therefore, WE is requesting that the license condition requiring this analysis and associated completion dates, be deleted from the PBNP Operating Licenses.

The current licensing basis analysis for radiological consequences for the control room was submitted in support of Amendments 174 and 178 for PBNP Unit 1 and Unit 2, respectively, issued on July 9, 1997. These analyses demonstrated, and the staff concluded, that there was reasonable assurance that the dose limits presented in 10 CFR 100 (offsite) and GDC-19 (control room) would not be exceeded. The staff's confirming evaluation determined that the radiological consequences at the Exclusion Area Boundary and Low Population Zone were within the acceptance criteria presented in SRP 15.6.5, Appendices A and B of NUREG-0800. Radiological doses to the control room operators were within the acceptance criteria of SRP 6.4 of NUREG-0800, based on the control room operators taking KI tablets in the event of a large-break LOCA.

The staff also required additional assurance above that required by the Technical Specification limits on Control Room Ventilation system operation and testing. Commitments providing this additional assurance were documented in a letter dated June 13, 1997. These commitments included: (1) performance of monthly leakage inspections of accessible portions of ECCS systems outside containment that could contain contaminated fluid; (2) inspecting accessible, pressurized ECCS piping outside containment during quarterly inservice testing; (3) performance of the leakage reduction

preventive maintenance program tests for the ECCS during any cold shutdown outage of sufficient duration (about 5 days or longer); (4) performance of corrective action based on the results of these inspections and tests to ensure ECCS leakage remains as low as reasonable achievable; (5) performance of periodic inspection of the control room ventilation system to verify adequacy of material condition; and, (6) increased testing of the control room ventilation system filters to approximately 6-month intervals.

The subsequent analysis submitted on February 26, 1998, assumed a lower overall containment leakage rate of 0.2 weight-percent per day and proposed reducing the Technical Specification leakage rate limit to this value. The submittal also communicated the intent to discontinue the additional tests and inspections above, based on the implementation of this lower containment leakage limit. This lower limit is one-half of the presently allowed Technical Specification leakage limit of 0.4 weight-percent per day and is supported by the most recent containment integrated leakage rate tests performed in 1997. These tests demonstrated overall leak rates of approximated 0.1 weight-percent per day in both units. An administrative leakage rate limit of 0.2 weight percent per day has been implemented as discussed on our letter of February 24, 1999, and the above listed tests and inspections have therefore, been discontinued. This lower leakage provides the additional assurance sought by the staff that the actual doses to the control room operators remain within limits during the limiting design basis event.

As PBNP will continue to operate in accordance with its license, analyses, commitments and Commission regulations following the deletion of this license condition, this proposed change will not be detrimental to the public health and safety.

NO SIGNIFICANT HAZARDS DETERMINATION
DOCKETS 50-266 AND 50-301
LICENSE AMENDMENT REQUEST 220
DELETION OF LICENSE CONDITION
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

On July 22, 1999, WE staff met with NRC staff to discuss the radiological dose analysis and related amendments for the control room submitted on February 26, 1998, as required by the Point Beach Nuclear Plant license. The requested amendments were submitted in compliance with a license condition that required in part, "(1) submitting a license amendment application including supporting analyses and evaluation by February 27, 1998, that contains the proposed methods and for compliance with GDC 19 dose limits under accident conditions based on system design and without reliance on the use of potassium iodide and/or supplied air breathing apparatus." At the meeting, WE staff requested that WE be allowed to withdraw the proposed amendments and supporting analyses, as they were inconsistent with the licensing basis for the Point Beach Nuclear Plant (PBNP).

By letter dated April 7, 2000, the NRC staff concurred that the use of KI as a prophylaxis measure to reduce operator dose within regulatory limits as assumed in the radiological analysis of record, that was submitted in support of amendments 174 and 178 to the Unit 1 and Unit 2 licenses, respectively, was not precluded by the historical PBNP licensing basis; and, that an assumption of a loss of offsite power and its effects on ventilation system operation be addressed, an expectation from the NRC August 6, 1997 letter, was also inconsistent with the PBNP licensing basis. Therefore, WE is requesting that the license condition requiring this analysis and associated completion dates, be deleted from the PBNP Operating Licenses.

In accordance with the requirements of 10 CFR 50.91(a), Wisconsin Electric Power Company (Licensee) has evaluated the proposed changes against the standards of 10 CFR 50.92 and has determined that the operation of Point Beach Nuclear Plant, Units 1 and 2, in accordance with the proposed amendments does not present a significant hazards consideration. The analysis against the requirements of 10 CFR 50.92 and the basis for this conclusion follows:

1. **Operation of the Point Beach Nuclear Plant in accordance with the proposed amendments will not create a significant increase in the probability or consequences of an accident previously evaluated.**

   The license condition that is proposed for deletion is an administrative condition related to analyses to demonstrate conformance to 10 CFR 50, GDC 19 dose limits, and the requirements for design and operation of the control room ventilation system as assumed in the analyses. The license condition proposed for deletion is not related to any factor or event that is an initiator of any accident and thus, deletion will not affect the probability of any accident previously evaluated.

   The dose analyses and the resultant required changes to the control room ventilation system were based in part on making changes to the licensing basis for the control room ventilation system and analyses. These changes were not solely to demonstrate compliance with GDC 19. The existing analysis of record for control room dose demonstrates that regulatory limits are met with the present

design and assumptions. Therefore, deletion of the license condition does not result in a significant increase in the consequences of an accident previously evaluated.

2. **Operation of the Point Beach Nuclear Plant in accordance with the proposed amendments will not create the possibility of a new or different kind of accident from any accident previously evaluated.**

   The license condition imposed administrative requirements for analyses of radiological consequences of presently analyzed events. Deletion of the license condition will not result in a change in the operation of any system as presently assumed. Therefore, no new accident initiators can result. Thus, the deletion of the license condition cannot result in a new or different kind of accident from any accident previously evaluated.

3. **Operation of the Point Beach Nuclear Plant in accordance with the proposed amendments does not create a significant reduction in a margin of safety.**

Deletion of the existing license condition will not result in a change in the way the plant is presently designed and operated. Operation will continue in accordance with presently approved analyses. Therefore, existing approved margins of safety are maintained. Operation in accordance with the proposed amendment does not create a reduction in a margin of safety.

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Attachment 3

OPERATING LICENSES
DPR-24 AND DPR-27
MARK-UPS

APPENDIX C
ADDITIONAL CONDITIONS
OPERATING LICENSE NO. DPR-24

Wisconsin Electric Power Company shall comply with the following conditions and the schedules noted below:

Amendment Number	Additional Conditions	Implementation Date
~~174~~	~~This amendment authorizes changes to Technical Specification requirements for the service water system, component cooling water systems, and control room ventilating system as detailed in an application dated September 30, 1996, as supplemented on November 26 and December 12, 1996, February 13, March 5, April 2, April 16, May 9, June 3, June 13 (two), and June 25, 1997, and evaluated in the staff's safety evaluation attached to this amendment. This amendment is authorized contingent on compliance with commitments provided by the licensee to meet the dose limits associated with Title 10, *Code of Federal Regulations*, Part 50, Appendix A, General Design Criterion (GDC) 19 by: (1) submitting a license amendment application including supporting analyses and evaluations by February 27, 1998, that contains the proposed methods for compliance with GDC 19 dose limits under accident conditions based on system design and without reliance on the use of potassium iodide and/or supplied air breathing apparatus, and (2) implementing the proposed changes within 2 years of the date that NRC approval for the proposed license amendment is granted.~~	~~(1) February 27, 1998~~ ~~(2) Two years from date amendment is approved~~
174	This amendment is authorized contingent on compliance with commitments provided by the licensee to operate Point Beach Nuclear Plant in accordance with its service water system analyses and approved procedures. Specifically, each unit will utilize only one component cooling water heat exchanger until such time as analyses are completed and the service water system reconfigured as necessary to allow operation of one or both units with two heat exchangers in service. If two component cooling water heat exchangers are required in one or both units for maintaining acceptable component cooling water temperature prior to completion of necessary analyses to allow operation in the required configuration, the service water system will be considered in an unanalyzed condition, declared inoperable, and action taken as specified by TS 15.3.0.B except for short periods of time as necessary to effect procedurally controlled changes in system lineups and unit operating conditions.	Immediately

APPENDIX C
ADDITIONAL CONDITIONS
OPERATING LICENSE NO. DPR-27

Wisconsin Electric Power Company shall comply with the following conditions and the schedules noted below:

Amendment Number	Additional Conditions	Implementation Date
~~178~~	~~This amendment authorizes changes to Technical Specification requirements for the service water system, component cooling water systems, and control room ventilating system as detailed in an application dated September 30, 1996, as supplemented on November 26 and December 12, 1996, February 13, March 5, April 2, April 16, May 9, June 3, June 13 (two), and June 25, 1997, and evaluated in the staff's safety evaluation attached to this amendment. This amendment is authorized contingent on compliance with commitments provided by the licensee to meet the dose limits associated with Title 10, *Code of Federal Regulations*, Part 50, Appendix A, General Design Criterion (GDC) 19 by: (1) submitting a license amendment application including supporting analyses and evaluations by February 27, 1998, that contains the proposed methods for compliance with GDC 19 dose limits under accident conditions based on system design and without reliance on the use of potassium iodide and/or supplied air breathing apparatus, and (2) implementing the proposed changes within 2 years of the date that NRC approval for the proposed license amendment is granted.~~	~~(1) February 27, 1998~~ ~~(2) Two years from date amendment is approved~~
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Attachment 4

OPERATING LICENSES
DPR-24 AND DPR-27
REVISED PAGES

APPENDIX C
ADDITIONAL CONDITIONS
OPERATING LICENSE NO. DPR-24

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Amendment Number	Additional Conditions	Implementation Date
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