



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 8, 2000

Mr. Ted C. Feigenbaum
Executive Vice President and
Chief Nuclear Officer
North Atlantic Energy Service Corporation
c/o Mr. James M. Peschel
P.O. Box 300
Seabrook, NH 03874

SUBJECT: SEABROOK STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT RE:
IMPLEMENTATION OF THE SECOND 10-YEAR INTERVAL INSERVICE TEST
PROGRAM AND APPROVAL TO USE A LATER CODE EDITION FOR
SEABROOK (TAC NO. MA8251)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 69 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No 1, in response to your application dated February 18, 2000.

The amendment revises Technical Specifications (TSs) Surveillance Requirements 4.0.5.a, 4.0.5.b, 4.0.5.e, and 4.4.6.2.2.e. These changes are required to ensure consistency between the TSs and the second 10-year inservice test program by approval to use the 1995 Edition and 1996 Addenda of the American Society of Mechanical Engineers (ASME) *Code for Operation and Maintenance of Nuclear Power Plants* (OM Code). The revision to TSs Surveillance Requirement 4.0.5.a also incorporates semi-quarterly and biennial intervals to the list of required frequencies for performing inservice test and inspection activities.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Robert M. Pulsifer, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosures: 1. Amendment No. 69 to NPF-86
2. Safety Evaluation

cc w/encls: See next page

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/RA/

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Office of Nuclear Reactor Regulation

DISTRIBUTION:

Docket No. 50-443

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2. Safety Evaluation

cc w/encls: See next page

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GHill (2)
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RUrban, RI
DTerao

*SE input provided 03/16/00
no major changes made.

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OFFICIAL RECORD COPY

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTH ATLANTIC ENERGY SERVICE CORPORATION, ET AL.*

DOCKET NO. 50-443

SEABROOK STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69
License No. NPF-86

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the North Atlantic Energy Service Corporation, et al. (the licensee), dated February 18, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*North Atlantic Energy Service Corporation (NAESCO) is authorized to act as agent for the: North Atlantic Energy Corporation, Canal Electric Company, The Connecticut Light and Power Company, Great Bay Power Corporation, Hudson Light & Power Department, Massachusetts Municipal Wholesale Electric Company, Little Bay Power Corporation, New England Power Company, New Hampshire Electric Cooperative, Inc., Taunton Municipal Light Plant, The United Illuminating Company, and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

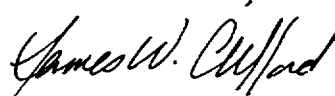
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-86 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 69 , and the Environmental Protection Plan contained in Appendix B are incorporated into Facility License No. NPF-86. NAESCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented by August 18, 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 8, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 69

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove

3/4 0-2
3/4 0-3
3/4 4-23
B3/4 0-6
B3/4 4-2
B3/4 4-2a

Insert

3/4 0-2
3/4 0-3
3/4 4-23
B3/4 0-6
B3/4 4-2
B3/4 4-2a

APPLICABILITY

SURVEILLANCE REQUIREMENTS

4.0.1 Surveillance Requirements shall be met during the OPERATIONAL MODES or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement.

4.0.2 Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval.

4.0.3 Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by Specification 4.0.2, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation. The time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.

4.0.4. Entry into an OPERATIONAL MODE or other specified condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation has been performed within the stated surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL MODES as required to comply with ACTION requirements.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, and 3 components shall be applicable as follows:

- a. Inservice inspection of ASME Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR Part 50, Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR Part 50, Section 50.55a(g)(6)(i).

Inservice testing of ASME Code Class 1, 2, and 3 components shall be performed in accordance with the Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code) and applicable Addenda as required by 10 CFR Part 50, Section 50.55a(f), except where specific written relief has been granted by the Commission pursuant to 10 CFR Part 50, Section 50.55a(f)(6)(i).

APPLICABILITY

SURVEILLANCE REQUIREMENTS

4.0.5 (Continued)

- b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and the ASME OM Code including applicable Addenda for the inservice inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and the ASME OM Code including applicable Addenda shall be applicable as follows in these Technical Specifications:

<u>ASME Boiler and Pressure Vessel Code and the ASME OM Code including applicable Addenda terminology for inservice inspection and testing activities</u>	<u>Required frequencies for performing service inspection and testing activities.</u>
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Semi-quarterly	At least once per 46 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days
Biennially or every 2 years	At least once per 731 days

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities;
- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements; and
- e. Nothing in the ASME Boiler and Pressure Vessel Code or the ASME OM Code shall be construed to supersede the requirements of any Technical Specification.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM LEAKAGE

OPERATIONAL LEAKAGE

SURVEILLANCE REQUIREMENTS

4.4.6.2.2 Each Reactor Coolant System Pressure Isolation Valve shall be demonstrated OPERABLE by verifying leakage to be within its limit:

- a. At least once per 18 months,
- b. Prior to entering MODE 2 whenever the plant has been in COLD SHUTDOWN for 7 days or more and if leakage testing has not been performed in the previous 9 months,
- c. Prior to returning the valve to service following maintenance, repair, or replacement work on the valve, and
- d. Within 24 hours following valve actuation due to automatic or manual action or flow through the valve.*
- e. Testing pursuant to Specification 4.0.5.

The provisions of Specification 4.0.4 are not applicable for entry into MODE 3 or 4.

* Not applicable to RHR Pumps 8A and 8B suction isolation valves.

3/4.0 APPLICABILITY

BASES

Specification 4.0.5 establishes the requirement that inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and the ASME OM Code including applicable Addenda as required by 10 CFR 50.55a. These requirements apply except when relief has been provided in writing by the Commission.

This specification includes a clarification of the frequencies for performing the inservice inspection and testing activities required by Section XI of the ASME Boiler and Pressure Vessel Code and the ASME OM Code including applicable Addenda. This clarification is provided to ensure consistency in surveillance intervals throughout the Technical Specifications and to remove any ambiguities relative to the frequencies for performing the required inservice inspection and testing activities.

Under the terms of this specification, the more restrictive requirements of the Technical Specifications take precedence over the ASME Boiler and Pressure Vessel Code and the ASME OM Code including applicable Addenda. The requirements of Specification 4.0.4 to perform surveillance activities before entry into an OPERATIONAL MODE or other specified condition takes precedence over the ASME OM Code provision which allows pumps that can only be tested during plant operation to be tested within 1 week following plant startup.

REACTOR COOLANT SYSTEM

BASES

3/4.4.2 SAFETY VALVES (Continued)

During plant operations in Mode 5, it is conservative and consistent with Technical Specifications that the OPERABLE pressurizer safety valve may be removed from its flange and continue to meet the intent of this Specification. The removal of the pressurizer safety valve will afford the reactor coolant system equivalent or superior protection as an overpressure device. This will also allow the removal of the three pressurizer safety valves to be used as a gravity vent path in place of removing the pressurizer manway when the plant is at reduced inventory conditions.

During operation, all pressurizer Code safety valves must be OPERABLE to prevent the RCS from being pressurized above its Safety Limit of 2735 psig. The combined relief capacity of all of these valves is greater than the maximum surge rate resulting from a complete loss of load assuming no Reactor trip until the first Reactor Trip System Trip Setpoint is reached (i.e., no credit is taken for a direct Reactor trip on the loss of load) and also assuming no operation of the power-operated relief valves or steam dump valves.

Demonstration of the safety valves' lift settings will be performed when removed from the reactor coolant system in accordance with the provisions of the ASME Code for Operation and Maintenance of Nuclear Power Plants.

3/4.4.3 PRESSURIZER

The limit on the maximum water volume in the pressurizer assures that the parameter is maintained within the normal steady-state envelope of operation assumed in the SAR. The limit is consistent with the initial SAR assumptions. The 12-hour periodic surveillance is sufficient to ensure that the parameter is restored to within its limit following expected transient operation. The maximum water volume also ensures that a steam bubble is formed and thus the RCS is not a hydraulically solid system. The requirement that a minimum number of pressurizer heaters be OPERABLE enhances the capability of the plant to control Reactor Coolant System pressure and establish natural circulation.

3/4.4.4 RELIEF VALVES

The power-operated relief valves (PORVs) and steam bubble function to relieve RCS pressure during all design transients up to and including the design step load decrease with steam dump. Operation of the PORVs minimizes the undesirable opening of the spring-loaded pressurizer Code safety valves. Each PORV has a remotely operated block valve to provide a positive shutoff capability should a relief valve become inoperable. The PORVs and their associated block valves are powered from Class 1E power supply busses.

REACTOR COOLANT SYSTEM

BASES

3/4.4.4 RELIEF VALVES (Continued)

The PORVs are equipped with automatic actuation circuitry and manual control capability. The PORVs are considered OPERABLE in either the automatic or manual mode for the following reasons:

- (1) No credit is taken in any FSAR accident analysis for automatic PORV actuation to mitigate the consequences of an accident.
- (2) No Surveillance Requirement (ACOT or TADOT) exists for verifying automatic operation.
- (3) The required ACTION for an inoperable PORV(s) (closing the block valve) conflicts with any presumed requirement for automatic actuation.

3/4.4.5 STEAM GENERATORS

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation, so that corrective measures can be taken.



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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. NPF-86

NORTH ATLANTIC ENERGY SERVICE CORPORATION

SEABROOK STATION, UNIT NO. 1

DOCKET NO. 50-443

1.0 INTRODUCTION

By letter dated February 18, 2000, the North Atlantic Energy Service Corporation (the licensee) submitted a request for changes to the Seabrook Station Technical Specifications (TSs). The requested changes would revise Surveillance Requirements (SRs) 4.0.5.a, 4.0.5.b, 4.0.5.e, and 4.4.6.2.2.e. These changes are required to ensure consistency between the TSs and the second 10-year inservice test program by approval to use the 1995 Edition and 1996 Addenda of the American Society of Mechanical Engineers (ASME) *Code for Operation and Maintenance of Nuclear Power Plants* (OM Code). The revision to TS Surveillance Requirement 4.0.5.a also incorporates semi-quarterly and biennial intervals to the list of required frequencies for performing inservice test and inspection activities. The revision to TSs Bases Section 3/4.4.2 also clarifies that the pressurizer safety relief valves are removed from the reactor coolant system and bench-tested to verify lift settings.

2.0 EVALUATION

Existing SRs 4.0.5.a, 4.0.5.b, 4.0.5.e, and 4.4.6.2.2.e and Bases Sections 3/4.4.2 and 4.0.5 reference Section XI of the ASME *Boiler and Pressure Vessel Code* when describing pump and valve inservice test requirements. The proposed SRs 4.0.5.a, 4.0.5.b, 4.0.5.e, and 4.4.6.2.2.e and Bases Sections 3/4.4.2 and 4.0.5 reference the ASME OM Code when describing pump and valve inservice test requirements.

In 1990 the ASME published the initial edition of the OM Code which gives rules for inservice testing of pumps and valves. The ASME intended that the ASME OM Code replace Section XI of the *Boiler and Pressure Vessel Code* for inservice testing of pumps and valves. The U.S. Nuclear Regulatory Commission (NRC) views the ASME OM Code as a replacement for Section XI referenced in 10 CFR 50.55a for the inservice testing of pumps and valves. The 1995 Edition including the 1996 Addenda of the ASME OM Code was incorporated by reference into the regulations in 10 CFR 50.55a(b) on September 22, 1999, (64 FR 51370). Therefore, the staff finds the proposed TSs change to reference the ASME OM Code to be acceptable.

Existing SR 4.0.5.b addresses surveillance intervals of weekly, monthly, quarterly, semiannually, every 9 months and annually, but not the surveillance intervals of semi-quarterly and biennially, or every 2 years. The proposed SR 4.0.5.b adds the semi-quarterly and biennially, or every 2 years surveillance intervals to the list of required frequencies for performing inservice inspection and testing activities.

The proposed change to add the reference of semi-quarterly and biennially or every 2 years to SR 4.0.5.b will maintain consistency with the Code requirements and will also permit the application of TS 4.0.2 criteria to Code semi-quarterly and 2-year surveillance activities. The TS 4.0.2 criteria will allow, if necessary, a 25-percent time extension to be applied to the semi-quarterly and 2-year surveillance test frequencies. The proposed TS change is consistent with the guidance delineated in NUREG-1482, "Guidelines for Inservice Testing and Nuclear Power Plants," and NUREG-1431, Revision 1, "Standard Technical Specifications for Westinghouse Plants." Therefore, the staff finds this proposed TS change to be acceptable. However, as indicated in NUREG-1482, Section 6.2, the 25-percent extension is not intended to be used repeatedly, but merely as an operational convenience to extend surveillance intervals beyond those required by the Code.

Existing TS Bases Section 3/4.4.2 states, in part, that demonstration of the safety valves' lift settings will occur only during shutdown. The proposed TS Bases Section 3/4.4.2 states, in part, that demonstration of the safety valves' lift settings will be performed when removed from the reactor coolant system.

In its letter dated February 18, 2000, the licensee stated that pressurizer safety valves are removed from the reactor coolant system and setpoint tested at an off-site test facility. The proposed change clarifies that the pressurizer safety valves are setpoint tested when removed from the system and, therefore, are not setpoint tested in place. The OM Code provides the general requirements for periodic performance testing and monitoring of pressure relief devices. The OM Code provides a number of options for setpoint testing pressure relief devices which include testing a valve after it has been removed from the system or replacement of the valve with a pretested valve. The staff finds that the proposed TS change complies with the requirements of the OM Code for setpoint testing pressurizer safety valves and is acceptable.

Existing TS Bases Section 4.0.5 specifies that the more restrictive requirements of the TSs take precedence over the requirements contained in Section XI of the *ASME Boiler and Pressure Vessel Code* for (1) testing pumps and valves after being returned to service; and (2) declaring a valve inoperable after identification that the valve is not capable of performing its specified function. The proposed TS Bases Section 4.0.5 specifies that the more restrictive requirements of the TSs take precedence over the requirement contained in the OM Code for testing pumps that can only be tested during plant operation.

In its letter dated February 18, 2000, the licensee stated that its current 120-month inservice test program is based on the requirements in the *ASME Boiler and Pressure Vessel Code*, Section XI, 1983 Edition through the Summer 1983 Addenda. The existing TS Bases Section 4.0.5 is based on the 1983 Edition and Summer 1983 Addenda of Section XI. The proposed TS Bases Section 4.0.5 is based on revised requirements contained in the OM Code 1995

Edition and 1996 Addenda which was incorporated by reference into the regulations in 10 CFR 50.55a(b) on September 22, 1999, (64 FR 51370). Therefore, the staff finds the proposed TS change to reference the latest requirements contained in the ASME OM Code to be acceptable, and is authorized pursuant to 10 CFR 50.55a(f)(4)(iv).

The NRC staff concludes that the changes to TSs SRs 4.0.5.a, 4.0.5.b, 4.0.5.e, and 4.4.6.2.2.e and Bases Sections 3/4.4.2 and 4.0.5 as described in the licensee's letter dated February 18, 2000, are acceptable. The staff further concludes that the licensee's proposal to use the 1995 Edition including the 1996 Addenda of the ASME OM Code for its second 10-year IST program is authorized pursuant to 10 CFR 50.55a(f)(4)(iv).

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Hampshire and Massachusetts State officials were notified of the proposed issuance of the amendment. The State officials had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 17917). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Tingen

Date: May 8, 2000