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Executive Vice President and  
Chief Nuclear Officer  
North Atlantic Energy Service Corporation  
c/o Mr. James M. Peschel  
P.O. Box 300  
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April 11, 2000

TClark

SUBJECT: SEABROOK STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT RE:  
RELOCATION OF THE NUCLEAR SAFETY AUDIT REVIEW COMMITTEE  
TECHNICAL SPECIFICATION (TAC NO. MA7706)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 67 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No 1, in response to your application dated November 19, 1999.

The amendment would relocate Technical Specification (TS) 6.4.3, "Nuclear Safety Audit Review Committee (NSARC)," to the Quality Assurance Program located in the Updated Final Safety Analysis Report.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,  
/RA/

Robert M. Pulsifer, Project Manager, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosures: 1. Amendment No. 67 to NPF-86  
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 11, 2000

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NORTH ATLANTIC ENERGY SERVICE CORPORATION, ET AL.\*

DOCKET NO. 50-443

SEABROOK STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67  
License No. NPF-86

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by the North Atlantic Energy Service Corporation, et al. (the licensee), dated November 19, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

\*North Atlantic Energy Service Corporation (NAESCO) is authorized to act as agent for the: North Atlantic Energy Corporation, Canal Electric Company, The Connecticut Light and Power Company, Great Bay Power Corporation, Hudson Light & Power Department, Massachusetts Municipal Wholesale Electric Company, Little Bay Power Corporation, New England Power Company, New Hampshire Electric Cooperative, Inc., Taunton Municipal Light Plant, The United Illuminating Company, and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

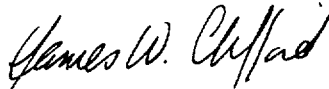
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-86 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 67 , and the Environmental Protection Plan contained in Appendix B are incorporated into Facility License No. NPF-86. NAESCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented at the facility within 90 days of issuance (including issuance of the Quality Assurance Program for use by licensee personnel). In addition, the licensee shall include the relocated information in the UFSAR, submitted to the NRC, pursuant to 10 CFR 50.71(e) as was described in the licensee's application dated November 19, 1999, and evaluated in the staff's safety evaluation dated April 11, 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



James W. Clifford, Chief, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: April 11, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 67

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. Overleaf pages have been provided.\*

Remove

XIV

6-8B

6-9

6-10

6-11

Insert

XIV

6-8B

6-9

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6.0 ADMINISTRATIVE CONTROLS

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6.4.3 THIS SPECIFICATION NUMBER IS NOT USED

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## 6.0 ADMINISTRATIVE CONTROLS

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## ADMINISTRATIVE CONTROLS

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## ADMINISTRATIVE CONTROLS

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### 6.5 REPORTABLE EVENT ACTION

The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC and the results of this review shall be submitted to the NSARC and the Executive Vice President & Chief Nuclear Officer.

### 6.6 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Executive Vice President & Chief Nuclear Officer and the NSARC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSARC, and the Executive Vice President & Chief Nuclear Officer within 14 days of the violation; and
- d. Operation of the station shall not be resumed until authorized by the Commission.



UNITED STATES  
**NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. NPF-86

NORTH ATLANTIC ENERGY SERVICE CORPORATION

SEABROOK STATION, UNIT NO. 1

DOCKET NO. 50-443

**1.0 INTRODUCTION**

By letter dated November 19, 1999, the North Atlantic Energy Service Corporation (NAESCO, the licensee) submitted a request for changes to the Seabrook Station, Technical Specifications (TS). NAESCO proposed modifying the Administrative Controls specifications, removing requirements that are adequately controlled by existing regulations and relocating details which are not otherwise needed to satisfy Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36. Guidance on the proposed changes was developed by the Nuclear Regulatory Commission (NRC) and provided in the Standard Technical Specifications (STS) for Westinghouse Plants, NUREG-1431, Revision 1, dated April 1995. The requested change would specifically relocate TS 6.4.3, "Nuclear Safety Audit Review Committee (NSARC)," to the Quality Assurance Program (QAP) located in the Updated Final Safety Analysis Report (UFSAR).

**2.0 BACKGROUND**

NAESCO stated that relocating the specific requirements of the NSARC requirements from the TS to the QAP Chapter 17.2 of the UFSAR allows these requirements to be controlled by the 10 CFR 50.54(a) and 10 CFR 50.59 change process rather than the TS amendment process. The licensee stated that this can reduce the regulatory burden on NAESCO and the NRC for those organizational and process changes that do not satisfy the criteria of 10 CFR 50.36 for inclusion in the TS.

Section 182.a of the Atomic Energy Act of 1954, as amended, (the Act) requires applicants for nuclear power plant operating licenses to state TS to be included as part of the license. The Commission's regulatory requirements related to the content of TS are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The Commission has provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"),

(58 FR 39132) July 22, 1993, in which the Commission indicated that compliance with the Final Policy Statement satisfies §182.a of the Act. In particular, the Commission indicated that certain items could be relocated from the TS to licensee-controlled documents.

Title 10 of the Code of Federal Regulations, Section 50.36, identifies four criteria to be used in determining whether particular safety functions are required to be included in the TS, as follows: (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product boundary; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of or presents a challenge to the integrity of a fission product boundary; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. As a result, TS requirements which fall within or satisfy any of the criteria in the regulations must be retained in the TS, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

The Final Policy Statement provides that those existing TS LCOs which do not satisfy these four specific criteria may be relocated to the UFSAR, such that future changes could be made to these provisions pursuant to 10 CFR 50.59. Other requirements may be relocated to more appropriate documents (e.g. Security Plan, QAP, and Emergency Plan) and controlled by the appropriate regulatory requirement. Similarly, while the required content of TS administrative controls is specified in 10 CFR 50.36(c)(5), particular details of administrative controls may be relocated to licensee-controlled documents where §50.54, §50.59, or other regulations provide adequate regulatory control.

While the criteria specifically apply to LCOs, in adopting the revision to 10 CFR 50.36 the Commission indicated that the intent of these criteria can be utilized to identify the optimum set of administrative controls in the TS (60 FR 36958). Addressing administrative controls 10 CFR 50.36 states that they are "the provisions relating to organization and management, procedures recordkeeping, review and audit, and reporting necessary to assure safe operation of the facility in a safe manner." The specific content of the administrative controls section of the TS is therefore that information that the Commission deems essential for the safe operation of the facility that is not already adequately covered by other regulations. Accordingly, the staff has determined that requirements that are not specifically required under §50.36(c)(5) and which are not otherwise necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety, can be removed from administrative controls.

### 3.0 EVALUATION

The following discussion sets forth details of the NRC staff's conclusions regarding the removal or relocation of selected Administrative Controls from the TS for the Seabrook Station. The changes were reviewed in accordance with the guidance provided in, or planned for, the applicable STS, NUREG-1431, Revision 1. In addition, these changes were reviewed in accordance with the guidance provided in Administrative Letter 95-06, "Relocation of Technical

Specification Administrative Controls Related to Quality Assurance" issued on December 12, 1995.

Given that the requirements in the QAP implement the Commission's regulations pertaining to the review and audit functions, inclusion of these particular provisions in TS is not necessary to assure safe operation of the facility. The review and audit functions define an administrative framework to confirm that plant activities have been properly conducted in a safe manner. The reviews and audits serve to provide a cohesive program that provides senior level utility management with assessments of facility operation and recommends actions to improve nuclear safety and reliability. However, the staff has determined that the review and audit functions are adequately addressed by existing regulations and the related QAP commitments. Based upon the relocation of the review and audit provisions to the QAP, it is not necessary to include redundant or additional requirements in the TS administrative controls.

The licensee will continue to implement a QAP in accordance with the requirements of 10 CFR Part 50, Appendix B, which provides appropriate controls for the approval of changes to the audit functions and frequencies. Changes to the QAP are controlled in accordance with 10 CFR 50.54(a) and include requirements for prior NRC review and approval if a change constitutes a reduction in a QAP commitment. The staff concludes that this regulatory requirement provides sufficient control for the audit functions and frequencies, so that removing these requirements from the TS is acceptable.

Audit requirements are specified in the QAP to satisfy 10 CFR Part 50, Appendix B. Audits are also covered by ANSI N18.7, ANSI N45.2, 10 CFR 50.54(t), 10 CFR 50.54(p), and 10 CFR Part 73.

The licensee has proposed to relocate the specific requirements of TS 6.4.3 intact to the QAP. The requirements have not been changed or reduced. The licensee provided, as requested by Administrative Letter 95-06, a copy of the new section of the QAP, Appendix 17C, from Chapter 17.2 of the UFSAR which includes the relocated requirements.

The staff concludes that sufficient regulatory controls exist under 10 CFR 50, Appendix B for the implementation of the functions specified in the QAP, and sufficient controls exist under 10 CFR 50.54(a) for subsequent changes to the QAP such that moving these review and audit requirements to the QAP is acceptable.

The required controls related to record retention specified in various regulations and the provision incorporated into the QAP are considered to be redundant to the requirements currently in TS. The staff has determined that record retention requirements are adequately addressed in the related QAP, it is not necessary to include redundant or additional requirements in the TS administrative controls.

The staff concludes that the regulatory requirements under 10 CFR Part 50, Appendix B provide sufficient control of the plant records, and sufficient regulatory controls exist for future changes to the program pursuant to 10 CFR 50.54(a), such that removing these provisions from the TS is acceptable.

In conclusion, the above relocated requirements relating to administrative controls are not required to be in the TS under 10 CFR 50.36 or §182.a of the Atomic Energy Act, and are not

required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. In addition, the staff finds that the resulting new administrative controls provide all of the requirements needed to satisfy 10 CFR 50.36(c)(5), and sufficient regulatory controls exist under 10 CFR 50.59 and 50.54(a), or other applicable regulations to assure continued protection of the public health and safety. Accordingly, the staff has concluded that these requirements may be relocated from the TS to the above specified document.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Hampshire and Massachusetts State officials were notified of the proposed issuance of the amendment. The State officials had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes record keeping, reporting, or administrative procedures or requirements. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 4281). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Pulsifer

Date: April 11, 2000