VIRGINIA ELECTRIC AND POWER COMPANY Richmond, Virginia 23261 May 8, 2000

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555-0001 Serial No. 00-253 SPS Lic/JSA R0 Docket Nos. 50-280 50-281 License Nos. DPR-32 DPR-37

Gentlemen:

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VIRGINIA ELECTRIC AND POWER COMPANY SURRY POWER STATION UNITS 1 AND 2 MONTHLY OPERATING REPORT

The Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of April 2000 is provided in the attachment.

If you have any questions or require additional information, please contact us.

Very truly yours,

E. S. Grecheck, Site Vice President Surry Power Station

Attachment

Commitments made by this letter: None

cc: United States Nuclear Regulatory Commission Region II Atlanta Federal Center 61 Forsyth Street, SW, Suite 23 T85 Atlanta, Georgia 30303-8931

> Mr. R. A. Musser NRC Senior Resident Inspector Surry Power Station

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VIRGINIA ELECTRIC AND POWER COMPANY SURRY POWER STATION MONTHLY OPERATING REPORT REPORT NO. 00-04

Approved:

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5/8/2000 Date Site Vice President

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OPERATING DATA REPORT

Docket No.: 50-280 Date: 05/02/00 Completed By: R. Stief Telephone: (757) 365-2486

1. 2. 3. 4. 5. 6.	Unit Name: Reporting Period: Licensed Thermal Power (MWt): Nameplate Rating (Gross MWe): Design Electrical Rating (Net MWe): Maximum Dependable Capacity (Gross MWe):	Surry Unit 1 April 2000 2546 847.5 788 840
6.	Maximum Dependable Capacity (Gross MWe):	840
7.	Maximum Dependable Capacity (Net MWe):	801

If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: 8.

Power Level To Which Restricted, If Any (Net MWe): 9.

10. Reasons For Restrictions, If Any:

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		This Month	Year-To-Date	Cumulative
11.	Hours in Reporting Period	719.0	2903.0	239807.0
12.	Hours Reactor Was Critical	360.0	2544.0	172620.5
13.	Reactor Reserve Shutdown Hours	0.0	0.0	3774.5
14.	Hours Generator On-Line	359.6	2543.6	170075.0
15.	Unit Reserve Shutdown Hours	0.0	0.0	3736.2
16.	Gross Thermal Energy Generated (MWH)	718827.5	6112966.6	402553215.0
17.	Gross Electrical Energy Generated (MWH)	239246.0	2032922.0	132178255.0
18.	Net Electrical Energy Generated (MWH)	229116.0	1961069.0	126063872.0
19.	Unit Service Factor	50.0%	87.6%	70.9%
20.	Unit Availability Factor	50.0%	87.6%	72.5%
21.	Unit Capacity Factor (Using MDC Net)	39.8%	84.3%	67.4%
22.	Unit Capacity Factor (Using DER Net)	40.4%	85.7%	66.7%
23.	Unit Forced Outage Rate	0.0%	0.0%	13.6%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Type and duration of scheduled shutdowns are no longer provided.	
[Reference: Letter S/N 00-069, dated February 7, 2000]	

If Shut Down at End of Report Period, Estimated Date of Start-up: 25.

Estimated start-up dates are no longer provided. [Reference: Letter S/N 00-069, dated February 7, 2000]

26. Unit In Test Status (Prior to Commercial Operation):

FORECAST	ACHIEVED

INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION

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OPERATING DATA REPORT

 Docket No.:
 50-281

 Date:
 05/02/00

 Completed By:
 R. Stief

 Telephone:
 (757) 365-2486

1. 2. 3. 4. 5.	Unit Name: Reporting Period: Licensed Thermal Power (MWt): Nameplate Rating (Gross MWe): Design Electrical Rating (Net MWe): Maximum Dependable Canacity (Gross MWo):	Surry Unit 2 April 2000 2546 847.5 788 840
6.	Maximum Dependable Capacity (Gross MWe):	840
7.	Maximum Dependable Capacity (Net MWe):	801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

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		This Month	Year-To-Date	Cumulative
11.	Hours in Reporting Period	719.0	2903.0	236688.0
12.	Hours Reactor Was Critical	719.0	2903.0	170432.1
13.	Reactor Reserve Shutdown Hours	0.0	0.0	328.1
14.	Hours Generator On-Line	719.0	2903.0	168291.4
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	1832631.2	7389979.0	399417972.2
17.	Gross Electrical Energy Generated (MWH)	617455.0	2490059.0	131122957.0
18.	Net Electrical Energy Generated (MWH)	596183.0	2406145.0	125103002.0
19.	Unit Service Factor	100.0%	100.0%	71.1%
20.	Unit Availability Factor	100.0%	100.0%	71.1%
21.	Unit Capacity Factor (Using MDC Net)	103.5%	103.5%	67.5%
22.	Unit Capacity Factor (Using DER Net)	105.2%	105.2%	67.1%
23.	Unit Forced Outage Rate	0.0%	0.0%	10.8%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

October 2000	-
Type and duration of scheduled shutdowns are no longer provided.	
[Reference: Letter S/N 00-069, dated February 7, 2000]	_

25. If Shut Down at End of Report Period, Estimated Date of Start-up:

Estimated start-up dates are no longer provided. [Reference: Letter S/N 00-069, dated February 7, 2000]

26. Unit In Test Status (Prior to Commercial Operation):

FORECAST	ACHIEVED

INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION

Surry Monthly Operating Report No. 00-04 Page 5 of 18

UNIT SHUTDOWN AND POWER REDUCTION (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: April 2000

Docket No.:	50-280
Unit Name:	Surry Unit 1
Date:	05/02/00
Completed by:	R. Stief
Telephone:	(757) 365-2486

	(1)		(2)	(3)		(4)	(5)	
Date	Туре	Duration Hours	Reason	Method of Shutting Down Rx	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
04/16/00	s	359H 36M	С	1	N/A	N/A	N/A	Refueling Outage

F: S:	(1) Forced Scheduled	(2) REASON: A - Equipment Failure (Explain) B - Maintenance or Test C - Refueling D - Regulatory Restriction E - Operator Training & Licensing Examination	1 2 3	-	(3) HOD: Manual Manual Scram Automatic Scram Other (Explain)
		F - Administrative G - Operational Error (Explain)			

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(4) Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5) Exhibit 1 - Same Source

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UNIT SHUTDOWN AND POWER REDUCTION (EQUAL TO OR GREATER THAN 20%)

REPORT MONTH: April 2000

Docket No.: 50-281 Unit Name: Surry Unit 2 Date: 05/02/00 Completed by: R. Stief Telephone: (757) 365-2486

None during the Reporting Period

F: S:	(1) Forced Scheduled	 (2) REASON: A - Equipment Failure (Explain) B - Maintenance or Test C - Refueling D - Regulatory Restriction E - Operator Training & Licensing Examination F - Administrative G - Operational Error (Explain) 	(3) METHOD: 1 - Manual 2 - Manual Scram 3 - Automatic Scram 4 - Other (Explain)
		G - Operational Error (Explain)	

(4)

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Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5) Exhibit 1 - Same Source

AVERAGE DAILY UNIT POWER LEVEL

Docket No .:	50-280
Unit Name:	Surry Unit 1
Date:	05/02/00
Completed by:	R. Stief
Telephone:	(757) 365-2486

MONTH: April 2000

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	681	17	0
2	648	18	0
3	669	19	0
4	665	20	0
5	660	21	0
6	655	22	0
7	646	23	0
8	642	24	0
9	638	25	0
10	630	26	0
11	629	27	0
12	622	28	0
13	611	29	0
14	615	30	0
15	536		
16	1		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

50-281
Surry Unit 2
)5/02/00
R. Stief
757) 365-2486

MONTH: April 2000

Day	Average Daily Power Level (MWe - Net)	Day	Average Daily Power Level (MWe - Net)
1	830	17	828
2	796	18	829
3	824	19	828
4	829	20	828
5	829	21	828
6	828	22	829
7	829	23	830
8	829	24	831
9	829	25	831
10	829	26	830
11	829	27	830
12	830	28	831
13	834	29	831
14	830	30	830
15	829		
16	822		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

SUMMARY OF OPERATING EXPERIENCE

MONTH/YEAR: April 2000

The following chronological sequence by unit is a summary of operating experiences for this month that required load reductions or resulted in significant non-load related incidents.

UNIT ONE:

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04/01/00	0000	Unit started the month at 83.7% / 710 MWe. Unit on coastdown for Spring Refueling Outage.
04/16/00	0037	Unit offline.
04/16/00	0059	Reactor manually tripped.
04/30/00	2400	Unit finished the month at 0% / 0 MWe.

UNIT TWO:

04/01/00	0000	Unit started the month at 100% / 861 MWe.
04/30/00	2400	Unit finished the month at 100% / 861 MWe.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

The second

MONTH/YEAR: April 2000

DCP 96-041 FS 97-020	Design Change Package UFSAR Change Request (Safety Evaluation 97-048 Rev. 1)	03/30/00
	Design Change Package 96-041, "CW 96" Rubber Expansion Joint Spray S 1&2", replaces the existing Circulating Water expansion joint carbon steel spray stainless steel spray shields. Revision 1 of Safety Evaluation 97-048 allows th gauge sheet to lower the stress to 72% of the maximum allowable if subjected to of 25 psi.	shields with the use of 12
FS 00-011	UFSAR Change Request (Safety Evaluation 00-039)	03/30/00
	As a result of the Integrated Configuration Management Project review, UFS Request FS 00-011 contains corrections and clarifications to the UFSAR s discuss Surry's Instrument Air System. They include clarification of compone correct description of components, and more accurate reflection of current de changes are to enhance accuracy and do not affect any Instrument Air system of or any of its component's operation or performance.	ections that nt activities, esign. These
ET S-00-0057	Engineering Transmittal (Safety Evaluation 00-040)	03/30/00
	Engineering Transmittal S-00-0057 allows the Fire Protection compensatory act Doors 1-BS-DR-46 and 2-BS-DR-50 to be revised to change from a continuous placement of the doors on the CO_2 blowoff chains and an hourly fire watch.	
DCP 99-097 FS 00-017	Design Change Package UFSAR Change Request (Safety Evaluation 00-043)	04/06/00
	Design Change Package 99-097, "Reactor Cavity Water Seal/Surry 1&2", r existing Inflatable Cavity Seal Ring with a new Segmented Reactor Cavity Seal.	eplaces the
TM S1-00-010	Temporary Modification (Safety Evaluation 00-044)	04/13/00
	Temporary Modification S1-00-010 installs additional pressure gauges to the System in order to obtain additional data to trouble shoot head/flow problems.	e Charging
FS 00-022	UFSAR Change Request (Safety Evaluation 00-045)	04/13/00
	As a result of the Integrated Configuration Management Project review, UFS Request FS 00-022 corrects the statements in the UFSAR that discuss Su coolant system cold shutdown low level alarm setpoint and the auxiliary feedwa indication accuracy for normal operating conditions. These changes are accuracy and do not involve any physical changes to the facility or any of its or operation or performance.	rry's reactor ter flow loop to enhance

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: April 2000

SE 00-046	Safety Evaluation	04/13/00
	Safety Evaluation 00-046 evaluates the 2000 Unit 1 Refueling Outage aga industry guidelines and Virginia power administrative requirements for shu management including NUMARC 91-06 and Nuclear Engineering Technical I Rev. 3. The schedule provides a plan for safely and efficiently accomplishin maintenance, modification, and testing activities.	utdown risk Report 865,
NE-1234	Reload Safety Evaluation Technical Report (Safety Evaluations 00-049)	04/13/00
	Technical Report NE-1234, Rev. 0, "Reload Safety Evaluation Surry 1 Cycle 17 refueling and operation of Surry Unit 1 Cycle 17 including using 135.5 inch burn (BP) rods and replacement removable top nozzles on twenty-nine fuel assemblie	able poison
TM S1-00-011	Temporary Modification (Safety Evaluation 00-052)	04/17/00
	Unit shutdown requires the 1A Main Feedwater Pump (MFWP) motor breakers t in the test position and closed in order to open MFWP discharge MOV 1-FW and recirc valve 1-FW-FCV-150A. This allows the Steam Generators to be condensate pump. When the Unit 1 "A" MFWP motor breakers were being cl- valve 1-FW-FCV-150A had failed closed and the MFWP breakers tripped and remain closed. Temporary Modification (TM) S1-00-011 disabled the low flow valve position trips for the MFWP breakers enabling the MFWP breakers to rem in the test position. This TM will be removed and the recirc valve repaired prior to the pump following the current refueling outage.	-MOV-150A fed by the osed, recirc d would not and recirc main closed
ET S-00-0050	Engineering Transmittal (Safety Evaluation 00-053)	04/19/00
	Engineering Transmittal S-00-0050 directs the mounting of a man-lift on top of Crane Trolley for the inspection, and repairing if needed, of the Containment plating. Safety Evaluation 00-053 evaluates the mounting of the lift.	
DCP 00-016 FS 00-029	Design Change Package UFSAR Change Request (Safety Evaluation 00-054)	04/20/00
	Design Change Package 00-016, "Replacement of 1-BC-TCV-101 and 2-BC-TC Manual Valves", removes the existing malfunctioning temperature control valves the Bearing Cooling System and replaces them with manual globe valves that flow operation.	(TCV) from
TM S2-00-002	Temporary Modification (Safety Evaluation 00-058)	04/24/00
	Temporary Modification S2-00-002 installs jumpers to replace the failed relay A Train "B" Auto Stop Oil in the Unit 2 train "B" reactor protection circuit.	ST1-XB for

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FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: April 2000

FS 00-007	UFSAR Change Request (Safety Evaluation 00-060)	04/27/00
	As a result of the Integrated Configuration Management Project Request FS 00-007 corrects the statements in the UFSAR that dis lines. These changes are to enhance accuracy and do not affer systems or structures, or any of its component's operation or perfor	scuss Surry's high energy act any high energy line
TM S1-00-015	Temporary Modification (Safety Evaluation 00-062)	04/28/00
	Temporary Modification S1-00-015 allows the installation of a temp the operation of the intrusion and tamper alarms for a security zon can be replaced.	
TM S1-00-014	Temporary Modification (Safety Evaluation 00-063)	04/30/00
	Temporary Modification S1-00-014 installs jumpers to replace the L 424-XB for Reactor Coolant Flow Loop-2 in the Unit 1 train "B" reac	

PROCEDURE OR METHOD OF OPERATION CHANGES THAT DID NOT REQUIRE NRC APPROVAL MONTH/YEAR: April 2000				
	MONTH TEAR. April 2000			
1-MOP-EP-401 1-MOP-EP-402	Maintenance Operating Procedures (Safety Evaluation 00-038)	03/30/00		
	Maintenance Operating Procedures 1-MOP-EP-401, "Unit 1 Generator Tago EP-402, "Return to Service of Unit 1 Main Generator", were revised Robertshaw Fire Detection alarm for the Low Pressure CO_2 Fire Supress Hazard Zones 15,13,11 due to maintenance work on the turbine and gener Refueling Outage.	to bypass the ion systems in		
1-MOP-DG-002	Maintenance Operating Procedure (Safety Evaluation 00-041)	03/31/00		
	Maintenance Operating Procedure 1-MOP-DG-002, "Removal and Return to PDTT Inlet PCV", was written to fail the Primary Drain Transfer Tank (P Control Valve, 1-DG-PCV-100, in the open position in order to provide a flor loop drains and other drains to the PDTT during the Refueling Outage.	DTT) Pressure		
1-OPT-CT-210 2-OPT-CT-210	Operations Periodic Test Procedures (Safety Evaluation 00-042)	04/06/00		
	Operations Periodic Test Procedures 1 & 2-OPT-CT-210, "Refueling Contain were revised to allow certain manual valves, used to provide refueling contain to be secured closed by the use of substitute locks or tamper seals inste tagging which increases radiation exposure.	nment integrity,		
0-MPM-0300-01	Mechanical Preventive Maintenance Procedure (Safety Evaluation 00-047)	04/13/00		
	Mechanical Preventive Maintenance Procedure 0-MPM-0300-01, "Limitorque SB, SBD, SMB, and HBC Lubrication and Inspection", was revised to provide controls to assure the capability to meet design basis for non-essential Servi isolation safety function when one of the valves is taken out of service maintenance. Failure of the out of service valve to close in response to a no isolation signal is addressed by 0-AP-12.01 and isolation can be accompli required 1 hour via manual isolation valves.	e administrative ice Water (SW) e for testing or n-essential SW		
GMP-011	General Maintenance Procedure (Safety Evaluation 00-048)	04/13/00		
	General Maintenance Procedure GMP-011, "Installation and Removal of S revised to prevent the level differential auto start functions for the screen v strainer from automatically starting when the Circulating Water bays are maintenance.	vash pump and		

PROCEDURE OR METHOD OF OPERATION CHANGES THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: April 2000

1-MOP-EP-206Maintenance Operating Procedures1-MOP-EP-207(Safety Evaluation 00-055)

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Maintenance Operating Procedures 1-MOP-EP-206, "Removing 4160V Bus 1J, 480V Buses 1J and 1J1, and 480V MCC 1J1-1 and 1J1-2 from Service" and 1-MOP-EP-207, "Returning 4160V Bus 1J, 480V Buses 1J and 1J1, and 480V MCC 1J1-1 and 1J1-2 to Service ", were revised to provide administrative controls to assure the capability to meet design basis for non-essential Service Water (SW) isolation safety function when one of the valves is taken out of service for testing or maintenance. Failure of the out of service valve to close in response to a non-essential SW isolation signal is addressed by 0-AP-12.01 and isolation can be accomplished within the required 1 hour via manual isolation valves.

1-MOP-EP-212 Maintenance Operating Procedure (Safety Evaluation 00-061)

Maintenance Operating Procedure 1-MOP-EP-212, "Removal From and Return to service of 4160/480 Feeder 1-EP-BKR-15G ", was revised to operate water well pump "A" on temporary power while breaker 1-EP-BKR-15G5 is out of service.

04/20/00

04/27/00

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR: April 2000

None during the Reporting Period

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CHEMISTRY REPORT

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MONTH/YEAR: April 2000

	Unit No. 1			Unit No. 2		
Primary Coolant Analysis	Max.	Min.	Avg.	Max.	Min.	Avg.
Gross Radioactivity, µCi/ml	5.11E-1	6.99E-4	8.95E-2	1.82E-1	1.19E-1	1.43E-1
Suspended Solids, ppm	0.050	0.010	0.022	-	-	-
Gross Tritium, μCi/ml	3.68E-2	3.48E-2	3.58E-2	8.62E-1	8.32E-1	8.40E-1
l ¹³¹ , μCi/ml	5.16E-4	1.63E-4	3.21E-4	≤ 1.04E-4	≤6.80E-5	≤ 8.71E-5
131 _{/1} 133	0.09	0.06	0.07	≤ 0.19	≤ 0.12	≤ 0.16
Hydrogen, cc/kg	36.2	0.8	18.3	40.2	34.7	36.9
Lithium, ppm	0.76	0.15	0.64	2.31	2.09	2.20
Boron - 10, ppm*	481.4	0.2	310.3	106.2	88.6	97.9
Oxygen, (DO), ppm	7	0.005	4.1	≤ 0.005	≤ 0.005	≤ 0.005
Chloride, ppm	≤ 0.05	≤ 0.001	≤ 0.004	≤ 0.05	<u> </u>	<u>≤ 0.007</u>
pH @ 25 degree Celsius	9.65	4.61	5.88	7.08	6.97	7.02

* Boron - 10 = Total Boron x 0.196

Comments:

None

FUEL HANDLING UNITS 1 & 2

MONTH/YEAR: April 2000

	New Fuel		Number of				New or Spent
	Shipment or	Date Stored or	Assemblies	Assembly	ANSI	Initial	Fuel Shipping
-	Cask No.	Received	per Shipment	Number	Number	Enrichment	Cask Activity

None during the Reporting Period

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DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

MONTH/YEAR: April 2000

None during the Reporting Period