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May 23, 2000

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555

Subject: Duke Energy Corporation Catawba Nuclear Station, Unit 2 Docket Number 50-414 Request for Relief Number 00-001 Limited Weld Coverage During End-of-Cycle 10 Refueling Outage

Pursuant to 10 CFR 50.55a(g)(5)(iii), please find attached Request for Relief 00-001. This request for relief is associated with limited weld examinations which were performed during the Unit 2 End-of-Cycle 10 Refueling Outage. The affected welds for which complete coverage could not be achieved are the Steam Generator 2B Inlet and Outlet Nozzleto-Safe-End and Safe-End-to-Pipe Welds and the Volume Control Tank Lower Head-to-Shell Weld.

The attachment to this letter contains all technical information necessary in support of this request for relief.

If you have any questions concerning this material, please call L.J. Rudy at (803) 831-3084.

Very truly yours

Gary Ŕ. Peterson

LJR/s

RGN-00

Attachment

AD47

Document Control Desk Page 2 May 23, 2000

xc (with attachment):

L.A. Reyes, Regional Administrator U.S. Nuclear Regulatory Commission, Region II Atlanta Federal Center 61 Forsyth St., SW, Suite 23T85 Atlanta, GA 30303

D.J. Roberts, Senior Resident Inspector U.S. Nuclear Regulatory Commission Catawba Nuclear Station

C.P. Patel, Senior Project Manager (addressee only) U.S. Nuclear Regulatory Commission Mail Stop O13-H3 Washington, D.C. 20555-0001

DUKE ENERGY CORPORATION

STATION: CATAWBA NUCLEAR STATION UNIT 2 10-YEAR INTERVAL REQUEST FOR RELIEF NO. 00-001

I. System/Component(s) for Which Relief is Requested:

ASME Section XI Code Class 1 Examination Category: B-F Pressure Retaining Dissimilar Metal Welds and ASME Section XI Code Class 2 Examination Category: C-A Pressure Retaining Welds in Pressure Vessels.

ID Number	Item Number
2SGB-INLET-SE	B05.070.003
2SGB-OUTLET-SE	B05.070.004
2NC11-02	B05.130.006
2NC11-03	B05.130.007
2VCT-LH-SH	C01.020.010

II. Code Requirement:

ASME Section XI 1989 Edition; Examination Category B-F Pressure Retaining Dissimilar Metal Welds, Table IWB-2500-1, Item Numbers B05.070 and B05.130 and Examination Category C-A Pressure Retaining Welds in Pressure Vessels, Table IWC-2500-1, Item Number C01.020 require a volumetric examination of essentially 100% of the weld volume. Duke Energy Corporation, with NRC approval, has adopted Code Case N-460 which defines "essentially 100%" as greater than 90% coverage.

III. Code Requirement from which Relief is Requested:

Relief is requested for the above identified ID Numbers:

 Class 1 Steam Generator 2B Inlet and Outlet Nozzle-to-Safe-End Welds and Safe-End-to-Pipe Welds from meeting the coverage requirements as defined in ASME Section XI, Appendix III, Paragraph III-4420, 1989 Edition with no addenda. "The examination shall be performed using a sufficiently long examination beam path to provide coverage of the required examination volume in two-beam path directions. The examination shall be performed from two sides of the weld where practical, or from one side of the weld, as a minimum."

Class 2 Volume Control Tank Lower Head-to-Shell Weld from meeting the coverage requirements as defined in ASME Section XI, Appendix III, Paragraph III-4420, 1989 Edition with no addenda. "The examination shall be performed using a sufficiently long examination beam path to provide coverage of the required examination volume in two-beam path directions. The examination shall be performed from two sides of the weld where practical, or from one side of the weld, as a minimum."

IV. Basis for Relief:

During the ultrasonic examination of the 2SGB Inlet and Outlet Nozzle-to-Safe-End and Safe-End-to-Pipe Welds, 2SGB-INLET-SE, 2SGB-OUTLET-SE, 2NC11-02 and 2NC11-03 (Item Numbers B05.070.003, B05.070.004, B05.130.006 and B05.130.007 respectively) shown in Attachments 2, 3, 4 and 5, greater than 90% coverage of the required examination volume could not be obtained. Material characteristics and single sided access caused by component geometry prevents two-beam path direction coverage of the examination volume and limits the examination coverage to 75%. The most effective ultrasonic technique for the examination of dissimilar metal welds and cast stainless steel welds uses refracted longitudinal waves. The longitudinal wave is preferred as the austenitic weld metal and buttering create highly attenuative barriers to shear wave ultrasound. The longitudinal wave is less affected by these difficulties. However, the longitudinal wave is affected by mode conversion when it strikes the inside surface of the safe end or pipe at any angle other than a right angle to the surface.

The calculations below show that a 45° refracted longitudinal wave striking the inside surface of a pipe will produce a 22.9° refracted shear wave in addition to the normally expected 45° reflected longitudinal wave.

 $\operatorname{Sin}^{-1}=(\sin 45^0 \times V_s) \div V_L$

 $= (0.707 \text{ x } 0.123) \div 0.223$

Where; sin⁻¹ is the shear wave angle

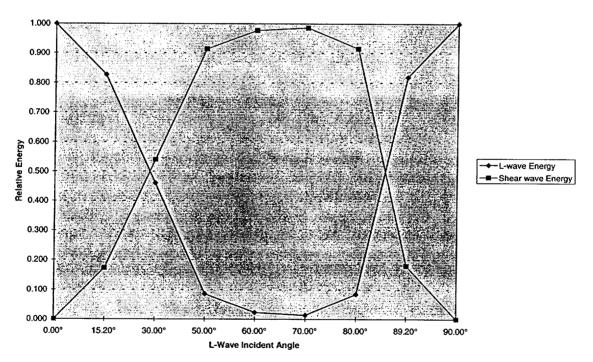
 V_s is the shear wave velocity of the stainless steel safe end/pipe material in inches / msec.

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 V_L is the longitudinal wave velocity of the stainless steel safe/pipe end material in inches/msec.

As shown in the graph below, the mode conversion process creates two sound beams of differing intensities reflecting off the inside surface¹. At incident angles greater than 30 degrees, the shear wave will predominate. However, the shear wave is attenuated and scattered by the austenitic weld metal and the layer of buttering. The examination sensitivity is degraded to such an extent that any examination using the second sound path leg is meaningless. Therefore, the two-beam path direction coverage requirement is impractical.

In order to obtain the required two-beam path direction coverage, welds would have to be re-designed to allow scanning from both sides.



Reflected Sound Beam Energy In Steel on A Free Face

1

¹ Firestone, F.A.: Tricks with the Supersonic Reflectoscope, J. Soc. Nondestructive Testing, vol. 7, no. 2 Fall 1948

During the ultrasonic examination of the Volume Control Tank Lower Head-to-Shell Weld, 2VCT-LH-SH (Item Number C01.020.010) shown in Attachment 6, greater than 90% coverage of the required examination volume could not be obtained. Coverage was therefore limited to 88.34% of the required examination volume. In order to achieve greater than 90% coverage, more access would have to be provided by moving the support legs.

Ultrasonic examination was performed to the extent practical in accordance with ASME Section XI, Appendix III as allowed by Section XI Code Case N-435-1.

V. Alternate Examinations or Testing:

No additional examinations are planned during the current interval for ID Numbers 2SGB-INLET-SE, 2SGB-OUTLET-SE, 2NC11-02, 2NC11-03 and 2VCT-LH-SH. Duke Energy Corporation will continue to use the most current ultrasonic techniques available to obtain maximum coverage for future examinations of these ID Numbers.

VI. Justification for the Granting of Relief:

Steam Generator 2B Inlet and Outlet Nozzle-to-Safe-End and Safe-End-to-Pipe Welds

Although the examination volume requirements as defined in ASME Section XI 1989 Edition with no addenda, Figure IWB-2500-8, Examination Volume C-D-E-F for ID Numbers 2SGB-INLET-SE, 2SGB-OUTLET-SE, 2NC11-02 and 2NC11-03 (Item Numbers B05.070.003, B05.070.004, B05.130.006 and B05.130.007 respectively) could not be met, the amount of coverage obtained for these examinations provides an acceptable level of quality and integrity. For results of the examinations, reference Attachments 2, 3, 4, and 5.

The nozzle to safe-end and safe-end to pipe welds on the Steam Generator Inlet and Outlet Nozzles are located inside containment and are part of the reactor coolant system pressure boundary. General Design Criterion 30, "Quality of Reactor Coolant Pressure Boundary," of Appendix A to 10 CFR Part 50, "General Design Criteria for Nuclear Power Plants," mandates that means be provided for detecting and, to the extent practical, identifying the location of the source of reactor coolant leakage. If a leak were to develop at these weld locations discussed in this relief request, the instrumentation available to the operators for detection and monitoring of leakage would provide a prompt and qualitative information necessary to permit them to take immediate corrective action. If a leak should develop in these aforementioned locations, the only corrective action would be shutdown and depressurize the reactor coolant system, since the welds are non-isolatable.

Plant Technical Specifications dictate that a reactor coolant system water inventory balance be performed on a regular basis. A normal operating practice is to perform this computer based mass balance on a daily frequency and/or whenever the operators suspect any abnormal changes to other leakage detection systems. Plant Technical Specification requires that if the leak rate cannot be reduced below 1 gpm unidentified that the plant be put in hot standby within 6 hours and in cold shutdown within the following 30 hours. Leakage as a result of a failed weld discussed in this section would show up as unidentified leakage and subject to the 1 gpm limit.

Other leakage detection systems available to the operator and dictated per plant technical specifications are:

- Containment Atmosphere Gaseous and Particulate Radioactivity Monitoring System (EMF monitors 38 & 39) which would detect airborne radiological activity;
- Containment Floor and Equipment Sump Level and Flow Monitoring Subsystem where unidentified accumulated water on the containment floor would be monitored and evaluated as sump level changes;
- Containment Ventilation Unit Condensate Drain Tank Level Monitoring Subsystem which collects and measures as unidentified leakage the moisture removed from the containment atmosphere.

Additionally, other indicators are also available to the operator that a leak exists or may be developing:

- Containment Atmosphere Iodine Monitor (EMF 40)
- Charging / Letdown system mismatches;
- Containment humidity indications;
- Pre-Cycle walkdowns performed each outage while system is at operating temperature and pressure prior to criticality;

• Post-Cycle walkdowns performed at operating temperature and pressure performed during unit shutdown.

Volume Control Tank Lower Head-to-Shell Weld

Although the examination volume requirements as defined in ASME Section XI, Appendix III, Paragraph III-4420, 1989 Edition with no addenda, for ID Number 2VCT-LH-SH (Item Number C01.020.010) could not be met, the amount of coverage obtained provides an acceptable level of quality and integrity. For results of the examination, reference Attachment 6.

The Volume Control Tank (VCT) is used in power operations. The VCT is located in the Auxiliary Building adjacent to the unit mechanical penetration room on floor elevation 560 feet. During power operations and unit refueling outages, the VCT is accessible for visual inspections.

If a leak were to occur at the weld in question (lower head to shell weld), there are several periodic tests and evaluations that are performed by established procedures that should identify the leakage for prompt OPS/ENG evaluation:

- During power operation, any leakage from the VCT would be identified as a mass loss in reactor coolant system water inventory balance. As described above, a normal operating practice is to perform this computer based mass balance on a daily frequency and/or whenever the operators suspect any abnormal changes to other leakage detection systems. Plant Technical Specification requires that if the leak rate cannot be reduced below 1 gpm unidentified that the plant be put in hot standby within 6 hours and in cold shutdown within the following 30 hours. Leakage as a result of a failed weld discussed in this section would show up as unidentified leakage and subject to the 1 gpm limit.
- If a leak were to occur at the subject weld, the water would spill on floor in VCT room and flow to floor drain and then to Floor Drain Tank. Our Chemistry department periodically monitors the tank level and evaluates unidentified leakage for correction.
- Weekly visual inspections are made by Operations into the VCT Room per PT/1(2)/A/4150/02 (Visual Inspection of Radioactive Components Outside Containment). Any leaks are required to be reported and evaluated per this Periodic Test.
- Quarterly walkdowns by the System Engineer include a check of the VCT and related components.

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- Periodically, visual material condition inspections in accordance with NSD 104 are made in the VCT room by the site owner of the Aux. Bldg. Elev. 560 area. Identified leakage would be reported for evaluation.
- At a frequency of each refueling outage, visual leakage inspections of the VCT and charging system are made per PT procedure PT/1(2)/A/4202/06, "Leak Rate Determination for NV System." Any NV components identified with external leakage are documented for evaluation, including the VCT.

VII. Implementation Schedule:

These examinations will continue to be scheduled in accordance with the requirements of ASME Section XI for future Inspection Intervals at Catawba Nuclear Station, Unit 2.

The following individuals contributed to the development of this RFR:

Jim McArdle (NDE Level III) provided Sections 3-5

David Goforth (System Engineer) provided Section 6

Andy Hogge and Jimmy Cherry (Sponsors) compiled the remaining sections

Sponsored By:

Date

Approved By:

Khyne Date

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Attachment 1	Description Table
Attachment 2	UT Examination Data B05.070.003
Attachment 3	UT Examination Data B05.070.004
Attachment 4	UT Examination Data B05.130.006
Attachment 5	UT Examination Data B05.130.007

Attachment 6 UT Examination Data C01.020.010

ASME Class 1 & 2 Inservice Inspection Request For Relief No. 00-001 For Catawba Unit 2 Based on ASME Section XI - 1989 Code

Item No.	Exam Category /Figure No.	System Or Component	Area To Be Examined	Reason For Request	Licensee Proposed Alternate Examination
B05.070.003	B-F IWB-2500-8 (c)	Steam Generator	Steam Generator 2B Inlet Nozzle-to- Safe-End Weld	Limited scan due to material characteristics and single-sided access. Actual coverage obtained = 75% (See Attachment 2)	None
B05.070.004	B-F IWB-2500-8 (c)	Steam Generator	Steam Generator 2B Outlet Nozzle-to- Safe-End Weld	Limited scan due to material characteristics and single-sided access. Actual coverage obtained = 75% (See Attachment 3)	None
B05.130.006	B-F IWB-2500-8 (c)	Steam Generator	Steam Generator 2B Inlet Nozzle Safe- End to Pipe Weld	Limited scan due to material characteristics and single-sided access. Actual coverage obtained = 75% (See Attachment 4)	None

Request for Relief Serial No. 00-001 Page 2 of 2 Attachment 1

ASME Class 1 & 2 Inservice Inspection Request For Relief No. 00-001

I	For Catawba Unit 2	Based on ASM	E Section XI -	1989 Code

Item No.	Exam Category /Figure No.	System Or Component	Area To Be Examined	Reason For Request	Licensee Proposed Alternate Examination
B05.130.007	B-F IWB-2500-8 (c)	Steam Generator	Steam Generator 2B Outlet Nozzle Safe- End to Pipe Weld	Limited scan due to material characteristics and single-sided access. Actual coverage obtained = 75% (See Attachment 5)	None
C01.020.010	C-A IWC-2500-1 (a)	Volume Control Tank	Volume Control Tank Lower Head-to- Shell Weld	Limited scan due to proximity of four support legs. Actual coverage obtained = 88.34% (See Attachment 6)	None

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Form 00184 (R4-88)

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3	45	CW	1.66	116	3	192.56	192	.56	100.00
4	45	CCW	1.66	116	6	192.56	192	.56	100.00

(Volume Examined) 577.68 / (Volume Reguired) 770.24 X 100 = 75% Coverage

Item No:	B05.070.004
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Form 00184 (R4-88)

DUKE POWER COMPANY

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DUKE PO	WER CO	OMPA	NY			Exam Sta	art: 10	010	Form	NDE-UT	Г-2А
ULTRASONIC EXAMINATION DA	TA SHE	ET FOI	R PLANAR	REFLEC	TORS	Exam Fir	nish: 10	035	R	evision 4	Ļ
Station: Catawba	Unit: 2	2 Co	omponent/V	Veld ID: 21	NC11-02				Date:	3/16/0	00
		Conditio	n: AS (GROUND	Lo:	9.1.1.1	Surface 7	Fempera	ture:	91°	_ <u>F</u>
Examiner: Marion T. Weaver			Scans:				Pyromete Cal Due:			DE 2720	5
	Level:		45 🛛 <u>63</u>		70		Configura	ation:	Safe E	End to Pip	e
Procedure: NDE-610 Rev: 4	FC:		45T ⊠ <u>63.</u>			dB		2			
Calibration Sheet No:				dB				PIPE Scan	to <u>Surface</u> :		<u>D</u>
0002037, 0002038		6	ют 🗆	dB				pplies t	o NDE-6		
			Other:		d	B	Skew An	gle:		N/A	
IND # 🕂 Max Mp W % Max Max Ref	L Max	L1	L2	W1	Mp1	W2	Mp2	Beam Dir.	Exam Surf.	Scan	Damps
DO NOT WRITE IN THIS SPACE		20%dac HMA 50%dac 100%dac	HMA	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	D IN	O NOT	WRITI SPACE	
NRI 45°										<u> </u>	

imitations: (see NDE-UT-4)	90% or greater	[.] coverage obtai			Sheet of _4_
Reviewed By:	Level:	Date:	Authorized Inspector;	Date:	Item No:
Jun S. Bill	TT	3-18-00	-DOI ATTACHMENT.	3-28.00	B05.130.006

	DUK	E POWER	COMPAN	Y		FORM NDE-UT-4
		LIMITATION				Revision 1
Component/Weld ID: 2NC11-02		1	Item No: B05.13	0.006	Remarks:	
🖾 NO SCAN	SURF	ACE	BEAM D	IRECTION	DUE TO NOZZ	LE CONFIGURATION
LIMITED SCAN	2 1	2	□ 1 ⊠ 2	2 🔲 cw 🔲 ccw		
FROM LN/A to LN/A		INCHES FROM	M WO1.5'	to <u>BEYOND</u>		
ANGLE: 0 🛛 45 🗌 60 🗌 Other			FROM0	DEG to 360 DEG		
	SURF	ACE	BEAM D	IRECTION		
	1	2		2 🔲 cw 🗌 ccw		
FROM L to L		INCHES FROM	n wo	to		
ANGLE: 0 0 45 0 60 0 Other			FROM	DEG toDEG		
	SURF	ACE	BEAM D	IRECTION		
	1	2		2 🔲 cw 🗌 ccw		
FROM L to L		INCHES FROM	4 WO	to		
ANGLE: 0 0 45 0 60 0 Other			FROM	DEG toDEG		
	SURF	ACE		IRECTION		
	□ 1	2		cw 🗆 ccw		
FROM L to L		INCHES FROM	/ WO	to		
ANGLE: 0 0 45 60 0 Other			FROM	DEG to		
Prepared By: Fam Maulli	Lev	/el: <u>777</u> D	ate: 3./6.00	Sketch(s) attached]yes □no	Sheet 2 of 4
Prepared By: Law Mauldu Reviewed By: Sun S. Bill				zed Inspector:	nulil	 Date: ج،ج8-((

A15/00

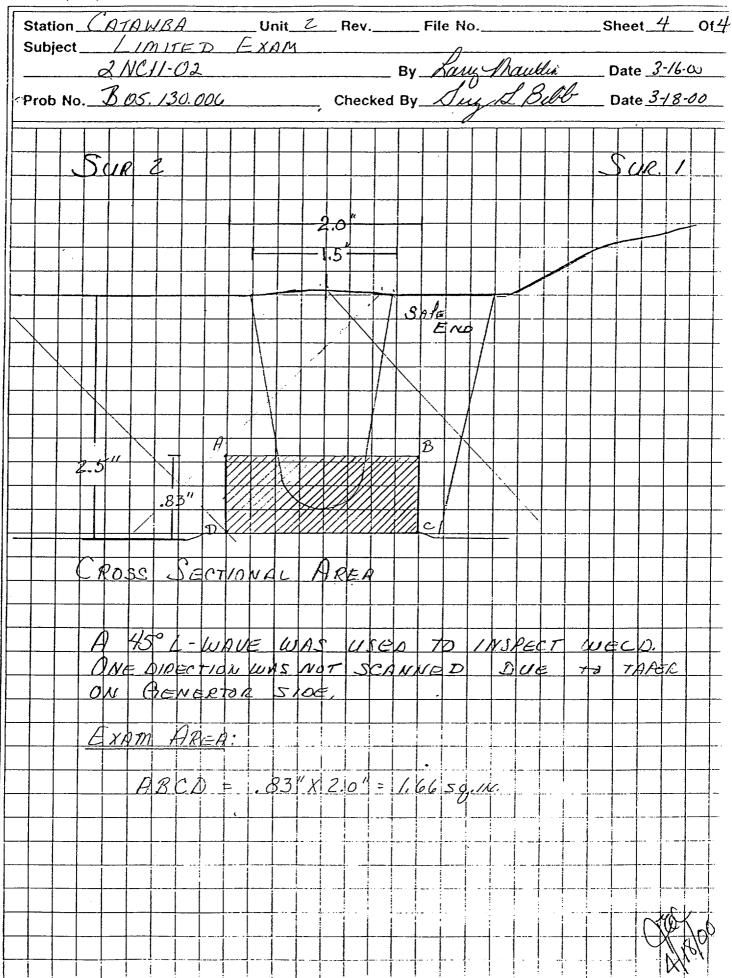
		DUK		COMP	ANY	1			NDE-91-1
		Limited Ex	camination Co	verage \	Nork	sheet			Revision 0
			Examinat	ion Volu	ime/A	rea Defined			
🖾 Bas	se Metal		Weld	🗆 Ne	ar Sui	rface [Bolting	1	Inner Radius
		Area Calc	ulation			Vo	lume Ca	Iculat	ion
.83" X 2	2.0" = 1.6	6 sq. in.			1.66	sq. in. X 116"	= 192.56	cu. in.	
						,			
			Cov	verage (Calcu	lations			
		Beam	Area	Leng	-	Volume	Volu		
Scan #	Angle	Direction	Examined (sq.in.)	Exam (in		Examined (cu.in.)	Requ (cu.		Percent Coverage
1	45	2	0	116	3	0	192	.56	0.00
2	45	1	1.66	116	3	192.56	192.	.56	100.00
3	45	CW	1.66	116	3	192.56	192.	56	100.00
4	45	CCW	1.66	116	3	192.56	192.	56	100.00

(Volume Examined) 577.68 / (Volume Reguired) 770.24 X 100 = 75% Coverage

		Item No:	B05.130.006
Prepared By:	Kary Mauldi	Level: III	Date: 3-/6-01
Reviewed By:	Sun S. Bibb	Level: T	Date: 3-18-00
		3 of 4	C.C.

Form 00184 (R4-88)

DUKE POWER COMPANY



DUKE PO	WER COM	PANY			Exam St	art: 1	036	Form	NDE-UT	Г-2А
ULTRASONIC EXAMINATION DA	TA SHEET F	OR PLANAF		TORS	Exam Fir	nish: 1	059	R	evision 4	•
Station: Catawba	Unit: 2	Component/V	Veld ID: 21	NC11-03				Date:	3/16/0	00
		tion: AS	GROUND	Lo:	9.1.1.1	Surface ⁻	Tempera	iture:	91 °	F
Examiner: Marion T. Weaver Marin V. Jean Examiner: James L. Panel Jours Plane	Level: II	Scans: 45 ⊠63	2* 40	70 []	dD	Pyromete Cal Due:			DE 2720	5
Procedure: NDE-610 Rev: 4	FC:	45T ⊠ <u>63.</u> 60 □					2	Flow _	S1	
Calibration Sheet No: 0002037, 0002038		60Т 🗆	dB	dE	3		Scan pplies f	to <u>Surface:</u> to NDE-6	OD	
IND # A Max Mp W % Max Max Max Ref	L Max L1	L2	W1	Mp1	W2	Mp2	Beam Dir.	Exam Surf.	Scan	Damps
DO OT WRITE I THIS SPACE	20%d HM/ 50%d 100%	A HMA ac 50%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	DO I		VRITE SPACE	
NRI 45°										

Limitations: (see NDE-UT-4)	90% or greate	r coverage obtai	ned: yes a no 🛛		Sheet _ 1 of	4
Reviewed By:	Level:	Date:	Authorized Inspector:	Date:	Item No:	
Aug S. Bill	TH	3-18-00	-DOI ATTACHMENT 5	3-28.00	B05.130.007	

	DUKE POWER	COMPANY		FORM NDE-UT-4
	ISI LIMITATION			Revision 1
Component/Weld ID: 2NC11-03		Item No: B05.130.007	Remarks:	
NO SCAN	SURFACE	BEAM DIRECTION	DUE TO NOZZL	E CONFIGURATION
LIMITED SCAN	□ 1 ⊠ 2	🖾 1 🗖 2 🔲 cw 🗌 ccw		
FROM L to LN/A		M WO1.5" toBEYOND		
ANGLE: 0 🛛 45 🗌 60 🗋 Other		FROM 0 DEG to 360 DEG		
	SURFACE	BEAM DIRECTION		
	1 2	🗌 1 🗌 2 🔲 cw 🗌 ccw		
FROM L to L	INCHES FROM	M WO to		
ANGLE: 0 45 60 0ther		FROM DEG to DEG		
	SURFACE	BEAM DIRECTION		····
LIMITED SCAN	□ 1 □ 2	□ 1 □ 2 □ cw □ ccw		
FROM L to L	INCHES FROM	M WO to		
ANGLE: 0 45 60 Other	2 ²	FROM DEG to DEG		
	SURFACE	BEAM DIRECTION		
LIMITED SCAN	1 2	🗌 1 🗌 2 🔲 cw 🗌 ccw		
FROM L to L		M WO to		
ANGLE: 0 45 60 0ther		FROM DEG to		
Prepared By: Kan Mauldin	Level: <u>777</u>	Date: 3-16.00 Sketch(s) attached	yes 🗌 no	Sheet_2_of_4
Reviewed By: Jun, J. Bel	14 Date: 3-18-00	Authorized Inspector: Refeat	NYA	Date: 3-28-00
			- <u>64.4-</u> V	<u> </u>

76/18/00

		DUK	E POWER (COMP	ANY	1			NDE-91-1	
		Limited Ex	amination Cov	verage \	Nork	sheet			Revision 0	
			Examinati	ion Volu	me/A	rea Defined	l		, dere franzischen Allen auf der Scharten ein stellten eine Bister einen auf der Franzensen Stad im 2014 aber 2	
🖾 Ba	se Meta		Veld	🗌 Nea	ar Sui	face	Bolting	J	Inner Radius	
		olume Ca	lculat	tion						
.83" X 2	2.0" = 1.6	6 sq. in.		sq. in. X 116"	= 192.56	cu. in				
			Cov	/erage (Calcu	lations				
		Beam	Area	Leng		Volume	Volu			
Scan#	Angle	Direction	Examined (sq.in.)	Exam (in		Examined (cu.in.)	Requ (cu.		Percent Coverage	
1	45	2	1.66	116		192.56	192		100.00	
2	45	1	0	116		0	192	.56	0.00	
3	45	CW	1.66	116		192.56	192	.56	100.00	
4	45	CCW	1.66	116	5	192.56	192.	.56	100.00	

(Volume Examined) 577.68 / (Volume Reguired) 770.24 X 100 = 75% Coverage

	Item No:	B05.130.007
Prepared By: have Mauldur		Date: 3-16.00
Reviewed By: Suy S. Bubb	Level: III	Date: 3-18-00
	3 of .	4 (4)

	SheetOf	Date 3-16.00	1	Sac.2							WECD.					
COMPANY	File No.	By Laws Mrubli) Klur			8996 / She			C		NNCD DUE 7			16659.116		
DUKE POWER COMPANY	Unit Z Rev.	6780	Checked		÷ 0. - CV					NAL AREA	E WAS NOT SCA	210€,		.83"X 2.0" = 1		
Form 00184 (Fi4-88)	Station CATAWBA	١.	Prob No. 205. 130.007	Save 1				"48.		CROSS SECTIO	- 10N	GENER1	Exam ARZA:	4 B B C Q B C Q		

DUKE POWER COMPANY

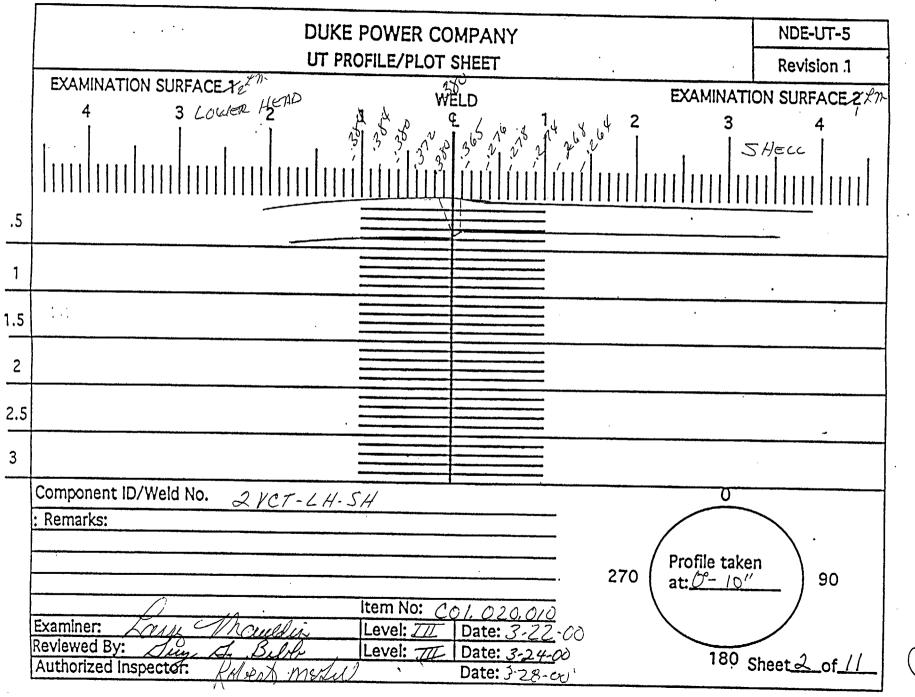
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					WER						Exam St	art:	1027	NE	DE-UT-3A
U					ATA SHE	ET FOF	R LAMIN	AR REF	LECTO	RS	Exam Fi	nish:	1047	R	Revision 2
Static	n:	Ca	tawba		Unit:	2	Compo	nent/Weld	1 ID: 2VC	CT-LH-S	Н			Date:	3/22/00
Nomi	nal Mate	rial Thick	ness (in):		0.312		Weld Le	ength (in.)): 2	82.7	Surf	ace Tempe	erature:	82°	Deg F
Meas	ured Ma	terial Thio	kness (ir	ו):	.241 Lo: 9.2.1				Pyrc	meter S/N	: N	ICNDE 27	205		
Surfa	ce Cond	ition:		AS GRO	GROUND Calibration Sheet No: 0002053				Cal	Due:		7/26/00	26/00		
Exam	iner: La	rry Mauld	in Janak	Mai	Rauldun Level: III				Con	figuration:		lead to Sh	nell		
Exam	iner: Ja	mes L. Pa	anel	10 2 6	Leve	el: 11							S2 Flo	w <u> S1</u>	
Proce	edure:	NDE-	640	Rev: 1	FC:	*						H	HEAD to	SH	ELL
IND NO.	Ą	Ampl ≥ rem BW LOB	L1 ≥ rem BW LOB	W1 ≥ rem BW LOB	Mp1 ≥ rem BW LOB	W2 ≥ rem BW LOB	Mp2 ≥ rem BW . LOB	L2 ≥ rem BW LOB	W1 ≥ rem BW LOB	Mp1 ≥ rem BW LOB	W2 ≥ rem BW LOB	Mp2 ≥ rem BW LOB	Exam Surf.		Damps
NRI	0°										_				<u>4</u>
(Call)										1 5154-1411					

Remarks: *FC 95-18			L	imitations: see NDE-UT-4 🛛	None:	Sheet /	of //
Reviewed By:	S. Bill	Level:	Date: <i>3-24-00</i>	Authorized Inspector:	 Date: えっ2名-でルノ	Item No: C01.020.010	
0	REQUEST	FOR	RELIEF	#00-001 ATTAC	HMENT 6	<u></u>	







			D	UKE PC	WER	COMI	PAN	١Y			Exam St	art: 1	050	Forr	n NDE-U	T-2A
ULT	RASO	NIC E	XAMINA	ATION D	ATA SH	EET F	OR	PLANAF		CTORS	Exam Fi	nish: 1	150		Revision 4	4
Statio	n:	(Catawba		Unit:	2	Con	nponent/V	Veld ID: 2	VCT-LH-S	н SH			Date:	3/22/	······
Weld	Length	(in.):	282	2.7	Surface	Condi	tion:	AS	GROUND	Lo:	9.2.1	Surface	Tempera	ature:	82 °	F
Exami	iner: La	arry Ma	uldin for	Maula	Level	: 111	Sc	cans:	<u> </u>	I		Pyromet	er S/N:		NDE 2720	
				nes # Par			4	5 🖾 4	5 dB	70 🗆	dB	Cal Due:				
	dure:			Rev: 2			1									
					99	-02) []				•	ver Head		S1 Shell	<u> </u>
Calibra	ation Si	heet N	o:					т 🗆					Scan	Surface	: OD	-
000205	53, 0002	2054, 00	002055								-	/ Skew An	Applies	to NDE-	680 only N/A	
				r						dl	B					
IND #	4	Max % Ref	Mp Max	W Max	L Max	L1		L2	W1	Mp1	W2	Mp2	Beam Dir.	Exam Surf.	Scan	Damps
			OT WR IS SPA			20%d HMA 50%d 100%d	ac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	DO I		WRITE SPACE	
1	45°	70	.363	.1	272"	360	, ,	INT.					1	2	AX	NO
2	45°	45	1.3	.9	272"	360'	,	INT.					1	2	AX	NO
3	45°	25	.384	.45	272"	360'	,	INT.					2	1	AX	NO
Remar	ks:							<u></u>								
Limitat	ions: (s	see ND)E-UT-4)	⊠ 90%	or great	er cov	erag	e obtaine	d: yes 🗆	no 🖾				Shee	t_ <u>3</u> _c	of //
,	ved By:	,	A		Level:	Ľ	Date:	. A	\uthørjzed	Inspector:	1		Date:	Item		
_AL	щ.1	<u>J.</u> k	3ible		TH	3	<u>^-2</u>	4-00	Kober	XMey	X)		3.28.00	- C01.0	020.010	
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		111 T	DASONI								•		Form	NDE-UT	-2B
····· ·· <u>·· ·</u> · ·							A SHEET F				s ontinuati	on)	R	evision 3	
Station	า:	(Catawba		Unit:	2 0	Component/V	Veld ID: 2	VCT-LH-S	Н			Date:	3/22/0	00
IND #	4	Max % Ref	Mp Max	W Max	L Max	L1	L2	W1	Mp1	W2	Mp2	Beam Dir.	Exam Surf.	Scan	Damps
			NOT WI HIS SP	RITE ACE		20%dad HMA 50%dad 100%da	HMA c 50%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	20%dac HMA 50%dac 100%dac	ם זו	O NOT N THIS	WRITI SPACE	_
4	45°	32	1.15	1.05	272"	360°	INT.					2	1	AX	NO

Examiner: Larry Mauldin	Ing Maulder	Level: I	II Examiner: James L. Par	el James sare	e Level: II
Remarks:	0				Sheet 4 of //
Reviewed By:	Level:	Date:	Authorized Inspector:	Date:	Item No:
Juy S. Bi	lih III	3-24-00	Robert Mysulf	3.28.00	C01.020.010
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					H.

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DUKE POWER COMPANY	NDE-UT-5
UT PROFILE/PLOT SHEET	Revision .1
EXAMINATION SURFACE TYM WELD EXAMINATI	ON SURFACE 2
	4 PM-
Lower Hend	SHELL
	<u>~</u>
ACTUAL SOUND PATH PLOTED SOUND	PATH
	•
	5
Component ID/Weld No. $2 VCT = 1 H = 5 H$	
: Remarks: INDICATIONS / \$ 2 ORE SAME INDICATION	
	· ·
Item No: CO/. 020, 0/0	
Paylewood Bur of Payle	
Authorized Inspector: Revent Misture Date: 3-28-co '	Sheet <u>5</u> of <u>/ </u>
	UT PROFILE/PLOT SHEET EXAMINATION SURFACE ASM 4 3 2 1/2 3 4

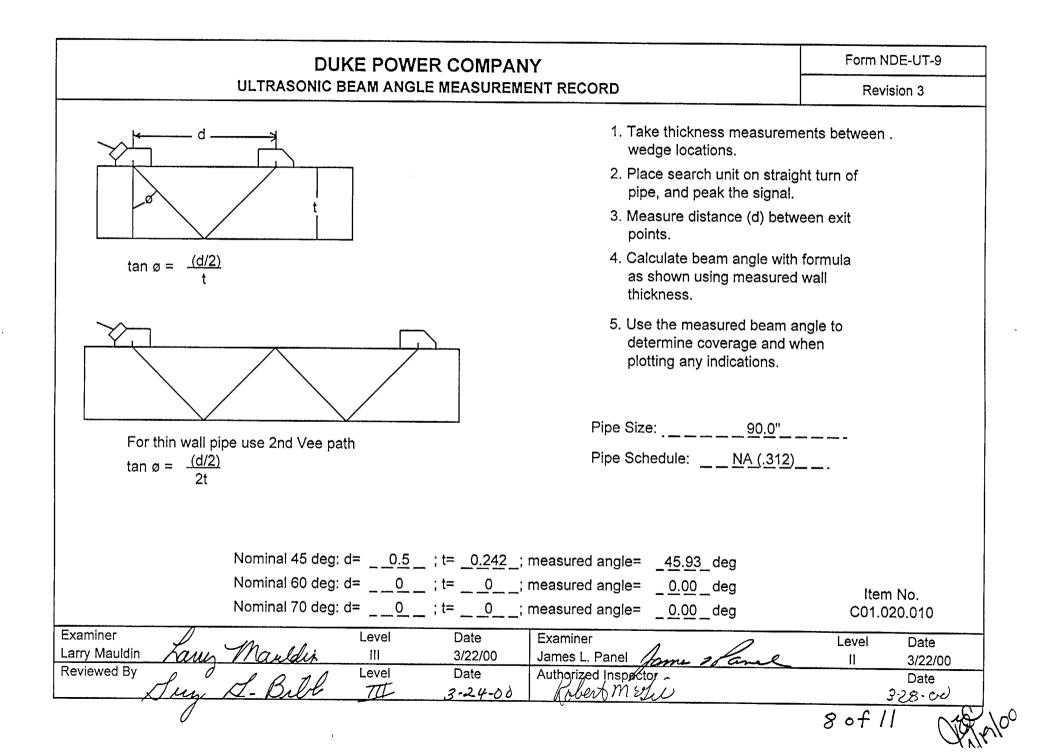
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	DUKE POWER COMPANY NDE-UT-5
	UT PROFILE/PLOT SHEET Revision 1
	EXAMINATION SURFACE TL 4 3 EM 2 1 EXAMINATION SURFACE 21 4 3 EM 2 1 E #3 144 2 3 4 LOWER HEAD 111111111111111111111111111111111111
.5	
1	
.5	
2	
.5	
3	
	Component ID/Weld No. 2 VCT- LH-SH : Remarks: / NAICATIONS 3 \$4 ARP. SAME INDICITION Profile taken
,	$\begin{array}{c ccccccccccccccccccccccccccccccccccc$
	Authorized Inspectors Relievent metho Date: 3-24-00 180 Sheet 6 of 11

[r1/00

DUKE POWER COI	Form NDE-UT-8	
ULTRASONIC INDICATION RES	Revision 1	
Acceptance Standard: AFTER PLOTTING INDICATIONS #1 & #2 WERE DETERMINE REFLECTOR DUE TO THE REDIRECTION OF THE SHEAR W SOUND TO THE WELD/BASE MATERIAL INTERFACE. AFTE SAME INDICATION. THIS INDICATION IS A GEOMETRIC REI CONFIRMED BY THE USE OF MULTIPLE ANGLES. 45° & 60° SHOW AND REFLECTORS NEITHER WOULD THE WSY-70.	AVE CAUSED BY THE AUSTENITIC WELD MA R PLOTTING INDICATIONS #3 & #4 WERE DE FLECTOR DUE TO WELD/BASE INTERFACE	TERIAL BENDING THE TERMINED TO BE THE ALL INDICATIONS WEBE
Item No: C01.020.010		
Acceptable Indications: #1, #2, #3, #4		
Rejectable Indications:		
-	data □ Yes ⊠ No previous data available	
These indications have been compared with previous ultrasonic	•	Sheet 7 of //
These indications have been compared with previous ultrasonic Examiner: Level: Date Larry Mauldin Law Mauldin III 3/22		Sheet _7of _//
I in the	:: /00	Sheet _7of _// Date:



		FORM NDE-UT-4			
		Revision 1			
Component/Weld ID: 2VCT-LH-SH	۲ ا	tem No: C01.020.010	Remarks:		
NO SCAN	SURFACE	BEAM DIRECTION	SUPPORT LEG 8.25" WIDE		
LIMITED SCAN	⊠ 1 ⊠ 2	🖾 1 🖾 2 🖾 cw 🖾 ccw			
FROM L to L 39.46		1 WON/A toN/A			
ANGLE: 🛛 0 🖾 45 🗌 60 🔲 Other		FROM 39.75 DEG to 50.25 DEG			
🖾 NO SCAN	SURFACE	BEAM DIRECTION	SUPPORT LEG	8.25" WIDE	
	⊠ 1 ⊠ 2	🖾 1 🖾 2 🖾 cw 🖾 ccw			
FROM L	INCHES FROM	1 WO toN/A			
ANGLE: 🛛 0 🖾 45 🗌 60 🗍 Other		FROM 129.75 DEG to 140.25 DEG			
🖾 NO SCAN	SURFACE	BEAM DIRECTION	SUPPORT LEG	8.25" WIDE	
LIMITED SCAN	⊠ 1 ⊠ 2	🖾 1 🖾 2 🖾 cw 🖾 ccw			
FROM L					
ANGLE: 🛛 0 🖾 45 🗖 60 🗍 Other		FROM 219.75 DEG to 230.25 DEG			
🖾 NO SCAN	SURFACE	BEAM DIRECTION	SUPPORT LEG	8.25" WIDE	
LIMITED SCAN	⊠ 1 □ 2	🖾 1 🖾 2 🖾 cw 🖾 ccw			
FROM L243.27" to L251.52	INCHES FROM	1 WO N/A to N/A			
ANGLE: 🛛 0 🖾 45 🗌 60 🔲 Other			1		
Prepared By: Land Maulton	Level: 777 Da	ate: 3.22.00 Sketch(s) attached	yes 🗌 no	Sheet 9 of //	
	Date: 3-24-00		Metligh	Date: 3.28.00	

A huloc

DUKE POWER COMPANY Limited Examination Coverage Worksheet						NDE-91-1		
							Revision 0	
			Examinat	ion Volu	me/Area D	efined		
🖾 Ba	se Meta		/eld	🗆 Nea	r Surface	Bolting	g	Inner Radius
Area Calculation Volume C					Volume Ca	lculat	ion	
1.4 / 2 (.360 + .330) = .483 SQ. IN				X 283 IN = 136.69 CU. IN.				
			Cov	verage C	alculation	5		
				-				
Scan #	Angle	Beam Direction	Area Examined (sq.in.)	Leng Exami (in.	ned Exam	nined Requ		Percent Coverage
1	45	1	.483	250	12	0.75 136	.69	88.34
2	45	2	.483	250	12	0.75 136	69	88.34
3	45	CW	.483	250	120).75 136	.69	88.34
4	45	CCW	.483	250	. 120	0.75 136	.69	88.34
					4	83 546	.76	88.34

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		Item No:	C01.020.010	
Prepared By: Laur Mauldus	Level:		Date: 3-22.00	
Reviewed By: Jun, J. Bulik	Level: T		Date: 3-24-00	
		10 of []		A plo

Stock Code No. 89203

