PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATING STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for **April 2000** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented in **April 2000.**

Sincerely,

Mark B. Bezilla
Vice President - Operations

RAR
Attachments

C Distribution

IE24

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DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 5/10/00
COMPLETED BY: F. Todd
TELEPHONE: (856) 339-1316

Reporting Period **April 2000**

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)	**1067**		
Maximum Dependable Capacity (MWe-Net)	**1031**		
	Month	**Year-to-date**	**Cumulative**
No. of hours reactor was critical	**504**	**2688**	**99415**
No. of hours generator was on line (service hours)	**503**	**2687**	**97834**
Unit reserve shutdown hours	**0**	**0**	**0**
Net Electrical Energy (MWH)	**500030**	**2687005**	**98985604**

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT
1	**4/21/00**	**S**	**216**	**C**	**1**	**RF09**

(1) Reason

A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training/License Examination
F - Administrative
G - Operational Error (Explain)
H - Other

(2) Method

1 - Manual
2 - Manual Trip/Scram
3 - Automatic Trip/Scram
4 - Continuation
5 - Other (Explain)

DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 5/10/00
COMPLETED BY: R. Ritzman
TELEPHONE: (856) 339-1445

Summary Of Monthly Operating Experience

- Hope Creek entered the month of April at approximately 86% reactor power. Power had been reduced in March for "A" feedwater heater string maintenance. Power was restored to approximately 100% on April 9.
- On April 10, Hope Creek began a coast down prior to its scheduled refueling outage.
- Hope Creek shut down for its 9th refueling outage on April 21, after completing 233 days of continuous power operation.
- At the end of the month, Hope Creek remained in its refueling outage.

DOCKET NO.:50-354
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COMPLETED BY:R. Ritzman
TELEPHONE: (856) 339-1445

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE HOPE CREEK GENERATING STATION

MONTH April 2000

The following items completed during **April 2000** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

4HE-0167, Package 2, Addition of Wide Range Reactor Vessel Level and Wide Range Pressure Signals to GETARS. This design change modified the "B" and "D" Wide Range Reactor Vessel Level Channels to provide an input for GETARS. This will provide additional information to facilitate analysis of transients, specifically the reactor vessel transmitter response following a reactor scram caused by a turbine trip.

GETARS is not safety related; its function is to record data before and after transients. All connections to safety systems are isolated by a Class 1E multiplexor. The new system does not perform any control functions or impact any existing control functions. This design change does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE HOPE CREEK GENERATING STATION – Cont'd

Temporary Modifications Summary of Safety Evaluations

TMR 00-010, Install a Pipe Plug Inside the "B" Service Water Loop Piping to Maintain Secondary Containment. This temporary modification installed a pipe plug inside a portion of the "B" Station Service Water piping to preserve Secondary Containment in support of maintenance activities.

The pipe plug is consistent in design and materials with the Service Water piping. The pipe plug exceeds the Secondary Containment pressure requirements, and does not have any additional stress concerns. This temporary modification meets and exceeds the requirements for maintaining a Secondary Containment pressure boundary. The Secondary Containment integrity will be maintained during all phases of pipe plug installation, operation, and removal. The pipe plug will not be installed when the "B" Station Service Water loop is in service or operable. The pipe plug will not affect the operation of "A" Station Service Water loop. The "A" Station Service Water loop will be able to respond to all automatic functions and fulfill the design requirements for the Service Water system. This temporary modification does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

TMR 00-011, Install Blind Flange on "B" Station Service Water Loop Emergency Overboard Pipe to Maintain Secondary Containment. This temporary modification replaces the "B" Station Service Water loop "beehive grate" with a blind flange. The blind flange provides a safety related, ASME, pressure boundary seal of Secondary Containment as allowed by Tech Specs. The blind flange is equivalent to the other Service Water piping in service, and exceeds the required Secondary Containment design pressures and the stresses have been evaluated and found to be acceptable. The valve and rupture disk will be removed after the blind flange is installed. The valve and rupture disk will be reinstalled before the blind flange is removed to maintain Secondary Containment integrity. The blind flange will be installed after the "B" Station Service Water loop is removed from service and will not be in service or operable while the blind flange is installed. The temporary modification will be removed before the "B" Station Service Water loop is returned to service. The "A" Station Service Water loop will remain unaffected by the installation of this temporary modification and will fulfill the design requirements of the system while the temporary modification is installed. This temporary modification does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

Procedures Summary of Safety Evaluations

There were no reportable changes in this category implemented during April 2000.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE HOPE CREEK GENERATING STATION – Cont'd

UFSAR Change Notices Summary of Safety Evaluations

UFSAR Change Notice H00-012, Hope Creek Cycle 10 Fuel Design and Methodology. This change notice introduces ABB fuel design and analysis methodology to the UFSAR in support of the Cycle 10 reload. These changes include fuel rod array configuration, changes in mechanical design, thermal-mechanical design, nuclear design, and thermal hydraulic design.

The design, while different, is similar to the previous design. The overall bundle and channel dimensions are the same, and the two designs are hydraulically compatible; therefore, the overall core pressure distribution is expected to be similar to previous cycles. The implementation of this methodology does not change any functional requirements of the fuel or other equipment that is important to safety. This change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notice H00-013, Drywell Head Seal Protector and Polar Crane Auxiliary/Main Hoist. This change notice allows for the use of the main hoist in certain applications. The auxiliary hoist will still be the preferred hoist for these lifts, but the use of either will be allowed. This change notice allows removes reference to the Drywell Head Sealing Surface Protector. This protector is not being removed from the plant; however, reference to it is being removed from the UFSAR.

The use of the main hoist in addition to the auxiliary hoist increases the safety and reliability of the load handling evolutions. The removal of reference to the Drywell Head Sealing Surface Protector is an administrative change that removes excessive detail from the UFSAR. This change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notice H00-019, Hope Creek Cycle 10 Core Design. This change notice discusses the Hope Creek Cycle 10 reload core. The Cycle 10 core will be comprised of a combination of fresh ABB fuel assemblies and GE fuel assemblies that had been loaded in previous cycles.

ABB performed the reload analysis using NRC approved methods. The reactivity characteristics of the Cycle 10 core have been determined to meet shutdown margin design requirements. The Anticipated Operational Occurrences Design Basis Accidents have been evaluated for the Cycle 10 reload core and the reload was determined to be acceptable. This change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.