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*A Member of the  
Constellation Energy Group*

May 11, 2000

U. S. Nuclear Regulatory Commission  
Washington, DC 20555

**ATTENTION:** Document Control Desk

**SUBJECT:** Calvert Cliffs Nuclear Power Plant  
Unit No. 1; Docket No. 50-317  
Cycle 15, Core Operating Limits Report, Revision 1

Pursuant to Calvert Cliffs Nuclear Power Plant Technical Specification 5.6.5, the attached Core Operating Limits Report for Unit 1, Cycle 15, Revision 1 (Attachment 1) is provided for your records. Also included for your information is the Core Operating Limits Report for Unit 1, Cycle 15, Revision 0 (Attachment 2). This revision was only for Modes 5 and 6.

Please replace the Unit 1 Core Operating Limits Report in its entirety, with the attached Revision 1.

Should you have questions regarding this matter, we will be pleased to discuss them with you.

Very truly yours,

A handwritten signature in black ink, appearing to read 'Peter E. Katz'. The signature is fluid and cursive, with the first and last names being more prominent than the middle initial.

for  
Charles H. Cruse  
Vice President, Nuclear Energy

CHC/DJM/bjd

Attachments: (1) Core Operating Limits Report for Unit 1, Cycle 15, Revision 1  
(2) Core Operating Limits Report for Unit 1, Cycle 15, Revision 0

cc: **(Without Attachments)**  
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A001

# CALVERT CLIFFS NUCLEAR POWER PLANT

## CORE OPERATING LIMITS REPORT

for

UNIT 1, CYCLE 15

REVISION 1

Timothy A. Scheerer 4/12/00  
RESPONSIBLE ENGINEER / DATE

J. Hansel Blum III 4/12/2000  
INDEPENDENT REVIEWER / DATE

W. A. F. 4/12/2000  
PRINCIPAL ENGINEER - NEU / DATE

# CORE OPERATING LIMITS REPORT

## CALVERT CLIFFS UNIT 1, CYCLE 15

The following limits are included in this Core Operating Limits Report:

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**CORE OPERATING LIMITS REPORT**  
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All pages are being reissued under revision 1, due to the substantial number of changes required to make the COLR applicable to all plant modes.

## INTRODUCTION

This report provides the cycle-specific core operating limits for operation of Calvert Cliffs Unit 1, Cycle 15. It contains the limits for:

- Shutdown Margin (SDM)
- Moderator Temperature Coefficient (MTC)
- Control Element Assembly (CEA) Alignment
- Regulating Control Element Assembly (CEA) Insertion Limits
- Linear Heat Rate (LHR)
- Total Planar Radial Peaking Factor ( $F_{xy}^T$ )
- Total Integrated Radial Peaking Factor ( $F_r^T$ )
- Axial Shape Index (ASI)
- Reactor Protective System (RPS) Instrumentation - Operating
- Boron Concentration

In addition, this report contains a number of figures which provides limits on the parameters listed above. If any of the limits contained in this report are exceeded, corrective action will be taken as defined in the Technical Specifications.

This report has been prepared in accordance with the requirements of the Technical Specifications. The cycle specific limits have been developed using the NRC-approved methodologies given in the "List of Approved Methodologies" section of this report and the Technical Specifications.

### COLR Revision 0

COLR Revision 0 was issued to support the Unit 1 Cycle 15 core onload and operation of Unit 1 Cycle 15 in Modes 5 and 6. Safety Evaluation SE00424 Rev. 0000 provided justification for this revision.

### COLR Revision 1

COLR Revision 1 was issued to incorporate those limits required for operation of Unit 1 Cycle 15 in Modes 1 through 4. No revision bars are included due to the significant number of changes required to incorporate the Modes 1 through 4 COLR limits. Safety Evaluation SE00407 Rev. 0000 provided justification for this revision.

## DEFINITIONS

### Axial Shape Index (ASI)

ASI shall be the power generated in the lower half of the core less the power generated in the upper half of the core, divided by the sum of the power generated in the lower and upper halves of the core.

$$ASI = \frac{\text{lower} - \text{upper}}{\text{lower} + \text{upper}} = Y_E$$

The Axial Shape Index ( $Y_I$ ) used for the trip and pretrip signals in the Reactor Protection System (RPS) is the above value ( $Y_E$ ) modified by an appropriate multiplier (A) and a constant (B) to determine the true core axial power distribution for that channel.

$$Y_I = AY_E + B$$

### Total Integrated Radial Peaking Factor - $F_r^T$

The Total Integrated Radial Peaking Factor is the ratio of the peak pin power to the average pin power in an unrodded core.

### Total Planar Radial Peaking Factor - $F_{xy}^T$

The Total Planar Radial Peaking Factor is the maximum ratio of the peak to average power density of the individual fuel rods in any of the unrodded horizontal planes.

## CYCLE SPECIFIC LIMITS FOR UNIT 1, CYCLE 15

### 3.1.1 Shutdown Margin (SDM) (SR 3.1.1.1)

*T<sub>avg</sub> > 200 °F - Modes 3 and 4:*

The shutdown margin shall be equal to or greater than the limit line of COLR Figure 3.1.1.

*T<sub>avg</sub> ≤ 200 °F - Mode 5:*

The shutdown margin shall be  $\geq 3.0\% \Delta p$ .

### 3.1.3 Moderator Temperature Coefficient (MTC) (SR 3.1.3.2)

The Moderator Temperature Coefficient (MTC) shall be less negative than  $-3.0 \times 10^{-4} \Delta p / ^\circ F$  at rated thermal power.

### 3.1.4 Control Element Assembly (CEA) Alignment (Action 3.1.4.B.1)

The allowable time to realign a CEA may be provided by the full core power distribution monitoring system (Better Axial Shape Selection System - BASSS) or COLR Figure 3.1.4, "Allowable Time to Realign CEA Versus Initial Total Integrated Radial Peaking Factor ( $F_r^T$ ).\" If COLR Figure 3.1.4 is used, the pre-misaligned  $F_r^T$  value used to determine the allowable time to realign the CEA shall be the latest measurement taken within 5 days prior to the CEA misalignment. If no measurements have been taken within 5 days prior to the misalignment and the full core power distribution monitoring system is unavailable then the time to realign is zero (0) minutes.

### 3.1.6 Regulating Control Element Assembly (CEA) Insertion Limits (SR 3.1.6.1 and SR 3.1.6.2)

The regulating CEA groups insertion limits are shown on COLR Figure 3.1.6.

### 3.2.1 Linear Heat Rate (LHR) (SR 3.2.1.2 and SR 3.2.1.4)

The linear heat rate shall not exceed the limits shown on COLR Figure 3.2.1-1.

The axial shape index power dependent control limits are given in COLR Figure 3.2.1-2.

When using the excore detector monitoring system (SR 3.2.1.2):

The alarm setpoints are equal to the ASI limits, therefore when the alarms are adjusted, they provide indication to the operator when ASI is not with the limits.

The maximum allowable fraction of rated thermal power vs.  $F_{xy}^T$  is shown in COLR Figure 3.2.1-3.

The axial shape index alarm setpoints are shown on COLR Figure 3.2.1-2.

When using the incore detector monitoring system (SR 3.2.1.4):

The alarm setpoints are adjusted to protect the Linear Heat Rate limits shown on COLR Figure 3.2.1-1 and uncertainty factors are appropriately included in the setting of these alarms.

## CYCLE SPECIFIC LIMITS FOR UNIT 1, CYCLE 15

The uncertainty factors for the incore detector monitoring system are:

1. A measurement-calculational uncertainty factor of 1.062,
2. An engineering uncertainty factor of 1.03,
3. A linear heat rate uncertainty factor of 1.002 due to axial fuel densification and thermal expansion, and
- 4.a For measured thermal power less than or equal to 50 percent but greater than 20 percent of rated full core power a thermal power measurement uncertainty factor of 1.035.
- 4.b For measured thermal power greater than 50 percent of rated full core power a thermal power measurement uncertainty factor of 1.020.

### **3.2.2 Total Planar Radial Peaking Factor ( $F_{xy}^T$ ) (SR 3.2.1.1 and SR 3.2.2.1)**

The calculated value of  $F_{xy}^T$  shall be limited to  $\leq 1.65$ .

The allowable combination of thermal power and  $F_{xy}^T$  are shown on COLR Figure 3.2.2.

### **3.2.3 Total Integrated Radial Peaking Factor ( $F_r^T$ ) (SR 3.2.3.1)**

The calculated value of  $F_r^T$  shall be limited to  $\leq 1.65$ .

The allowable combinations of thermal power and  $F_r^T$  are shown on COLR Figure 3.2.3.

### **3.2.5 Axial Shape Index (ASI) (SR 3.2.5.1)**

The axial shape index and thermal power shall be maintained within the limits established by the Better Axial Shape Selection System (BASSS) for CEA insertions of the lead bank of < 55% when BASSS is operable, or within the limits of COLR Figure 3.2.5 for CEA insertions specified by COLR Figure 3.1.6.

### **3.3.1 Reactor Protective System (RPS) Instrumentation - Operating (Reactor Trip Setpoints) (TS Table 3.3.1-1)**

The Axial Power Distribution - High trip setpoint and allowable values are given in COLR Figure 3.3.1-1.

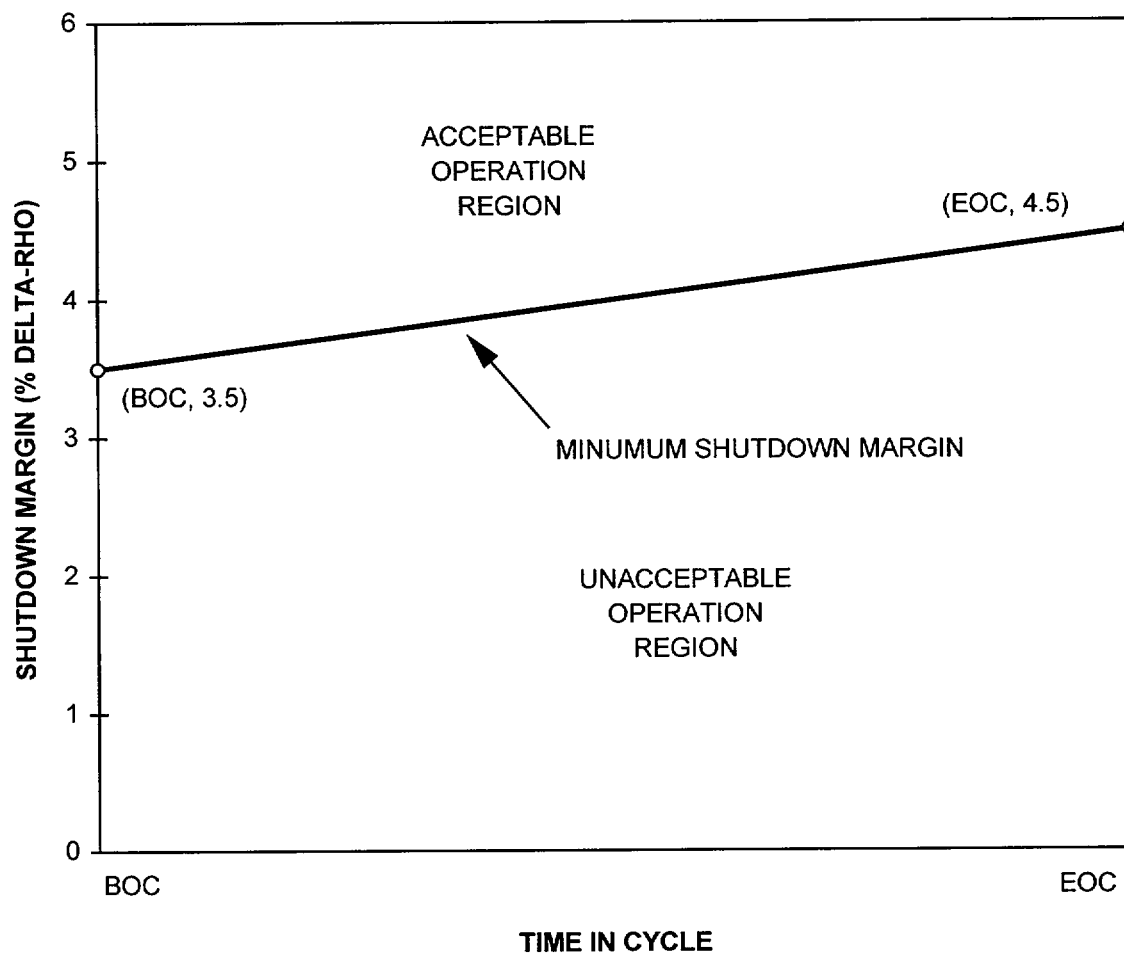
The Thermal Margin/Low Pressure (TM/LP) trip setpoint is given in COLR Figures 3.3.1-2 and 3.3.1-3. The allowable values are to be not less than the larger of (1) 1875 psia or (2) the value calculated from COLR Figures 3.3.1-2 and 3.3.1-3.

### 3.9.1 Boron Concentration (SR 3.9.1.1)

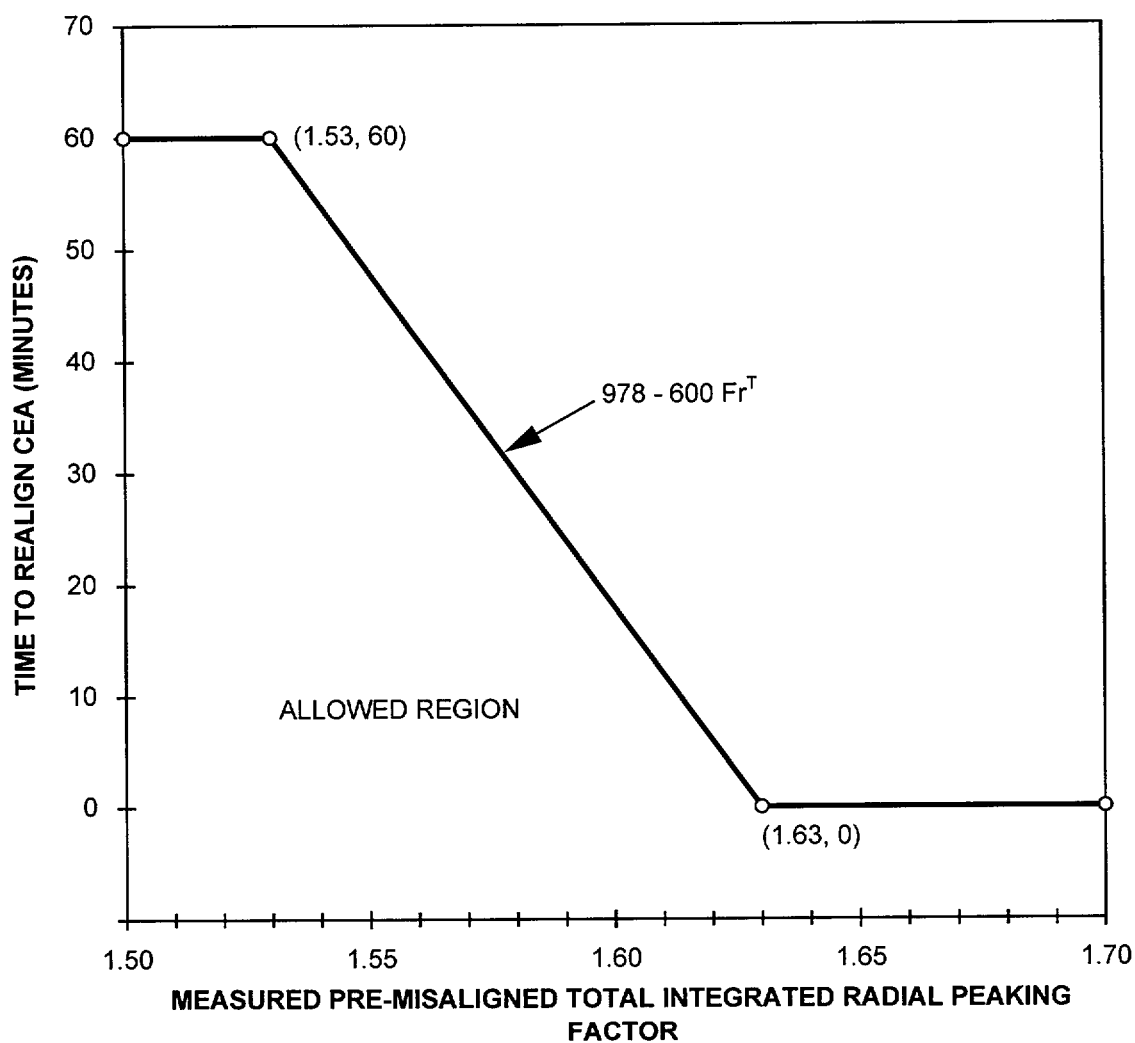
The refueling boron concentration will maintain the  $K_{eff}$  at 0.95 or less (including a 1%  $\Delta K/K$  conservative allowance for uncertainties. The refueling boron concentration shall be maintained uniform.

#### Refueling Boron Concentration Limits

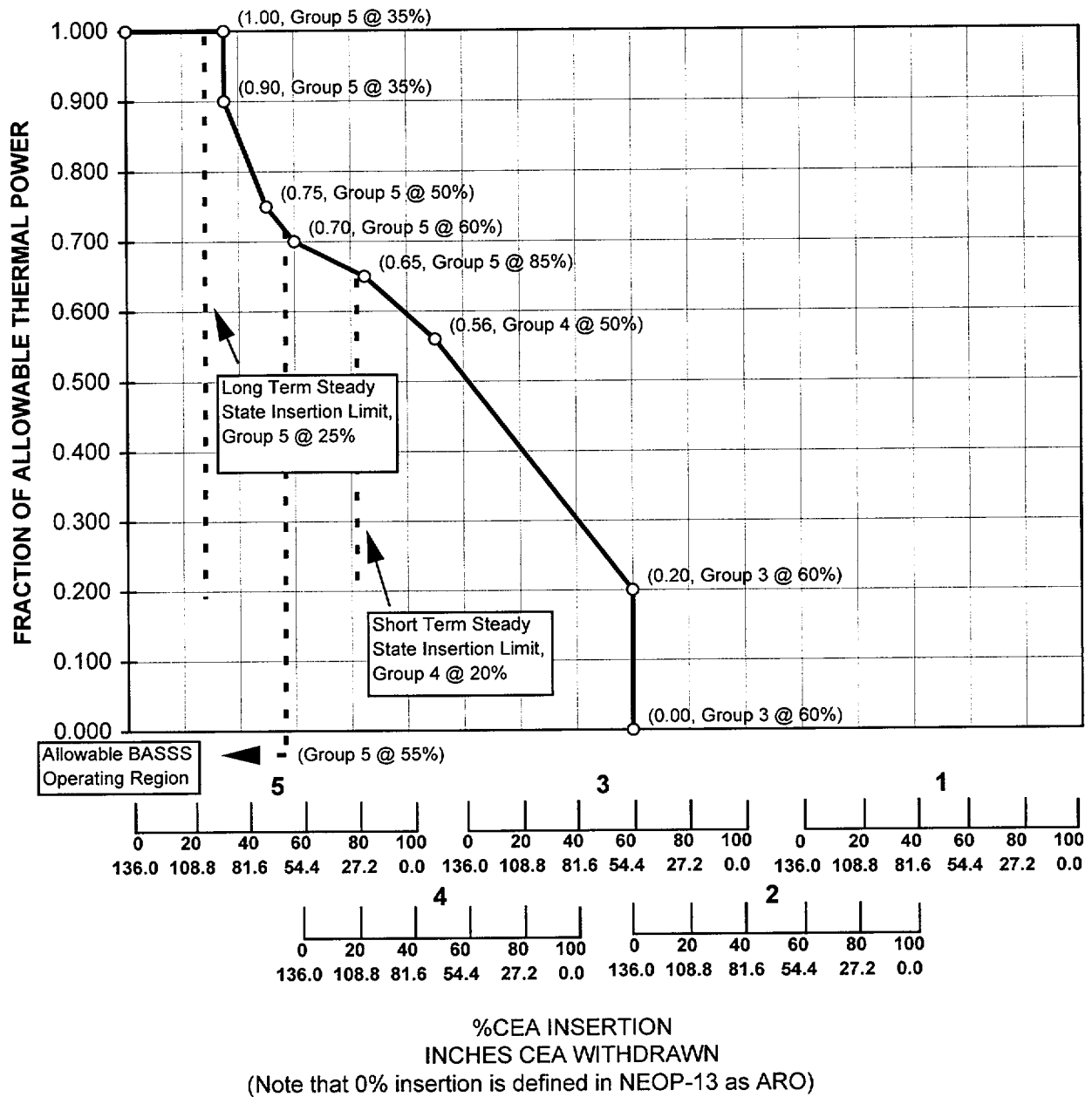
	56 Credited CEAs
COLR/Tech Spec Limit	2272 ppm
Chemistry Sampling	23 ppm
Boron-10 Depletion	48 ppm
Refueling Boron Concentration Limit including Chemistry Sampling uncertainty and Boron-10 Depletion	$\geq 2343$ ppm
Dilution of the Refueling Pool between Low and High Level Alarms with Refueling Pool Flooded	72 ppm
Additional Contingencies for Batch 1RT LFAs	20 ppm
Temporary Rotations of Fuel Assemblies	20 ppm
Refueling Boron Concentration Administrative Limit	$\geq 2455$ ppm



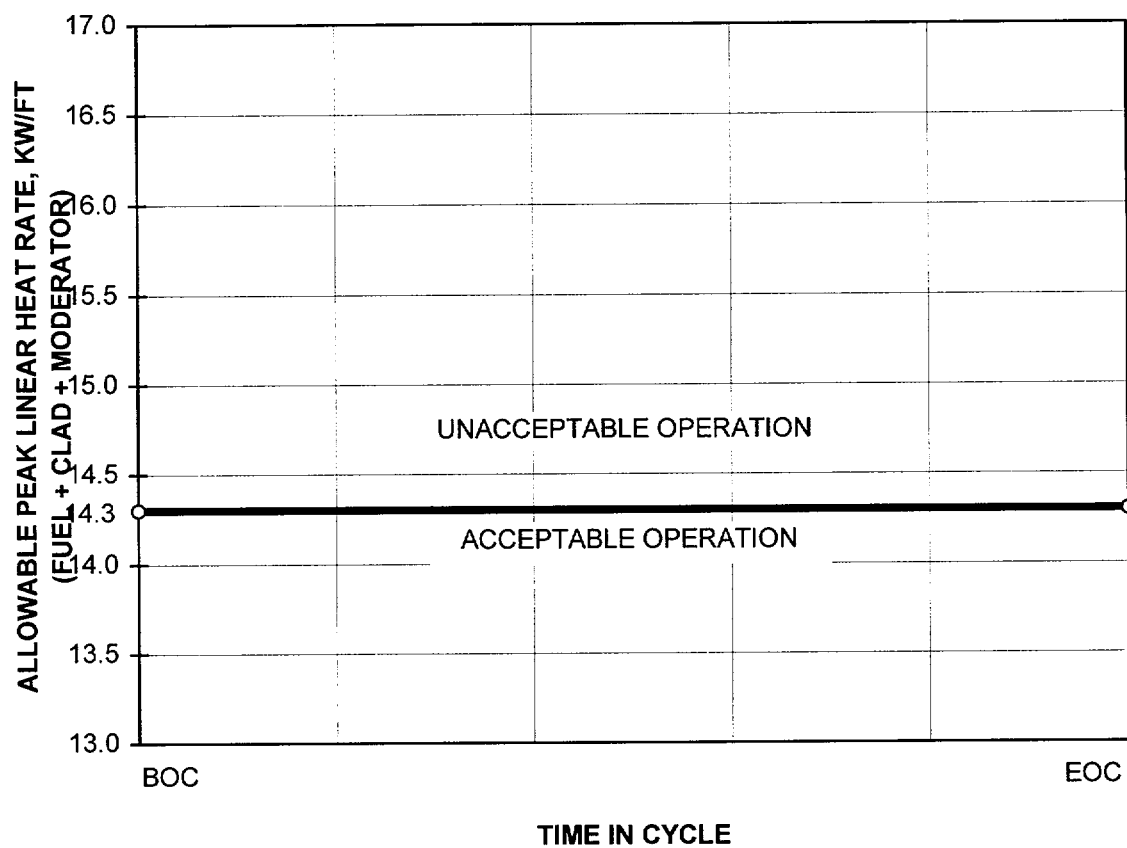
**Figure 3.1.1**  
**Shutdown Margin vs. Time in Cycle**



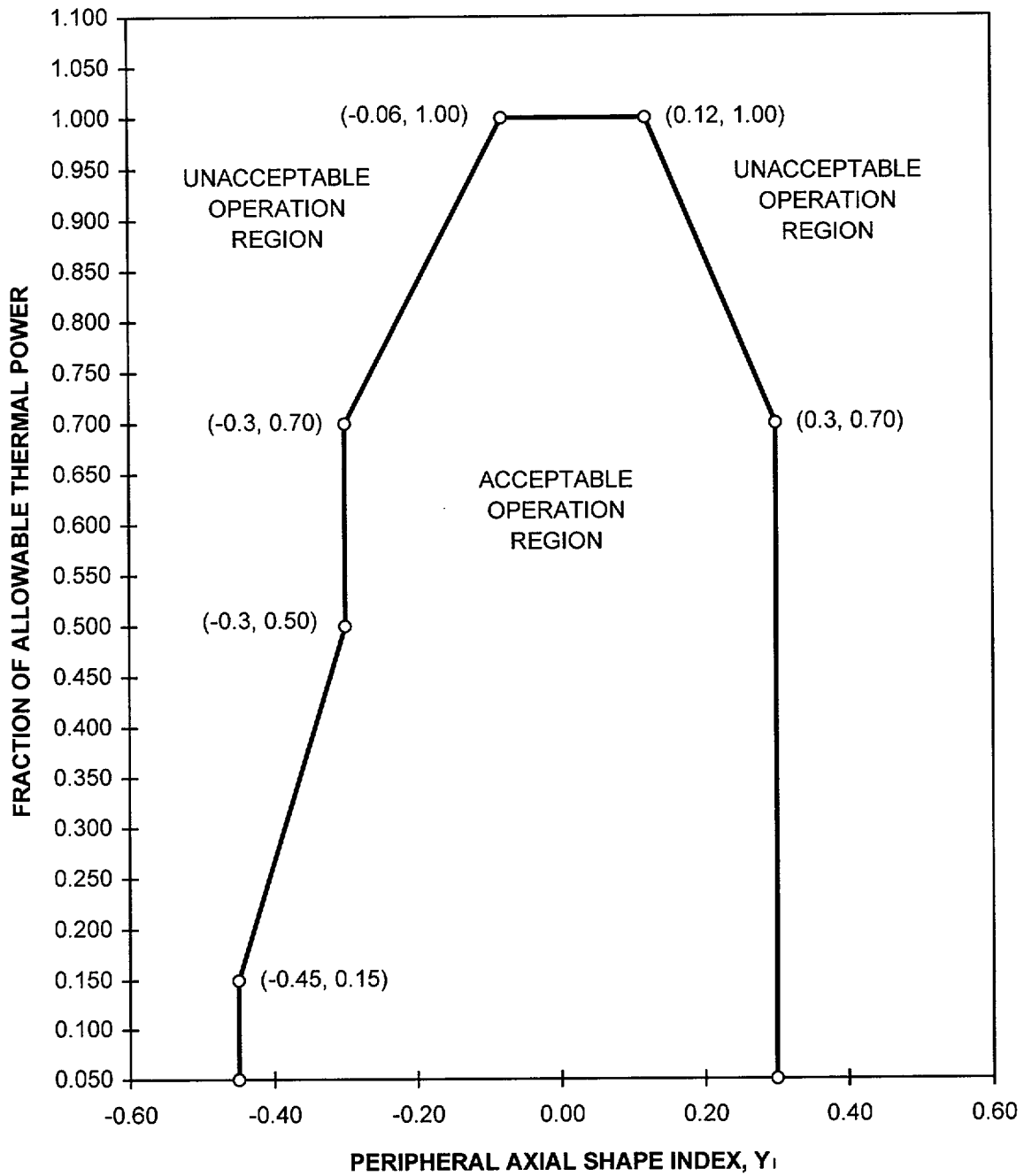
**Figure 3.1.4**  
**Allowable Time to Realign CEA Versus**  
**Initial Total Integrated Radial Peaking Factor ( $F_r^T$ )**



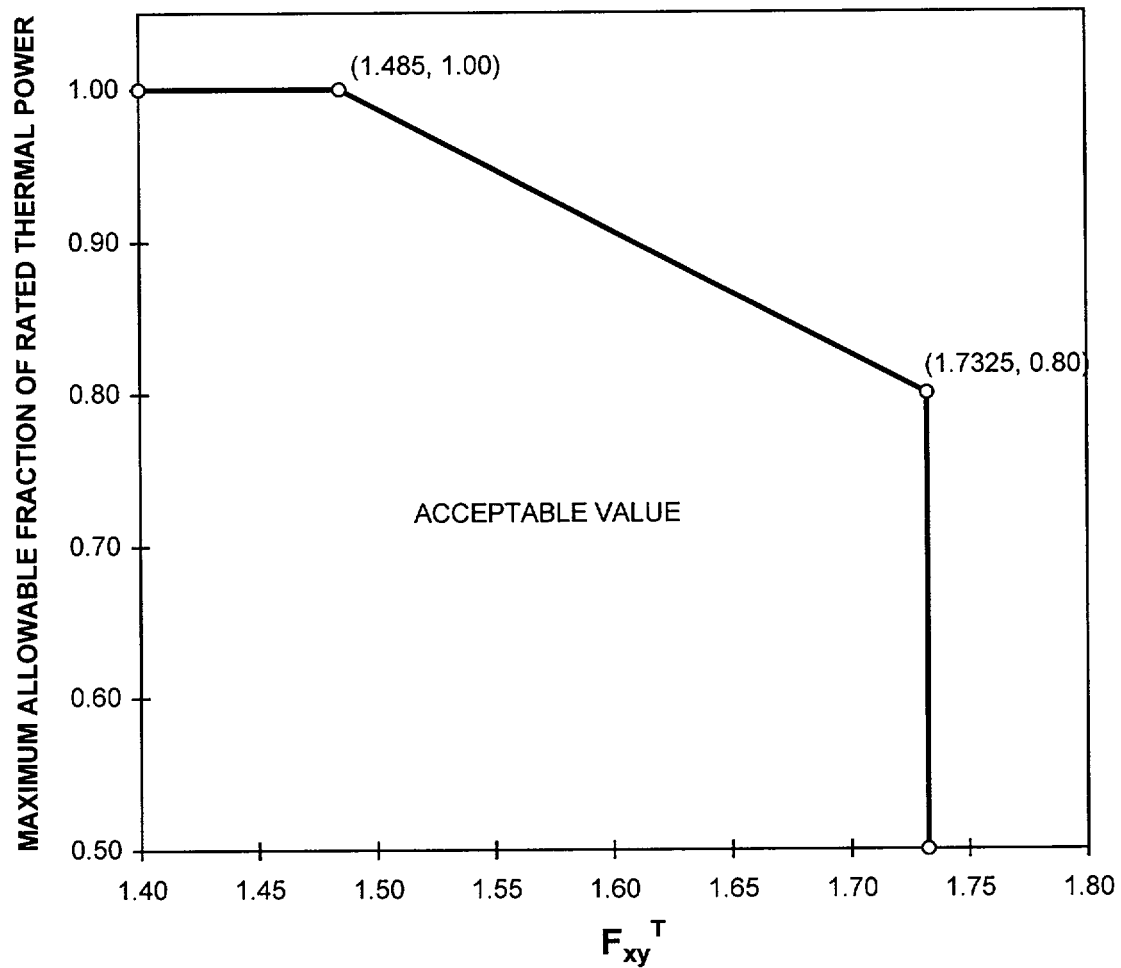
**Figure 3.1.6**  
**CEA Group Insertion Limits vs. Fraction of Rated Thermal Power**



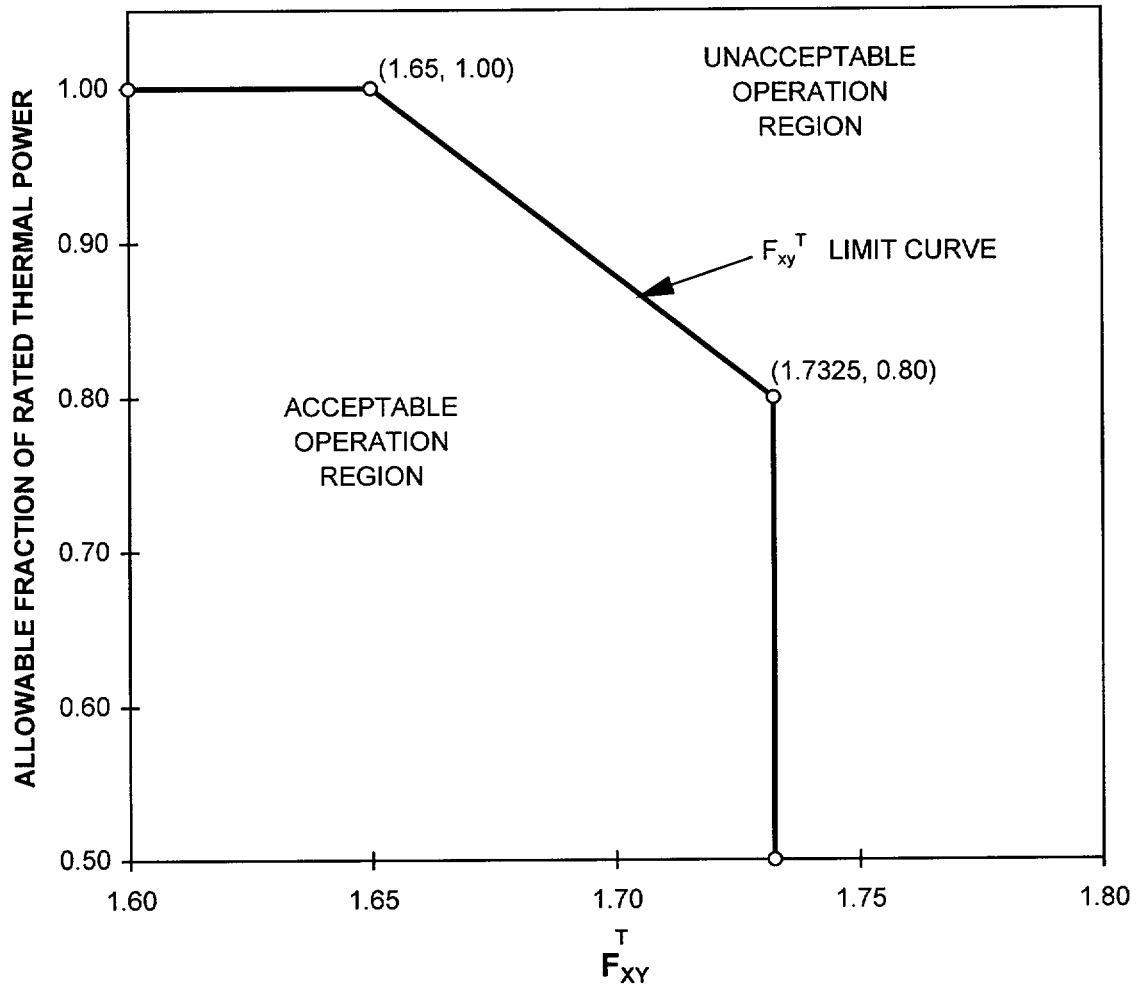
**Figure 3.2.1-1**  
**Allowable Peak Linear Heat Rate vs. Time in Cycle**



**Figure 3.2.1-2**  
**Linear Heat Rate Axial Flux Offset Control Limits**

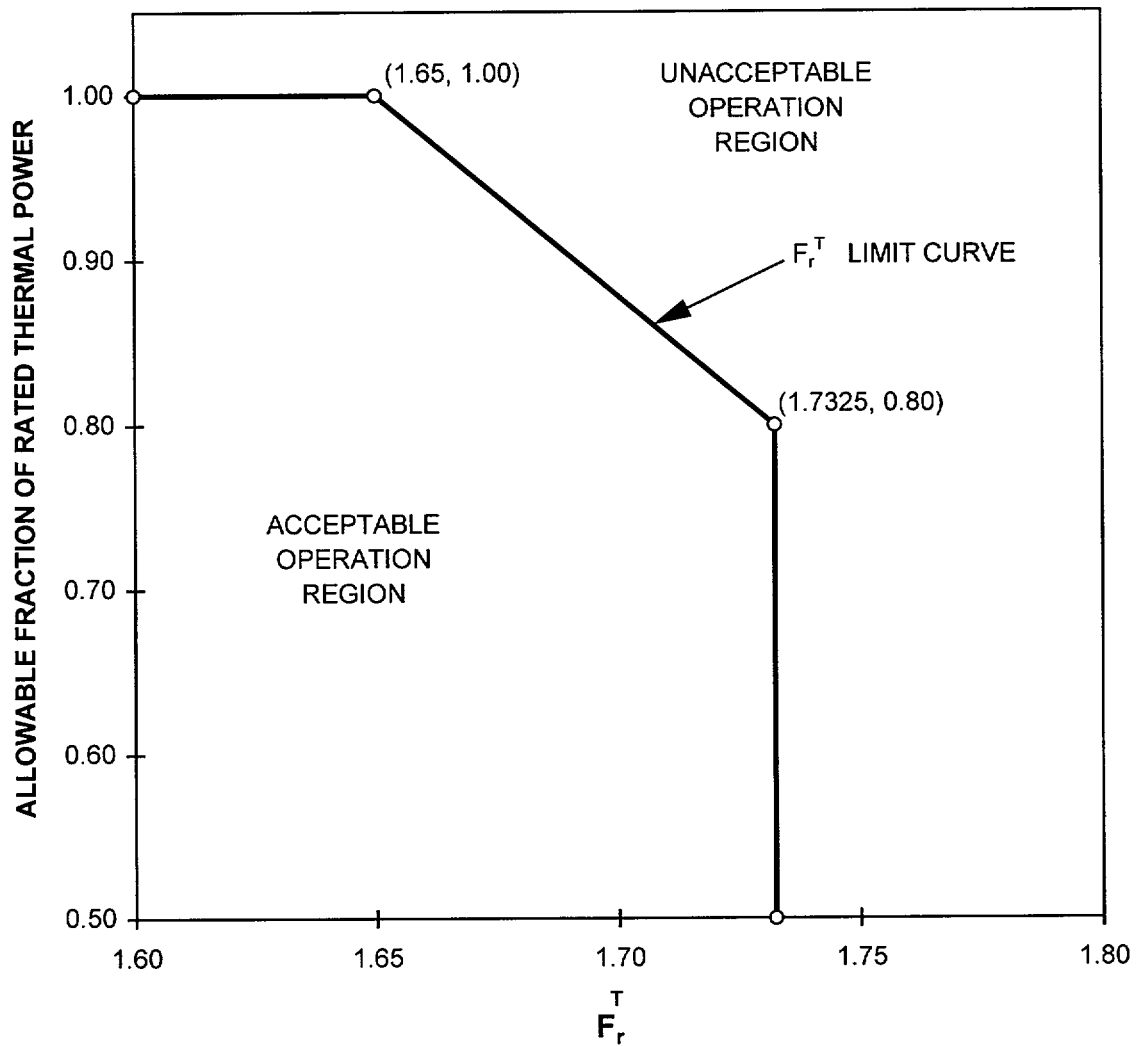


**Figure 3.2.1-3**  
**Total Planar Radial Peaking Factor ( $F_{xy}^T$ ) vs.**  
**Maximum Allowable Fraction of Rated Thermal Power**

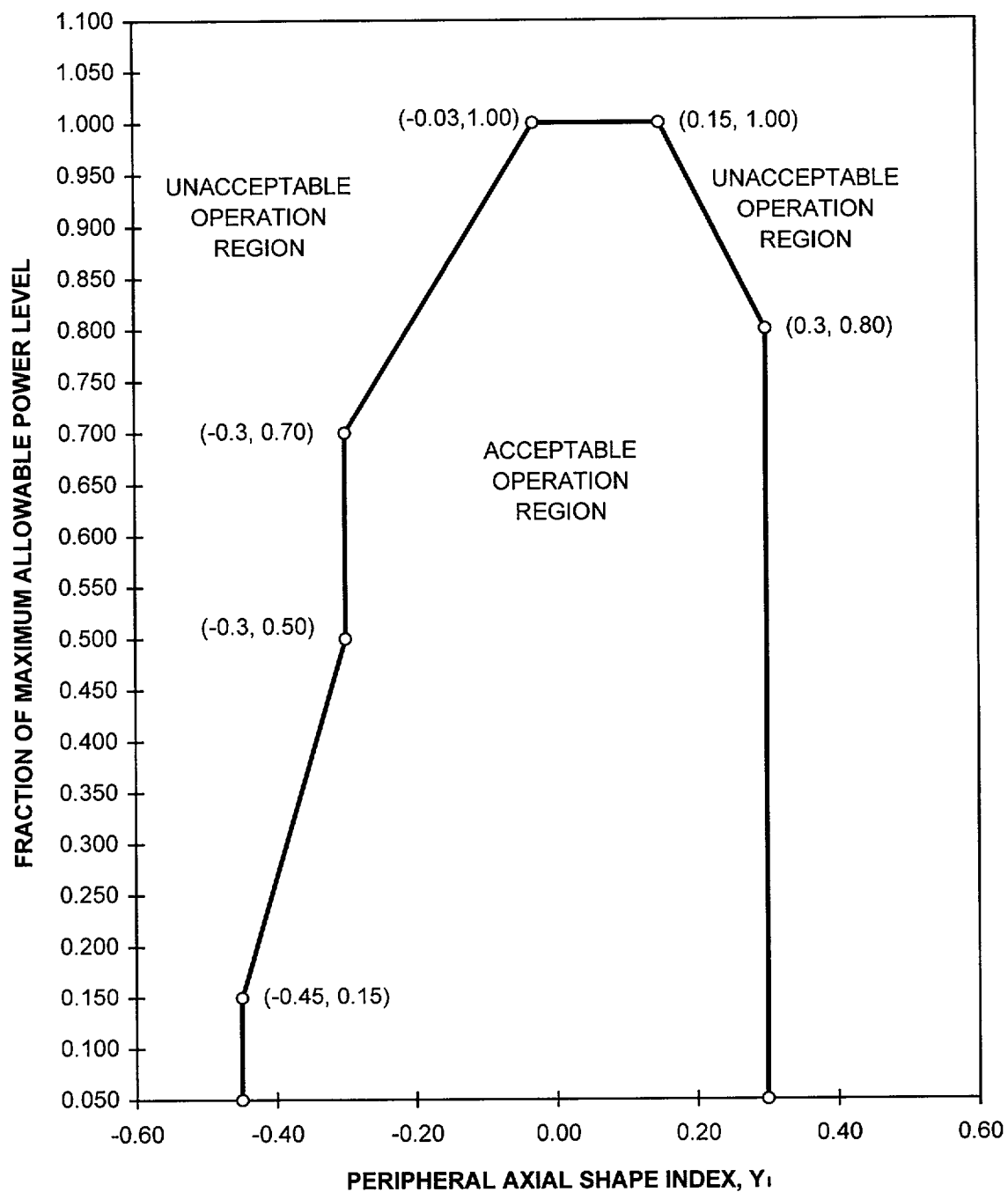


**Figure 3.2.2**

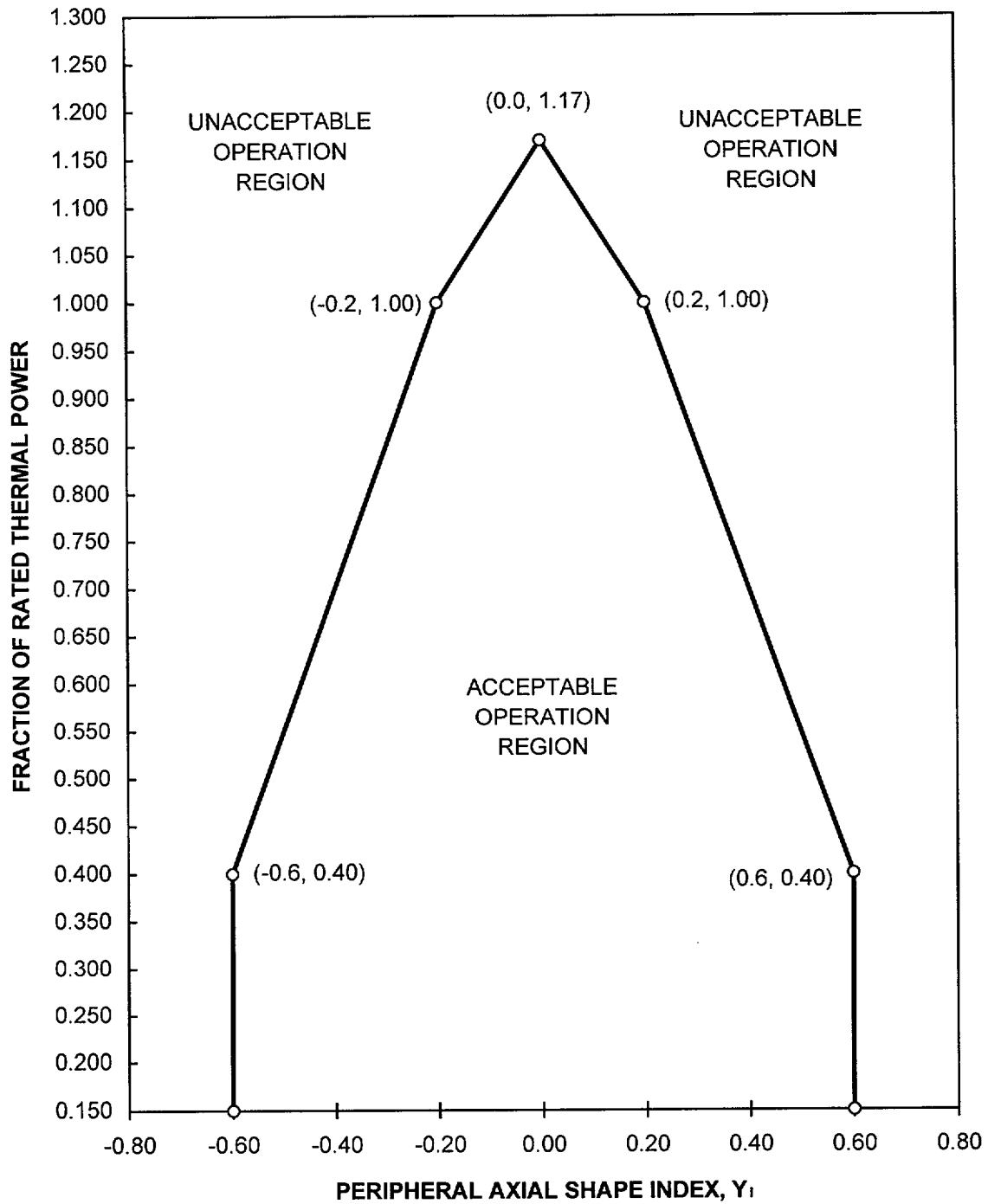
**Total Planar Radial Peaking Factor ( $F_{xy}^T$ ) vs.  
Allowable Fraction of Rated Thermal Power**



**Figure 3.2.3**  
**Total Integrated Radial Peaking Factor ( $F_r^T$ ) vs.**  
**Allowable Fraction of Rated Thermal Power**

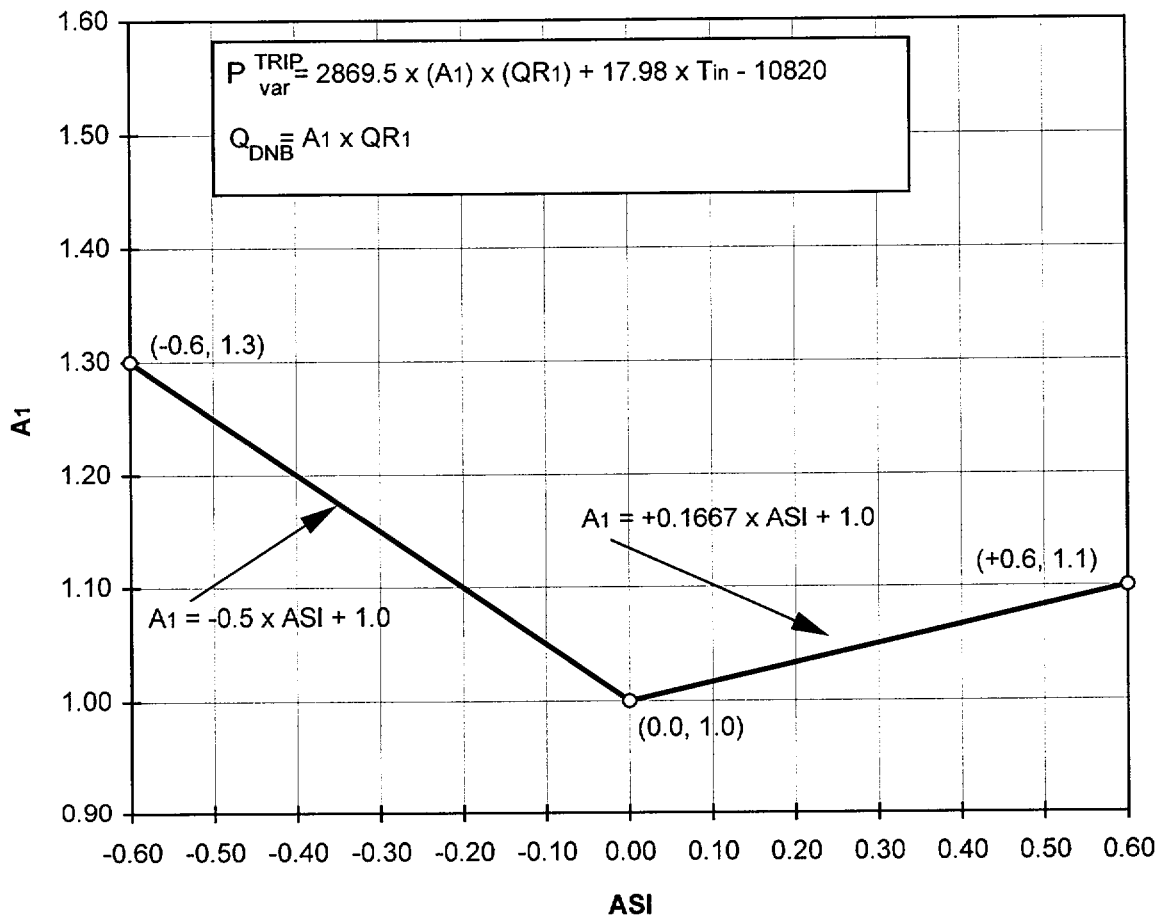


**Figure 3.2.5**  
**DNB Axial Flux Offset Control Limits**

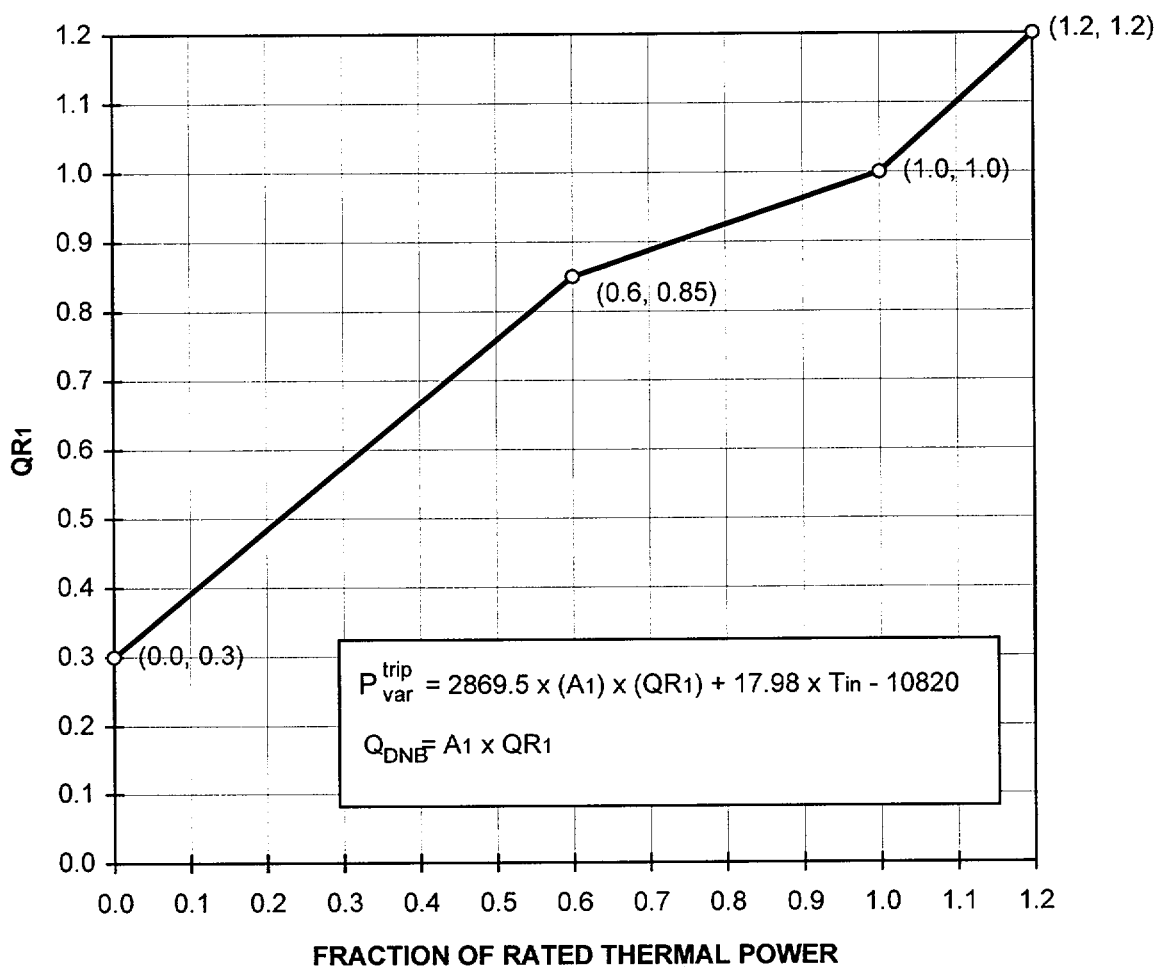


**Figure 3.3.1-1**

**Axial Power Distribution - High Trip Setpoint**  
**Peripheral Axial Shape Index vs. Fraction of Rated Thermal Power**



**Figure 3.3.1-2**  
**Thermal Margin/Low Pressure Trip Setpoint - Part 1**  
**(ASI vs.  $A_1$ )**



**Figure 3.3.1-3**

**Thermal Margin/Low Pressure Trip Setpoint - Part 2  
(Fraction of Rated Thermal Power vs. QR<sub>1</sub>)**

## LIST OF APPROVED METHODOLOGIES

- (1) CENPD-199-P, Rev 1-P-A, "C-E Setpoint Methodology: C-E Local Power Density and DNB LSSS and LCO Setpoint Methodology for Analog Protection Systems," January 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (2) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 1: C-E Calculated Local Power Density and Thermal Margin/Low Pressure LSSS for Calvert Cliffs Units I and II," December 1979 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.2, 3.2.3)
- (3) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 2: Combination of System Parameter Uncertainties in Thermal Margin Analyses for Calvert Cliffs Units 1 and 2," January 1980 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (4) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 3: C-E Calculated Departure from Nucleate Boiling and Linear Heat Rate Limiting Conditions for Operation for Calvert Cliffs Units 1 and 2," March 1980 (Methodology for Specifications 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (5) CEN-191(B)-P, "CETOP-D Code Structure and Modeling Methods for Calvert Cliffs Units 1 and 2," December 1981 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (6) Letter from Mr. D. H. Jaffe (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated June 24, 1982, Unit 1 Cycle 6 License Approval (Amendment No. 71 to DPR-53 and SER) [Approval to CEN-124(B)-P (three parts) and CEN-191(B)-P]
- (7) CEN-348(B)-P, "Extended Statistical Combination of Uncertainties," January 1987 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (8) Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated October 21, 1987, Docket Nos. 50-317 and 50-318, "Safety Evaluation of Topical Report CEN-348(B)-P, Extended Statistical Combination of Uncertainties"
- (9) CENPD-161-P-A, "TORC Code, A Computer Code for Determining the Thermal Margin of a Reactor Core," April 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (10) CENPD-162-P-A, "Critical Heat Flux Correlation of C-E Fuel Assemblies with Standard Spacer Grids Part 1, Uniform Axial Power Distribution," April 1975 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (11) CENPD-207-P-A, "Critical Heat Flux Correlation for C-E Fuel Assemblies with Standard Spacer Grids Part 2, Non-Uniform Axial Power Distribution," December 1984 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (12) CENPD-206-P-A, "TORC Code, Verification and Simplified Modeling Methods," June 1981 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (13) CENPD-225-P-A, "Fuel and Poison Rod Bowing," June 1983 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

- (14) CENPD-266-P-A, "The ROCS and DIT Computer Code for Nuclear Design," April 1983 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (15) CENPD-275-P-A, "C-E Methodology for Core Designs Containing Gadolinia - Urania Burnable Absorbers," May 1988 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (16) CENPD-382-P-A, "Methodology for Core Designs Containing Erbium Burnable Absorbers," August 1993 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (17) CENPD-139-P-A, "C-E Fuel Evaluation Model Topical Report," July 1974 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (18) CEN-161-(B)-P-A, "Improvements to Fuel Evaluation Model," August 1989 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (19) CEN-161-(B)-P, Supplement 1-P, "Improvements to Fuel Evaluation Model," April 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (20) Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated February 4, 1987, Docket Nos. 50-317 and 50-318, "Safety Evaluation of Topical Report CEN-161-(B)-P, Supplement 1-P, Improvements to Fuel Evaluation Model" (Approval of CEN-161(B), Supplement 1-P)
- (21) CEN-372-P-A, "Fuel Rod Maximum Allowable Gas Pressure," May 1990 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (22) Letter from Mr. A. E. Scherer (CE) to Mr. J. R. Miller (NRC) dated December 15, 1981, LD-81-095, Enclosure 1-P, "C-E ECCS Evaluation Model Flow Blockage Analysis" (Methodology for Specifications 3.2.1, 3.2.2)
- (23) CENPD-132, Supplement 3-P-A, "Calculative Methods for the C-E Large Break LOCA Evaluation Model for the Analysis of C-E and W Designed NSSS," June 1985 (Methodology for Specifications 3.2.1, 3.2.2 and approval of Letter LD-81-095, dated December 15, 1981)
- (24) CENPD-133, Supplement 5, "CEFLASH-4A, a FORTRAN77 Digital Computer Program for Reactor Blowdown Analysis," June 1985 (Methodology for Specifications 3.2.1, 3.2.2)
- (25) CENPD-134, Supplement 2, "COMPERC-II, a Program for Emergency Refill-Reflood of the Core," June 1985 (Methodology for Specifications 3.2.1, 3.2.2)
- (26) Letter from Mr. D. M. Crutchfield (NRC) to Mr. A. E. Scherer (CE), dated July 31, 1986, "Safety Evaluation of Combustion Engineering ECCS Large Break Evaluation Model and Acceptance for Referencing of Related Licensing Topical Reports (Approval of CENPD-133, Supplement 5 and CENPD-134, Supplement 2)
- (27) CENPD-135, Supplement 5-P, "STRIKIN-II, A Cylindrical Geometry Fuel Rod Heat Transfer Program," April 1977 (Methodology for Specifications 3.2.1, 3.2.2)

- (28) Letter from Mr. R. L. Baer (NRC) to Mr. A. E. Scherer (CE) dated September 6, 1978, "Evaluation of Topical Report CENPD-135, Supplement 5"
- (29) CENPD-137, Supplement 1-P, "Calculative Methods for the C-E Small Break LOCA Evaluation Model," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)
- (30) CENPD-133, Supplement 3-P, "CEFLASH-4AS, "A Computer Program for the Reactor Blowdown Analysis of the Small Break Loss of Coolant Accident," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)
- (31) Letter from Mr. K. Kniel (NRC) to Mr. A. E. Scherer (CE), dated September 27, 1977, "Evaluation of Topical Reports CENPD-133, Supplement 3-P and CENPD-137, Supplement 1-P"
- (32) CENPD-138, Supplement 2-P, "PARCH, A FORTRAN-IV Digital Program to Evaluate Pool Boiling, Axial Rod and Coolant Heatup," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)
- (33) Letter from Mr. C. Aniel (NRC) to Mr. A. E. Scherer, dated April 10, 1978. "Evaluation of Topical Report CENPD-138, Supplement 2-P"
- (34) Letter from Mr. A. E. Lundvall, Jr. (BG&E) to Mr. J. R. Miller (NRC) dated February 22, 1985, "Calvert Cliffs Nuclear Power Plant Unit 1; Docket No. 50-317, Amendment to Operating License DPR-53, Eighth Cycle License Application" (Section 7.3.2 contains Methodology for Specifications 3.1.1 and 3.1.3 and 3.1.6)
- (35) Letter from Mr. D. H. Jaffe (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated May 20, 1985, "Safety Evaluation Report Approving Unit 1 Cycle 8 License Application"
- (36) Letter from Mr. A. E. Lundvall, Jr. (BG&E) to Mr. R. A. Clark (NRC), dated September 22, 1980, "Amendment to Operating License No. 50-317, Fifth Cycle License Application" (Section 7.1.2 contains Methodology for Specifications 3.1.1, 3.9.1)
- (37) Letter from Mr. R. A. Clark (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated December 12, 1980, "Safety Evaluation Report Approving Unit 1, Cycle 5 License Application"
- (38) Letter from Mr. J. A. Tiernan (BG&E) to Mr. A. C. Thadani (NRC), dated October 1, 1986, "Calvert Cliffs Nuclear Power Plant Unit Nos. 1 & 2, Docket Nos. 50-317 & 50-318, Request for Amendment" (Methodology for Specifications 3.1.4)
- (39) Letter from S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated July 7, 1987, Docket Nos. 50-317 and 50-318, Approval of Amendments 127 (Unit 1) and 109 (Unit 2) (Support for Specification 3.1.4)
- (40) CENPD-188-A, "HERMITE: A Multi-Dimensional Space-Time Kinetics Code for PWR Transients," July 1976 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

- (41) The power distribution monitoring system referenced in various specifications and the BASES, is described in the following documents:
- i. CENPD-153-P, Revision 1-P-A, "Evaluation of Uncertainty in the Nuclear Power Peaking Measured by the Self-Powered, Fixed Incore Detector System," May 1980 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
  - ii. CEN-119(B)-P, "BASSS, Use of the Incore Detector System to Monitor the DNB-LCO on Calvert Cliffs Unit 1 and Unit 2," November 1979 (Referenced in Appendix B of Unit 2 Cycle 9 License Application)
  - iii. Letter from Mr. G. C. Creel (BG&E) to NRC Document Control Desk, dated February 7, 1989, "Calvert Cliffs Nuclear Power Plant Unit No. 2; Docket No. 50-318, Request for Amendment, Unit 2 Ninth Cycle License Application" (Appendix B contains Methodologies for Specifications 3.1.4, 3.2.2, 3.2.3, 3.2.5)
  - iv. Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. G. C. Creel (BG&E), dated January 10, 1990, "Safety Evaluation Report Approving Unit 2 Cycle 9 License Application"
- (42) Letter from Mr. D. G. McDonald, Jr. (NRC) to Mr. R. E. Denton (BGE), dated May 11, 1995, "Approval to Use Convolution Technique in Main Steam Line Break Analysis - Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2 (TAC Nos. M90897 and M90898)" (Methodology for Specification 3.2.3).

# CALVERT CLIFFS NUCLEAR POWER PLANT

## CORE OPERATING LIMITS REPORT

for

UNIT 1, CYCLE 15

REVISION 0

Timothy A. Scheerer 3/17/00

RESPONSIBLE ENGINEER / DATE

J. Janet Blum 3/17/2000

INDEPENDENT REVIEWER / DATE

[Signature] 3/21/2000

PRINCIPAL ENGINEER - NEU / DATE

# CORE OPERATING LIMITS REPORT

## CALVERT CLIFFS UNIT 1, CYCLE 15

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**CORE OPERATING LIMITS REPORT**  
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## INTRODUCTION

This report provides the cycle-specific core operating limits for operation of Calvert Cliffs Unit 1, Cycle 15. It contains the limits for:

- Shutdown Margin (SDM)
- Moderator Temperature Coefficient (MTC)
- Control Element Assembly (CEA) Alignment
- Regulating Control Element Assembly (CEA) Insertion Limits
- Linear Heat Rate (LHR)
- Total Planar Radial Peaking Factor ( $F_{xy}^T$ )
- Total Integrated Radial Peaking Factor ( $F_r^T$ )
- Axial Shape Index (ASI)
- Reactor Protective System (RPS) Instrumentation - Operating
- Boron Concentration

In addition, this report contains a number of figures which provides limits on the parameters listed above. If any of the limits contained in this report are exceeded, corrective action will be taken as defined in the Technical Specifications.

This report has been prepared in accordance with the requirements of Technical Specifications. The cycle specific limits have been developed using the NRC-approved methodologies given in the "List of Approved Methodologies" section of this report and the Technical Specifications.

### COLR Revision 0

COLR Revision 0 was issued to support the Unit 1 Cycle 15 core onload and operation of Unit 1 Cycle 15 in Modes 5 and 6. Safety Evaluation SE00424 Rev. 0000 provided for this revision.

## DEFINITIONS

### Axial Shape Index (ASI)

ASI shall be the power generated in the lower half of the core less the power generated in the upper half of the core, divided by the sum of the power generated in the lower and upper halves of the core.

$$ASI = \frac{\text{lower} - \text{upper}}{\text{lower} + \text{upper}} = Y_E$$

The Axial Shape Index ( $Y_I$ ) used for the trip and pretrip signals in the Reactor Protection System (RPS) is the above value ( $Y_E$ ) modified by an appropriate multiplier (A) and a constant (B) to determine the true core axial power distribution for that channel.

$$Y_I = AY_E + B$$

### Total Integrated Radial Peaking Factor - $F_r^T$

The Total Integrated Radial Peaking Factor is the ratio of the peak pin power to the average pin power in an unrodded core.

### Total Planar Radial Peaking Factor - $F_{xy}^T$

The Total Planar Radial Peaking Factor is the maximum ratio of the peak to average power density of the individual fuel rods in any of the unrodded horizontal planes.

## **CYCLE SPECIFIC LIMITS FOR UNIT 1, CYCLE 15**

### **3.1.1 Shutdown Margin (SDM) (SR 3.1.1.1)**

*T<sub>avg</sub> > 200 °F - Modes 3 and 4:*

( Not Applicable in Modes 5 and 6 )

*T<sub>avg</sub> ≤ 200 °F - Mode 5:*

The shutdown margin shall be  $\geq 3.0\% \Delta\rho$ .

### **3.1.3 Moderator Temperature Coefficient (MTC) (SR 3.1.3.2)**

( Not Applicable in Modes 5 and 6 )

### **3.1.4 Control Element Assembly (CEA) Alignment (Action 3.1.4.B.1)**

( Not Applicable in Modes 5 and 6 )

### **3.1.6 Regulating Control Element Assembly (CEA) Insertion Limits (SR 3.1.6.1 and SR 3.1.6.2)**

( Not Applicable in Modes 5 and 6 )

### **3.2.1 Linear Heat Rate (LHR) (SR 3.2.1.2 and SR 3.2.1.4)**

( Not Applicable in Modes 5 and 6 )

## **CYCLE SPECIFIC LIMITS FOR UNIT 1, CYCLE 15**

The uncertainty factors for the incore detector monitoring system are:

( Not Applicable in Modes 5 and 6 )

### **3.2.2 Total Planar Radial Peaking Factor ( $F_{xy}^T$ ) (SR 3.2.1.1 and SR 3.2.2.1)**

(Not Applicable in Modes 5 and 6)

### **3.2.3 Total Integrated Radial Peaking Factor ( $F_r^T$ ) (SR 3.2.3.1)**

(Not Applicable in Modes 5 and 6)

### **3.2.5 Axial Shape Index (ASI) (SR 3.2.5.1)**

( Not Applicable in Modes 5 and 6 )

### **3.3.1 Reactor Protective System (RPS) Instrumentation - Operating (Reactor Trip Setpoints) (TS Table 3.3.1-1)**

The Thermal Margin/Low Pressure (TM/LP) trip setpoint is given in COLR Figures 3.3.1-2 and 3.3.1-3. The allowable values are to be not less than the larger of (1) 1875 psia or (2) the value calculated from COLR Figures 3.3.1-2 and 3.3.1-3.

Note: The TM/LP trip is not needed in modes 5 and 6. However, the TM/LP trip setpoints are being issued at this time to allow time for the field to implement the new setpoints.

### 3.9.1 Boron Concentration (SR 3.9.1.1)

The refueling boron concentration will maintain the  $K_{eff}$  at 0.95 or less (including a 1%  $\Delta K/K$  conservative allowance for uncertainties. The refueling boron concentration shall be maintained uniform.

#### Refueling Boron Concentration Limits

	56 Credited CEAs
COLR/Tech Spec Limit	2272 ppm
Chemistry Sampling	23 ppm
Boron-10 Depletion	48 ppm
Refueling Boron Concentration Limit including Chemistry Sampling uncertainty and Boron-10 Depletion	$\geq 2343$ ppm
Dilution of the Refueling Pool between Low and High Level Alarms with Refueling Pool Flooded	72 ppm
Additional Contingencies for Batch 1RT LFAs	20 ppm
Temporary Rotations of Fuel Assemblies	20 ppm
Refueling Boron Concentration Administrative Limit	$\geq 2455$ ppm

**( Not Applicable in Modes 5 and 6 )**

**Figure 3.1.1**  
**Shutdown Margin vs. Time in Cycle**

**( Not Applicable in Modes 5 and 6 )**

**Figure 3.1.4**  
**Allowable Time to Realign CEA Versus**  
**Initial Total Integrated Radial Peaking Factor ( $F_r^T$ )**

**( Not Applicable in Modes 5 and 6 )**

**Figure 3.1.6**  
**CEA Group Insertion Limits vs. Fraction of Rated Thermal Power**

**( Not Applicable in Modes 5 and 6 )**

**Figure 3.2.1-1**  
**Allowable Peak Linear Heat Rate vs. Time in Cycle**

**( Not Applicable in Modes 5 and 6)**

**Figure 3.2.1-2**  
**Linear Heat Rate Axial Flux Offset Control Limits**

**( Not Applicable in Modes 5 and 6 )**

**Figure 3.2.1-3**  
**Total Planar Radial Peaking Factor ( $F_{xy}^T$ ) vs.**  
**Maximum Allowable Fraction of Rated Thermal Power**

**( Not Applicable in Modes 5 and 6 )**

**Figure 3.2.2**  
**Total Planar Radial Peaking Factor ( $F_{xy}^T$ ) vs.**  
**Allowable Fraction of Rated Thermal Power**

**( Not Applicable in Modes 5 and 6 )**

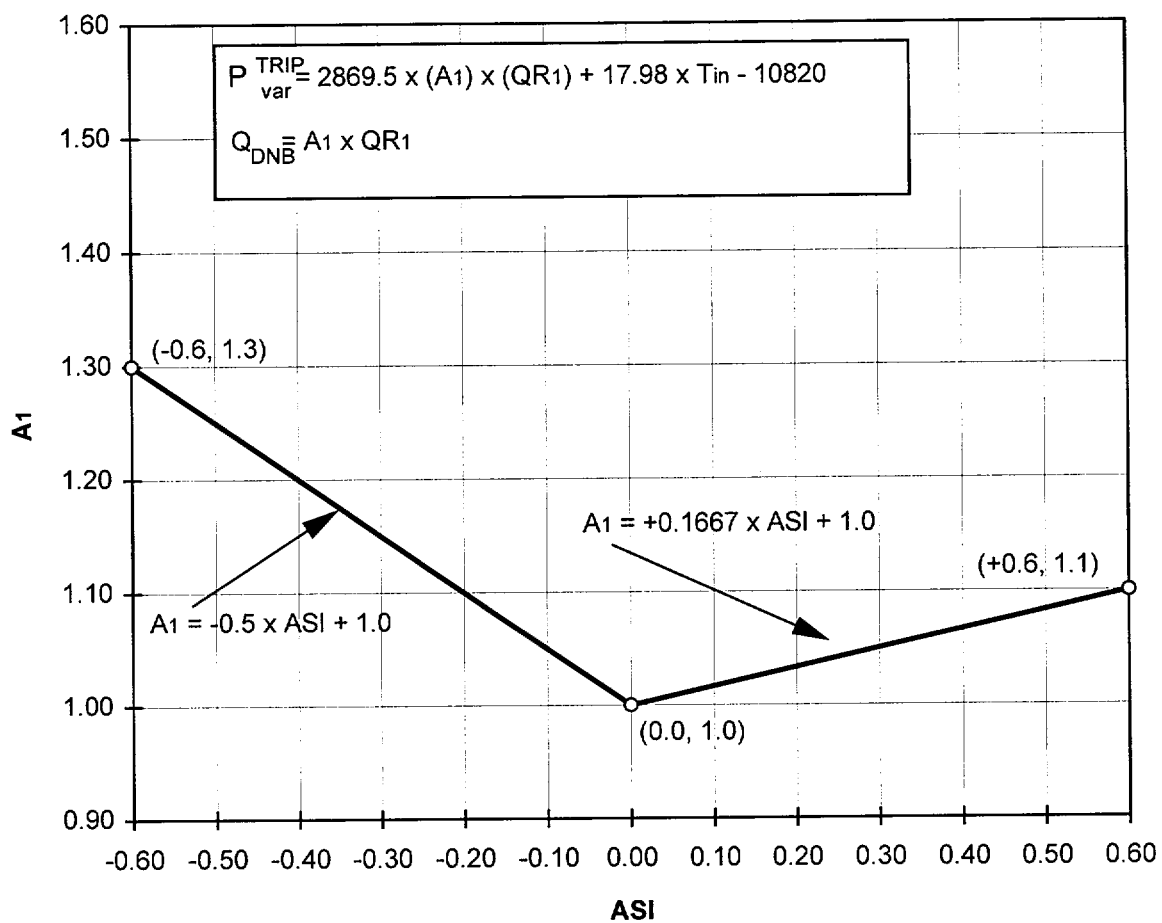
**Figure 3.2.3**  
**Total Integrated Radial Peaking Factor ( $F_r^T$ ) vs.**  
**Allowable Fraction of Rated Thermal Power**

**( Not Applicable in Modes 5 and 6 )**

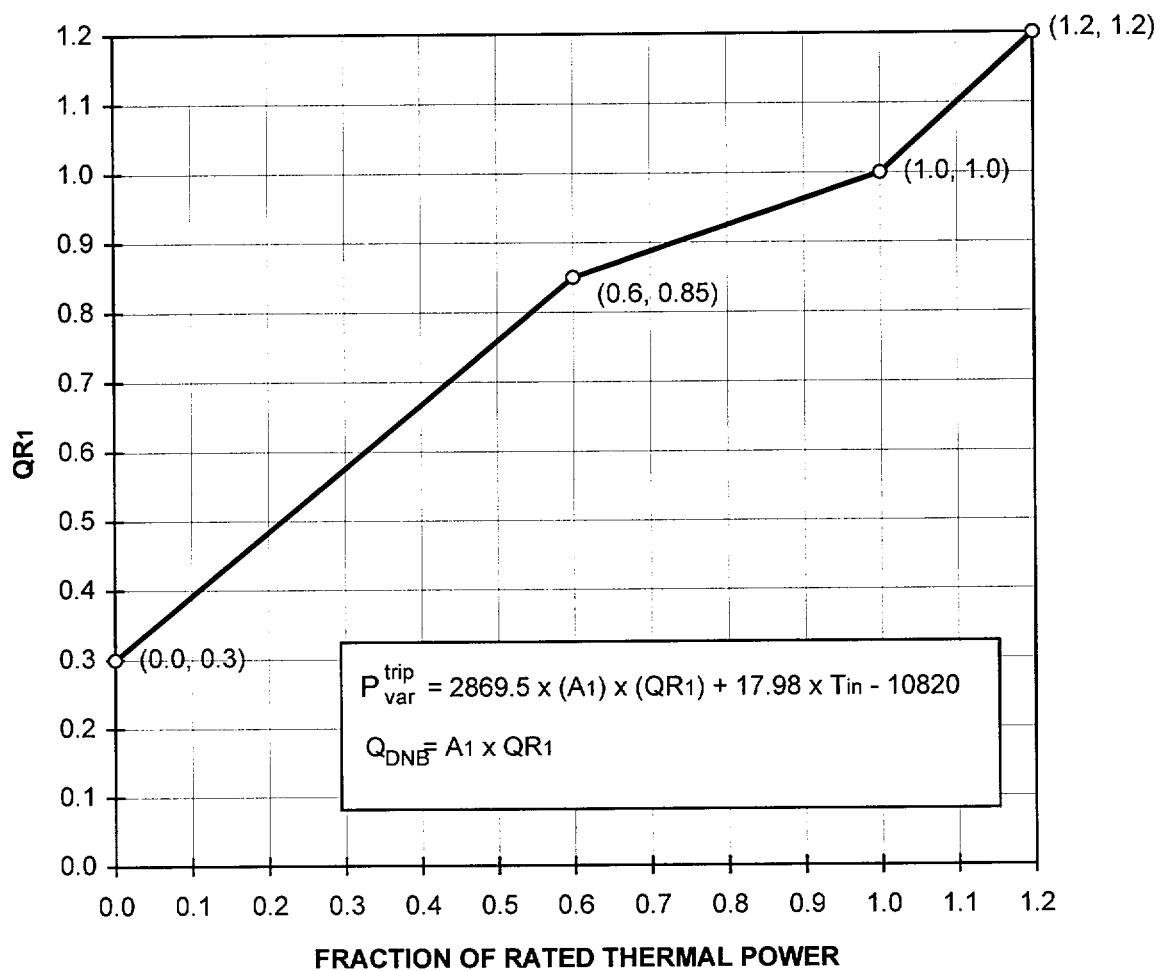
**Figure 3.2.5**  
**DNB Axial Flux Offset Control Limits**

**( Not Applicable in Modes 5 and 6 )**

**Figure 3.3.1-1**  
**Axial Power Distribution - High Trip Setpoint**  
**Peripheral Axial Shape Index vs. Fraction of Rated Thermal Power**



**Figure 3.3.1-2**  
**Thermal Margin/Low Pressure Trip Setpoint - Part 1**  
**(ASI vs.  $A_1$ )**



**Figure 3.3.1-3**  
**Thermal Margin/Low Pressure Trip Setpoint - Part 2**  
**(Fraction of Rated Thermal Power vs. QR<sub>1</sub>)**

## LIST OF APPROVED METHODOLOGIES

- (1) CENPD-199-P, Rev 1-P-A, "C-E Setpoint Methodology: C-E Local Power Density and DNB LSSS and LCO Setpoint Methodology for Analog Protection Systems," January 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (2) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 1: C-E Calculated Local Power Density and Thermal Margin/Low Pressure LSSS for Calvert Cliffs Units I and II," December 1979 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.2, 3.2.3)
- (3) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 2: Combination of System Parameter Uncertainties in Thermal Margin Analyses for Calvert Cliffs Units 1 and 2," January 1980 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (4) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 3: C-E Calculated Departure from Nucleate Boiling and Linear Heat Rate Limiting Conditions for Operation for Calvert Cliffs Units 1 and 2," March 1980 (Methodology for Specifications 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (5) CEN-191(B)-P, "CETOP-D Code Structure and Modeling Methods for Calvert Cliffs Units 1 and 2," December 1981 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (6) Letter from Mr. D. H. Jaffe (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated June 24, 1982, Unit 1 Cycle 6 License Approval (Amendment No. 71 to DPR-53 and SER) [Approval to CEN-124(B)-P (three parts) and CEN-191(B)-P]
- (7) CEN-348(B)-P, "Extended Statistical Combination of Uncertainties," January 1987 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (8) Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated October 21, 1987, Docket Nos. 50-317 and 50-318, "Safety Evaluation of Topical Report CEN-348(B)-P, Extended Statistical Combination of Uncertainties"
- (9) CENPD-161-P-A, "TORC Code, A Computer Code for Determining the Thermal Margin of a Reactor Core," April 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (10) CENPD-162-P-A, "Critical Heat Flux Correlation of C-E Fuel Assemblies with Standard Spacer Grids Part 1, Uniform Axial Power Distribution," April 1975 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (11) CENPD-207-P-A, "Critical Heat Flux Correlation for C-E Fuel Assemblies with Standard Spacer Grids Part 2, Non-Uniform Axial Power Distribution," December 1984 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (12) CENPD-206-P-A, "TORC Code, Verification and Simplified Modeling Methods," June 1981 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)
- (13) CENPD-225-P-A, "Fuel and Poison Rod Bowing," June 1983 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

- (14) CENPD-266-P-A, "The ROCS and DIT Computer Code for Nuclear Design," April 1983 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (15) CENPD-275-P-A, "C-E Methodology for Core Designs Containing Gadolinia - Urania Burnable Absorbers," May 1988 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (16) CENPD-382-P-A, "Methodology for Core Designs Containing Erbium Burnable Absorbers," August 1993 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
- (17) CENPD-139-P-A, "C-E Fuel Evaluation Model Topical Report," July 1974 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (18) CEN-161-(B)-P-A, "Improvements to Fuel Evaluation Model," August 1989 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (19) CEN-161-(B)-P, Supplement 1-P, "Improvements to Fuel Evaluation Model," April 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (20) Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated February 4, 1987, Docket Nos. 50-317 and 50-318, "Safety Evaluation of Topical Report CEN-161-(B)-P, Supplement 1-P, Improvements to Fuel Evaluation Model" (Approval of CEN-161(B), Supplement 1-P)
- (21) CEN-372-P-A, "Fuel Rod Maximum Allowable Gas Pressure," May 1990 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)
- (22) Letter from Mr. A. E. Scherer (CE) to Mr. J. R. Miller (NRC) dated December 15, 1981, LD-81-095, Enclosure 1-P, "C-E ECCS Evaluation Model Flow Blockage Analysis" (Methodology for Specifications 3.2.1, 3.2.2)
- (23) CENPD-132, Supplement 3-P-A, "Calculative Methods for the C-E Large Break LOCA Evaluation Model for the Analysis of C-E and W Designed NSSS," June 1985 (Methodology for Specifications 3.2.1, 3.2.2 and approval of Letter LD-81-095, dated December 15, 1981)
- (24) CENPD-133, Supplement 5, "CEFLASH-4A, a FORTRAN77 Digital Computer Program for Reactor Blowdown Analysis," June 1985 (Methodology for Specifications 3.2.1, 3.2.2)
- (25) CENPD-134, Supplement 2, "COMPERC-II, a Program for Emergency Refill-Reflood of the Core," June 1985 (Methodology for Specifications 3.2.1, 3.2.2)
- (26) Letter from Mr. D. M. Crutchfield (NRC) to Mr. A. E. Scherer (CE), dated July 31, 1986, "Safety Evaluation of Combustion Engineering ECCS Large Break Evaluation Model and Acceptance for Referencing of Related Licensing Topical Reports (Approval of CENPD-133, Supplement 5 and CENPD-134, Supplement 2)
- (27) CENPD-135, Supplement 5-P, "STRIKIN-II, A Cylindrical Geometry Fuel Rod Heat Transfer Program," April 1977 (Methodology for Specifications 3.2.1, 3.2.2)

- (28) Letter from Mr. R. L. Baer (NRC) to Mr. A. E. Scherer (CE) dated September 6, 1978, "Evaluation of Topical Report CENPD-135, Supplement 5"
- (29) CENPD-137, Supplement 1-P, "Calculative Methods for the C-E Small Break LOCA Evaluation Model," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)
- (30) CENPD-133, Supplement 3-P, "CEFLASH-4AS, "A Computer Program for the Reactor Blowdown Analysis of the Small Break Loss of Coolant Accident," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)
- (31) Letter from Mr. K. Kniel (NRC) to Mr. A. E. Scherer (CE), dated September 27, 1977, "Evaluation of Topical Reports CENPD-133, Supplement 3-P and CENPD-137, Supplement 1-P"
- (32) CENPD-138, Supplement 2-P, "PARCH, A FORTRAN-IV Digital Program to Evaluate Pool Boiling, Axial Rod and Coolant Heatup," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)
- (33) Letter from Mr. C. Aniel (NRC) to Mr. A. E. Scherer, dated April 10, 1978. "Evaluation of Topical Report CENPD-138, Supplement 2-P"
- (34) Letter from Mr. A. E. Lundvall, Jr. (BG&E) to Mr. J. R. Miller (NRC) dated February 22, 1985, "Calvert Cliffs Nuclear Power Plant Unit 1; Docket No. 50-317, Amendment to Operating License DPR-53, Eighth Cycle License Application" (Section 7.3.2 contains Methodology for Specifications 3.1.1 and 3.1.3 and 3.1.6)
- (35) Letter from Mr. D. H. Jaffe (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated May 20, 1985, "Safety Evaluation Report Approving Unit 1 Cycle 8 License Application"
- (36) Letter from Mr. A. E. Lundvall, Jr. (BG&E) to Mr. R. A. Clark (NRC), dated September 22, 1980, "Amendment to Operating License No. 50-317, Fifth Cycle License Application" (Section 7.1.2 contains Methodology for Specifications 3.1.1, 3.9.1)
- (37) Letter from Mr. R. A. Clark (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated December 12, 1980, "Safety Evaluation Report Approving Unit 1, Cycle 5 License Application"
- (38) Letter from Mr. J. A. Tiernan (BG&E) to Mr. A. C. Thadani (NRC), dated October 1, 1986, "Calvert Cliffs Nuclear Power Plant Unit Nos. 1 & 2, Docket Nos. 50-317 & 50-318, Request for Amendment" (Methodology for Specifications 3.1.4)
- (39) Letter from S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated July 7, 1987, Docket Nos. 50-317 and 50-318, Approval of Amendments 127 (Unit 1) and 109 (Unit 2) (Support for Specification 3.1.4)
- (40) CENPD-188-A, "HERMITE: A Multi-Dimensional Space-Time Kinetics Code for PWR Transients," July 1976 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

- (41) The power distribution monitoring system referenced in various specifications and the BASES, is described in the following documents:
- i. CENPD-153-P, Revision 1-P-A, "Evaluation of Uncertainty in the Nuclear Power Peaking Measured by the Self-Powered, Fixed Incore Detector System," May 1980 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)
  - ii. CEN-119(B)-P, "BASSS, Use of the Incore Detector System to Monitor the DNB-LCO on Calvert Cliffs Unit 1 and Unit 2," November 1979 (Referenced in Appendix B of Unit 2 Cycle 9 License Application)
  - iii. Letter from Mr. G. C. Creel (BG&E) to NRC Document Control Desk, dated February 7, 1989, "Calvert Cliffs Nuclear Power Plant Unit No. 2; Docket No. 50-318, Request for Amendment, Unit 2 Ninth Cycle License Application" (Appendix B contains Methodologies for Specifications 3.1.4, 3.2.2, 3.2.3, 3.2.5)
  - iv. Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. G. C. Creel (BG&E), dated January 10, 1990, "Safety Evaluation Report Approving Unit 2 Cycle 9 License Application"
- (42) Letter from Mr. D. G. McDonald, Jr. (NRC) to Mr. R. E. Denton (BGE), dated May 11, 1995, "Approval to Use Convolution Technique in Main Steam Line Break Analysis - Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2 (TAC Nos. M90897 and M90898)" (Methodology for Specification 3.2.3).