

May 18, 2000

Mr. Craig G. Anderson
Vice President, Operations ANO
Entergy Operations, Inc.
1448 S. R. 333
Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NO. 2 - ISSUANCE OF AMENDMENT RE:
RELOCATION OF HIGH STEAM GENERATOR LEVEL TRIP FUNCTION TO
THE TECHNICAL REQUIREMENTS MANUAL (TAC NO. MA7296)

Dear Mr. Anderson:

The Commission has issued the enclosed Amendment No. 216 to Facility Operating License No. NPF-6 for Arkansas Nuclear One, Unit No. 2. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 29, 1999.

The amendment relocates the requirements associated with the high steam generator level trip functions of the Reactor Protection System from the TSs to the Technical Requirements Manual.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Thomas W. Alexion, Project Manager, Section 1
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-368

Enclosures:

1. Amendment No. 216 to NPF-6
2. Safety Evaluation

cc w/encls: See next page

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Accession No.:

*no substantive change to SE input

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Docket No. 50-368

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1. Amendment No. 216 to NPF-6
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cc w/encls: See next page

Arkansas Nuclear One

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 216
License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated November 29, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 216, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Gramm, Chief, Section 1
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 18, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 216

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

2-6
2-7
2-8
2-9
B 2-7
3/4 3-3
3/4 3-5
3/4 3-5a
3/4 3-7
3/4 3-14
3/4 3-15

Insert

2-6

B 2-7
3/4 3-3
3/4 3-5
3/4 3-5a
3/4 3-7
3/4 3-14
3/4 3-15

TABLE 2.2-1 (Continued)

REACTOR PROTECTIVE INSTRUMENTATION TRIP SETPOINT LIMITS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
9. Local Power Density - High	≤ 21.0 kw/ft (5)	≤ 21.0 kw/ft (5)
10. DNBR - Low	≥ 1.25 (5)	≥ 1.25 (5)

TABLE NOTATION

- (1) Trip may be manually bypassed above $10^{-4}\%$ power; bypass shall be automatically removed before decreasing below $10^{-4}\%$ power.
- (2) Value may be decreased manually, to a minimum value of 100 psia, during a planned reduction in pressurizer pressure, provided the margin between the pressurizer pressure and this value is maintained at ≤ 200 psi; the setpoint shall be increased automatically as pressurizer pressure is increased until the trip setpoint is reached. Trip may be manually bypassed below 400 psia; bypass shall be automatically removed before pressurizer pressure exceeds 500 psia.
- (3) Value may be decreased manually during a planned reduction in steam generator pressure provided the margin between the steam generator pressure and this value is maintained at ≤ 200 psi; the setpoint shall be increased automatically as steam generator pressure is increased until the trip setpoint is reached.
- (4) % of the distance between steam generator upper and lower narrow range level instrument nozzles.
- (5) As stored within the Core Protection Calculator (CPC). Calculation of the trip setpoint includes measurement, calculational and processor uncertainties, and dynamic allowances. Trip may be manually bypassed below $10^{-2}\%$ power; bypass shall be automatically removed before exceeding $10^{-2}\%$ power.

SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

BASES

a.	RCS Cold Leg Temperature-Low	$\geq 490^{\circ}\text{F}$
b.	RCS Cold Leg Temperature-High	$\leq 585^{\circ}\text{F}$
c.	Axial Shape Index-Positive	Not more positive than +0.6
d.	Axial Shape Index-Negative	Not more negative than -0.6
e.	Pressurizer Pressure-Low	≤ 1785 psia
f.	Pressurizer Pressure-High	≤ 2415 psia
g.	Integrated Radial Peaking Factor-Low	≥ 1.28
h.	Integrated Radial Peaking Factor-High	≤ 7.00
i.	Quality Margin-Low	≤ 0

TABLE 3.3-1 (Continued)

REACTOR PROTECTIVE INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
10. DNBR - Low	4	2 (c) (d)	3	1, 2	2, 3
11. Reactor Protection System Logic					
A. Matrix Logic	6	1	3	1, 2	1
	6	1	3	3*, 4*, 5*	8
B. Initiation Logic	4	2	4	1, 2	5
	4	2	4	3*, 4*, 5*	8
12. Reactor Trip Breakers	4 (f)	2	4	1, 2	5
	4 (f)	2	4	3*, 4*, 5*	8
13. Core Protection Calculators	4	2 (c) (d)	3	1, 2	2, 3, 7
14. CEA Calculators	2	1	2 (e)	1, 2	6, 7

TABLE 3.3-1 (Continued)

ACTION STATEMENTS

ACTION 2 - With the number of channels OPERABLE one less than the Total Number of Channels, operation in the applicable MODES may continue provided the inoperable channel is placed in the bypassed or tripped condition within 1 hour. If the inoperable channel is bypassed for greater than 48 hours, the desirability of maintaining this channel in the bypassed condition shall be reviewed at the next regularly scheduled PSC meeting in accordance with the QA Manual Operations. The channel shall be returned to OPERABLE status prior to startup following the next COLD SHUTDOWN.

With a channel process measurement circuit that affects multiple functional units inoperable or in test, bypass or trip all associated functional units as listed below.

<u>Process Measurement Circuit</u>	<u>Functional Unit Bypassed</u>
1. Linear Power (Subchannel or Linear)	Linear Power Level - High Local Power Density - High DNBR - Low Log Power Level - High*
2. Pressurizer Pressure - NR	Pressurizer Pressure - High Local Power Density - High DNBR - Low
3. Containment Pressure - NR	Containment Pressure - High (RPS) Containment Pressure - High (ESFAS) Containment Pressure - High-High (ESFAS)
4. Steam Generator 1 Pressure	Steam Generator 1 Pressure - Low Steam Generator 1 ΔP (EFAS 1) Steam Generator 2 ΔP (EFAS 2)
5. Steam Generator 2 Pressure	Steam Generator 2 Pressure - Low Steam Generator 1 ΔP (EFAS 1) Steam Generator 2 ΔP (EFAS 2)
6. Steam Generator 1 Level	Steam Generator 1 Level - Low Steam Generator 1 ΔP (EFAS 1)
7. Steam Generator 2 Level	Steam Generator 2 Level - Low Steam Generator 2 ΔP (EFAS 2)
8. Core Protection Calculator	Local Power Density - High DNBR - Low

* Only for failure common to both linear power and log power.

TABLE 3.3-1 (Continued)

ACTION STATEMENTS

ACTION 3 - With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement, operation in the applicable MODES may continue provided the following conditions are satisfied:

- a. Verify that one of the inoperable channels has been bypassed and place the other inoperable channel in the tripped condition within 1 hour, and
- b. All functional units affected by the bypassed/tripped channel shall also be placed in the bypassed/tripped condition as listed below:

<u>Process Measurement Circuit</u>	<u>Functional Unit Bypassed/Tripped</u>
1. Linear Power (Subchannel or Linear)	Linear Power Level - High Local Power Density - High DNBR - Low Log Power Level - High**
2. Pressurizer Pressure - NR	Pressurizer Pressure - High Local Power Density - High DNBR - Low
3. Containment Pressure - NR	Containment Pressure - High (RPS) Containment Pressure - High (ESFAS) Containment Pressure - High-High (ESFAS)
4. Steam Generator 1 Pressure	Steam Generator 1 Pressure - Low Steam Generator 1 ΔP (EFAS 1) Steam Generator 2 ΔP (EFAS 2)
5. Steam Generator 2 Pressure	Steam Generator 2 Pressure - Low Steam Generator 1 ΔP (EFAS 1) Steam Generator 2 ΔP (EFAS 2)
6. Steam Generator 1 Level	Steam Generator 1 Level - Low Steam Generator 1 ΔP (EFAS 1)
7. Steam Generator 2 Level	Steam Generator 2 Level - Low Steam Generator 2 ΔP (EFAS 2)
8. Core Protection Calculator	Local Power Density - High DNBR - Low

Operation in the applicable MODES may continue until the performance of the next required CHANNEL FUNCTIONAL TEST. Subsequent operation in the applicable MODES may continue if one channel is restored to OPERABLE status and the provisions of ACTION 2 are satisfied.

** Only for failure or activities common to both linear power and log power.

TABLE 4.3-1

REACTOR PROTECTION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
1. Manual Reactor Trip	N.A.	N.A.	S/U(1)	N.A.
2. Linear Power Level - High	S	D(2,4) M(3,4) Q(4)	TA(10)	1, 2
3. Logarithmic Power Level - High	S	R(4)	TA(10) S/U(1)	1, 2, 3, 4, 5 and *
4. Pressurizer Pressure - High	S	R	TA(10)	1, 2
5. Pressurizer Pressure - Low	S	R	TA(10)	1, 2, 3*, 4*, 5*
6. Containment Pressure - High	S	R	TA(10)	1, 2
7. Steam Generator Pressure - Low	S	R	TA(10)	1, 2, 3*, 4*, 5*
8. Steam Generator Level - Low	S	R	TA(10)	1, 2
9. Local Power Density - High	S	D(2,4) R(4,5)	TA(10) R(6)	1, 2
10. DNBR - Low	S	S(7), D(2,4), M(8), R(4,5)	TA(10) R(6)	1, 2
11. Reactor Protection System Logic	N.A.	N.A.	TA(10)	1, 2, 3*, 4*, 5*
12. Reactor Trip Breakers	N.A.	N.A.	M	1, 2, 3*, 4*, 5*
13. Core Protection Calculators	S	D(2,4) R(4,5)	TA(9,10) R(6)	1, 2
14. CEA Calculators	S	R	TA(10) R(6)	1, 2

TABLE 3.3-3 (Continued)

TABLE NOTATION

- (a) Trip function may be bypassed in this MODE when pressurizer pressure is below 400 psia; bypass shall be automatically removed before pressurizer pressure exceeds 500 psia.
- (b) An SIAS signal is first necessary to enable CSAS logic.
- (c) Remote manual not provided for RAS. These are local manuals at each ESF auxiliary relay cabinet.

ACTION STATEMENTS

ACTION 9 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

ACTION 10 - With the number of channels OPERABLE one less than the Total Number of Channels, operation in the applicable MODES may continue provided the inoperable channel is placed in the bypassed or tripped condition within 1 hour. If the inoperable channel is bypassed for greater than 48 hours, the desirability of maintaining this channel in the bypassed condition shall be reviewed at the next regularly scheduled PSC meeting in accordance with the QA Manual Operations. The channel shall be returned to OPERABLE status prior to startup following the next COLD SHUTDOWN.

If an inoperable Steam Generator Δ P or RWT Level - Low channel is placed in the tripped condition, remove the inoperable channel from the tripped condition within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

With a channel process measurement circuit that affects multiple functional units inoperable or in test, bypass or trip all associated functional units as listed below.

<u>Process Measurement Circuit</u>	<u>Functional Unit Bypassed</u>
1. Containment Pressure - NR	Containment Pressure - High (RPS) Containment Pressure - High (ESFAS) Containment Pressure - High-High (ESFAS)
2. Steam Generator 1 Pressure	Steam Generator 1 Pressure - Low Steam Generator 1 Δ P (EFAS 1) Steam Generator 2 Δ P (EFAS 2)
3. Steam Generator 2 Pressure	Steam Generator 2 Pressure - Low Steam Generator 1 Δ P (EFAS 1) Steam Generator 2 Δ P (EFAS 2)
4. Steam Generator 1 Level	Steam Generator 1 Level - Low Steam Generator 1 Δ P (EFAS 1)
5. Steam Generator 2 Level	Steam Generator 2 Level - Low Steam Generator 2 Δ P (EFAS 2)

TABLE 3.3-3 (Continued)

TABLE NOTATION

ACTION 11 - With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement, operation in the applicable MODES may continue provided the following conditions are satisfied:

- a. Verify that one of the inoperable channels has been bypassed and place the other inoperable channel in the tripped condition within 1 hour, and
- b. All functional units affected by the bypassed/tripped channel shall also be placed in the bypassed/tripped condition as listed below:

<u>Process Measurement Circuit</u>	<u>Functional Unit Bypassed/Tripped</u>
1. Containment Pressure - NR	Containment Pressure - High (RPS) Containment Pressure - High (ESFAS) Containment Pressure - High-High (ESFAS)
2. Steam Generator 1 Pressure	Steam Generator 1 Pressure - Low Steam Generator 1 ΔP (EFAS 1) Steam Generator 2 ΔP (EFAS 2)
3. Steam Generator 2 Pressure	Steam Generator 2 Pressure - Low Steam Generator 1 ΔP (EFAS 1) Steam Generator 2 ΔP (EFAS 2)
4. Steam Generator 1 Level	Steam Generator 1 Level - Low Steam Generator 1 ΔP (EFAS 1)
5. Steam Generator 2 Level	Steam Generator 2 Level - Low Steam Generator 2 ΔP (EFAS 2)

If an inoperable Steam Generator ΔP or RWT Level - Low channel is placed in the tripped condition, remove the inoperable channel from the tripped condition within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

Operation in the applicable MODES may continue until the performance of the next required CHANNEL FUNCTIONAL TEST. Subsequent operation in the applicable MODES may continue if one channel is restored to OPERABLE status and the provisions of ACTION 10 are satisfied.

ACTION 12 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

ACTION 13 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours; however, one channel may be bypassed for up to 1 hour for surveillance testing provided the other channel is OPERABLE.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO216 TO

FACILITY OPERATING LICENSE NO. NPF-6

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 2

DOCKET NO. 50-368

1.0 INTRODUCTION

By letter dated November 29, 1999 (Ref. 1), Entergy Operations, Inc. (the licensee), submitted a request for changes to the Arkansas Nuclear One, Unit No. 2 (ANO-2), Technical Specifications (TSs). The requested changes would relocate the Safety Limit Settings, Limiting Conditions for Operation (LCO), Surveillance Requirements (SR), and bases associated with the high steam generator (SG) level trip function of the Reactor Protection System (RPS) from the TSs to the ANO-2 Technical Requirements Manual (TRM), which is a part of the Safety Analysis Report (SAR).

2.0 BACKGROUND

Section 182a of the Atomic Energy Act of 1954, as amended, requires applicants for nuclear power plant operating licenses to include TSs as a part of the license. The Nuclear Regulatory Commission's (NRC or the Commission) regulatory requirements related to the content of TSs are set forth in Title 10 of the Code of Federal Regulations (10 CFR), Section 50.36, which requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) LCOs; (3) SRs; (4) design features; and (5) administrative controls.

10 CFR 50.36 sets forth the following four criteria to be used in determining whether an LCO is required to be included in the TSs:

- (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary (RCPB);
- (2) A process variable, design feature, or operating restriction that is an initial condition of a design basis accident (DBA) or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier;

- (3) A structure, system, or component (SSC) that is part of the primary success path and which functions or actuates to mitigate a DBA or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; or
- (4) An SSC which operating experience or probabilistic risk assessment (PRA) has shown to be significant to public health and safety.

Therefore, existing TS LCOs and related SRs that fall within or satisfy any of these criteria must be retained in the TSs, while those TS requirements that do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

3.0 EVALUATION

The RPS "Steam Generator Level - High" trip function is provided to protect the main turbine from excessive moisture carryover that may result in damage to the turbine in the event of a feedwater transient. During a feedwater malfunction, SG level may rise to the point that significant portions of the moisture separators are covered with water. At some point, the moisture carryover, normally less than 1%, begins to rise. The moisture mist entering the steam line could cause increased vibration, blade wear, and eventual permanent damage to the main turbine. Therefore, upon the SG level exceeding the high SG level trip setpoint, a reactor trip is initiated, which, in turn, automatically trips the main turbine. However, the main turbine is not a safety-related component and its loss does not impact the safety of the reactor core.

The high SG level trip function does not act to protect the reactor core. This trip function is not credited in any DBA or transient analysis, nor does it correspond to any safety limit. This trip function (1) is not an instrumentation that is used to detect a significant abnormal degradation of the RCPB; (2) is not a process variable, design feature, nor operating restriction that is an initial condition of a DBA or transient analysis; and (3) is not relied upon as a part of the primary success path nor functions or actuates to mitigate a DBA or transient.

As discussed in its safety evaluation (Ref. 2) on Combustion Engineering Owners Group's (CEOG) response to NRC Generic Letter 89-09 regarding the SG overflow protection, the NRC staff accepted the CEOG's contention from a PRA viewpoint that SG overflow events do not have a significant impact on the public health and safety. The NRC staff also granted the relocation of the turbine overspeed trip function from the ANO-2 TS to the TRM (Ref. 3), concluding that there is low likelihood of significant risk to public health and safety because of turbine overspeed events. Similar to the turbine overspeed trip function, the high SG level trip function is not significant to public health and safety, as its main purpose is to help prevent SG overflow and protect the main turbine from excessive moisture carryover.

Accordingly, the RPS high SG level trip function does not meet any of the 10 CFR 50.36 criteria for inclusion in the TS and can be removed from the TS. The licensee-requested TS changes would delete the following items related to the high SG level trip function, and relocate them to the TRM.

- (1) The setpoint and allowable value of Functional Unit 11, "Steam Generator Level - High," in TS Table 2.2-1, "Reactor Protective Instrumentation Trip Setpoint Limits," and the associated notation and Bases;

- (2) The functional requirements for Functional Unit 11, "Steam Generator Level - High," and references to the "Steam Generator 1 and 2 Level - High" Functional Unit in Actions 2 and 3 in TS Table 3.3-1, "Reactor Protective Instrumentation;" and
- (3) The SRs associated with Functional Unit 11, "Steam Generator Level - High" RPS channel in TS Table 4.3-1, "Reactor Protection Instrumentation Surveillance Requirements."

In addition, the licensee requested to delete the "Steam Generator 1 Level - High" and "Steam Generator 2 Level - High" functions as "Functional Units Bypassed" in Actions 10 and 11 under TS Table 3.3-3, "Engineered Safeguards Features Actuation System (ESFAS) Instrumentation." Because the high SG level function does not activate any ESFAS components and has no role within the Emergency Safeguards Features system, the staff concludes that references to these function units as bypassed functions in the action statements may be deleted without a need to relocate to the TRM.

The staff has reviewed the revised ANO-2 TRM provided in Enclosure 2 to the November 29, 1999, letter. Item 1 above has been appropriately incorporated into the TRM Section 2.2.1, "Reactor Trip Setpoints." Items 2 and 3 have also been appropriately incorporated into TRM LCO 3.3.1.1 and SR 4.3.1.1 for "Reactor Protective Instrumentation."

The ANO-2 SAR, Section 13.8.2, states that the TRM is administered as part of the SAR, changes to the TRM are subject to the criteria of 10 CFR 50.59, and administrative controls for processing TRM changes are included in the site procedure. Therefore, the staff finds that sufficient regulatory controls exist for the TRM.

Accordingly, the staff has concluded that the above requirements associated with the high SG level trip function may be deleted from the TSs because (1) the 10 CFR 50.36 TS inclusion criteria are not applicable to the high SG level trip function, and (2) these requirements have been appropriately relocated to the licensee's TRM.

Additionally, the licensee requested authorization to delete the footnote under Action 2 at the bottom of TS Table 3.3-1, which states that Channel "D" of Functional Units 2, 3a, 9, 10, and 14 is not required to be restored to OPERABLE status until the startup following the mid-cycle outage (2P99). This footnote was added because the Channel "D" excore detector was inoperable for the first half of Cycle 14. The licensee stated that the Channel "D" excore detector referenced in this footnote was replaced during the ANO-2 SG inspection outage (2P99), and has since returned to an operable status. Therefore, the staff concludes that the allowance to exclude Functional Units 2, 3a, 9, 10, and 14 of Channel "D" from operable status is no longer required, and the footnote may be deleted.

4.0 SUMMARY

The staff has reviewed the licensee's request to relocate portions of the ANO-2 TS associated with the "Steam Generator Level - High" RPS trip function to the TRM, and to delete the

references to the high SG level ESFAS function units and the footnote related to the Channel "D" excore detector allowance. Based on its evaluation, the staff found these requested TS changes acceptable.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes SRs. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 6404, dated February 9, 2000). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with issuance of the amendment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Y. Hsii

Date: May 18, 2000

REFERENCES

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