

Tennessee Valley Authority, Post Office Box 2000, Spring City, Tennessee 37381-2000

## MAY 2 2000

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Gentlemen:

In the Matter of Tennessee Valley Authority Docket No. 50-390

WATTS BAR NUCLEAR PLANT (WBN) - EMERGENCY PLAN IMPLEMENTING PROCEDURE (EPIP) REVISION

In accordance with the requirements of 10 CFR Part 50, Appendix E, Section V, the enclosure provides the following EPIP:

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EPIP	Rev	Title	Effective Date
EPIP-1	14	Emergency Plan Classification Flowchart	04-10-2000

Filing instructions are included with these documents.

If you should have any questions, please contact me at (423) 365-1824.

Sincerely,

SCH

P. L. Pace Manager, Licensing and Industry Affairs

Enclosure cc: See Page 2 U.S. Nuclear Regulatory Commission Page 2

## MAY 2 2000

Enclosure cc (Enclosure) NRC Resident Inspector Watts Bar Nuclear Plant 1260 Nuclear Plant Road Spring City, Tennessee 37381

> Mr. Robert E. Martin, Senior Project Manager U.S. Nuclear Regulatory Commission One White Flint North 11555 Rockville Pike Rockville, Maryland 20852

U.S. Nuclear Regulatory Commission (2 copies) Region II Atlanta Federal Center 61 Forsyth St., SW, Suite 23T85 Atlanta, Georgia 30303

# FILING INSTRUCTIONS

DOCUMENT NUMBER <u>FPJP</u> -	1.
REMOVE REVISION $13$ insert	REVISION 14
Comments	

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## **TENNESSEE VALLEY AUTHORITY**

## WATTS BAR NUCLEAR PLANT

## EMERGENCY PLAN IMPLEMENTATING PROCEDURES

## EPIP-1

## EMERGENCY PLAN CLASSIFICATION FLOWCHART

Revision 14

Unit 0

## **QUALITY RELATED**

PREPARED BY: <u>Ben McNew</u> (Type Name)

SPONSORING ORGANIZATION: <u>Emergency Planning</u>

APPROVED BY: \_\_\_\_Frank L. Pavlechko\_\_\_\_\_

EFFECTIVE DATE: 04/10/00

LEVEL OF USE: REFERENCE

WBN

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## EMERGENCY PLAN CLASSIFICATION FLOWCHART

## **REVISION LOG**

Revision	Implementation	Description of Revision	
Number	Date		
0	04/13/90	New WBN-EPIP. Supersedes IP-1.	
1	02/04/91	Revised to separate RCS leak and identified S/G tube leak initiating conditions. Clarified initiating condition in fire. Updated ODS telephone numbers.	
2	11/28/91	Add initiation conditions. Clarify reference to Attachment 1 Definitions. Define Protected Area, Owner Controlled Area, and Vital Areas throughout procedures. Clarify NOUE declaration for Uncontrolled Shutdown.	
3	03/04/92	Change all Technical Specification references to reflect new "Merit" Tech Specs and ODCM references.	
4	02/10/93	Procedure revised to reflect the new methodology for development of Emergency Action Levels per: NUMARC/NESP-007, Rev. 3, 1/92, endorsed by REG GUIDE 1.101 Emergency Planning and Preparedness For Nuclear Power Reactors Rev. 3, 8/92.	
5	09/15/93	Editorial (non-intent) and formal changes. Text changes made to EALs to meet review comments identified by the NRC.	
6	01/01/94	Procedure revised to reflect new 10 CFR 20 changes.	
7	05/27/94	Procedure revised to reflect changes to System 90 (Radmonitoring) and establish site perimeter monitoring points.	
8	01/10/95	FPBM, EAL 1.3.4, CNTMT, Bypass, Loss (1), revised to eliminate potential for misclassification. Maps revised to reference north and wind direction. Table 7-2, Alert, Radiation Levels enhanced to provide Operators additional information.	
9	4/28/98	Revised Revision Log to include page numbers. References added to the document. Fission Product Barrier Matrix revised to reflect information found in the EOP Set Point Verification Document (WBN-OS64-188). Reference to AOI-27 revised to AOI-30.2. Phone numbers to the National Weather Service changed due to their reorganization. Annunciator window references for the earthquake corrected to match Main Control Room alignment. All references to RM were changed to RE to make it consistent with site description documents. Tables in section seven revised to reflect the following: System 90 changes, monitor efficiencies, default flow rates, release time durations, and annual meteorological data enhancements.	

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## **REVISION LOG**(Continued)

Revision	Implementation	Pages	Description of Revision
Number	Date	Affected	
CN-1	09/28/95	10, 14, 26	The following non-intent enhancements were made: (CCP) Acronym added to the Fission Product Barrier Matrix in 1.2 RCS Barrier, (2. RCS Leakage LOCA), to enhance description. New SI reference number for Reactor Coolant System Water Inventory Balance identified in event 2.5 (RCS Unidentified Leakage) and 2.6 (RCS Identified Leakage). Area code and phone number in event 5.2 (Tornado) revised to new number.
CN-2	11/10/95	3, 6, 34	The following non-intent enhancements were made: Corresponding ERFDS system identifiers were added next to the rad monitors on Table 7-1; Table 7-1 was realigned to improve its usability; an enhanced description for RE-404 was provided in Note 3 of Table 7-1; the ERFDS Operators Manual was added to the Reference section.
CN-3	05/24/96	8, 11, 16, 19, 23, 24, 26, 29, 32, 34	The following non-intent enhancements were made: Due to revisions made to AOI-27, it was added back to the EALs in event 4.5 "Control Room Evacuation"; The Assessment Method on Table 7-1 was enhanced to correspond with the note at the top of the table. In addition, the reference to TI-30 was removed since this procedure will be terminated due to the enhancements being made to EPIP-16 and ERFDS. The word Projectile was added to the index and title reference to event 5.3 "Aircraft/Projectile Crash", to make it consistent with the EALs within it's classification.
10	3/15/99	All	The following non-intent enhancement were made: Software revised to Microsoft Word which re-formatted pages along with other enhancements; minor typographical errors corrected; two references revised - one added; SOS/ASOS replaced with SM/US; index page, effluent added to gaseous; vital area definition enhanced; spent fuel pit revised to pool on Table 7-2; SP revised to EAB in Event 7.1; TVA Load Dispatcher/Water Resources revised to River Systems Operations and revised ERFDS/P-2500 to ICS.
11	4/15/99	2, 34	Non intent change. Typo corrected. Changed >1.0 to >0.1.
11A	7/1/99	3,26	Corrected typo on phone number The remaining pages of this procedure are Rev 11 only page 3, and the fold out page for 26 have been changed.

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## **REVISION LOG** (Continued)

Revision Number	Implementation Date	Pages Affected	Description of Revision
12	9/30/99	All	Non intent change. Minor editorial/format changes made. Typographical errors corrected. Seismic windows revised to reflect DCN-50007 per ERPI Report 6695. (LTL) Lower toxicity limit replaced with (PEL) Permissible Exposure Limit. This revision is also part of the resolution to PER 99-009326-000.
13	12/08/99	All	Non-intent change. Revised page 33 for resolution of PER 99-015478-000. Minor editorial change to Event 5.1 step 1 of the Alert classification.
14	04/10/00	All (Pg.4 & 45)	Non-intend change. Revised page 45 for DCN 50484, stage 1 which moved 0-RE-90-101B, & -132B from ICS Screen 4RM2 to 4RM1. DCN also moved 1-RE-90-421B thru -424B and 0-RE-90-120 & -121 from ICS Screen 4RM1 to 4RM2. This revision allows all liquid radiation monitors to be observable on one ICS screen and all gaseous radiation monitors to be observed on a separate ICS screen.

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## 1.0 PURPOSE<sup>4</sup>

This Procedure provides guidance in determining the classification and declaration of an emergency based on plant conditions.

## 2.0 **RESPONSIBILITY**<sup>2,4</sup>

The responsibility of declaring an Emergency based on the guidance within this procedure belongs to the Shift Manager/Site Emergency Director (SM/SED) or designated Unit Supervisor (US) when acting as the SM or the TSC Site Emergency Director (SED). These duties <u>CAN NOT</u> be delegated.

## 3.0 INSTRUCTIONS<sup>4</sup>

- 3.1 The criteria in WBN EPIP-1 are given for GUIDANCE ONLY: knowledge of actual plant conditions or the extent of the emergency may require that additional steps be taken. In all cases, this logic procedure should be combined with the sound judgment of the SM/SED and/or the TSC SED to arrive at a classification for a particular set of circumstances.
- 3.2 The Nuclear Power (NP) Radiological Emergency Plan (REP) will be activated when any one of the conditions listed in this logic is detected.
- 3.3 Classification Determination
  - 3.3.1 To determine the classification of the emergency, review the Initiating Conditions of the Events described in this procedure with the known or suspected conditions and CARRY OUT the notifications and actions referenced.
  - 3.3.2 If a Critical Safety Function (CSF) is listed as an Initiating Condition: the respective status tree criteria will be monitored and used to determine the Event classification for the modes listed on the classification flowchart.
  - 3.3.3 The highest classification for which an Emergency Action level (EAL) currently exists shall be declared.

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## 3.0 **INSTRUCTIONS** (continued)

- 3.3.4 After an Event classification, if the following investigation shows that Initiating Conditions were met that dictate a higher Event classification, the new event classification shall be declared at the clock time of the determination.
- 3.3.5 **IF** an EAL for a higher classification <u>was</u> exceeded but the present situation indicates a lower classification, the fact that the higher classification occurred SHALL be reported to the NRC and Central Emergency Control Center (CECC), but should <u>not</u> be declared.
- 3.3.6 **IF** the Parameter is indeterminate due to instrument malfunction and the existence of the condition **CAN NOT** be reasonably discounted (i.e., spurious or false alarm that can be substantiated within 15 minutes) the condition is considered **MET** and the SM/SED SHALL follow the indications provided until such time as the alarm is verified to be false.
- 3.3.7 **IF** an EAL was exceeded, but the emergency has been totally resolved (prior to declaration), the emergency condition that was appropriate <u>shall</u> <u>not</u> be declared but reported to the NRC and Operations Duty Specialist (ODS) at the same clock time.
- 3.3.8 The **ACCEPTABLE** time frame for notification to the Operation Duty Specialist (ODS) is considered to be five (5) minutes. This is the time period between declaration of the emergency and notifying the ODS.

## 4.0 RECORDS

4.1 QA Records

None

4.2 Non-QA Records

None

## 5.0 REFERENCES

## 5.1 Interfacing Documents

BP-236, Event Critique and Root Cause Analysis

SPP 3.5, Regulatory Reporting Requirements

WBN-EPIP-2, Unusual Event

WBN-EPIP-3, Alert

WBN-EPIP-4, Site Area Emergency

WBN-EPIP-5, General Emergency

WBN-EPIP-9, Loss of Meteorological Data

WBN-EPIP-13, Termination of the Emergency and Recovery

WBN-EPIP-14, Radiological Control Response

WBN-EPIP-16, Initial Dose Assessment For Radiological Emergencies

CECC-EPIP-9, Emergency Environmental Radiological Monitoring Procedures

SI-4.04, Measurement of Identified and Unidentified Leakage of the Reactor Coolant System

## 5.2 Other Documents

10 CFR 50, Domestic Licensing of Production and Utilization Facilities

10 CFR 20, Standards for Protection From Radiation

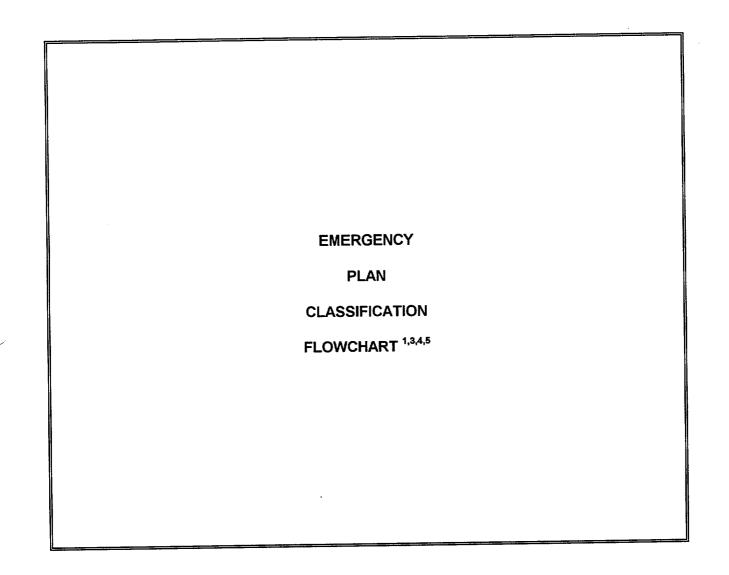
REG GUIDE-1.101, Emergency Planning and Preparedness For Nuclear Power Reactors endorsing NUMARC NESP-007 Methodology for Development of Emergency Action Levels.

Site Technical Specifications (Tech Specs), Abnormal Operating Instructions (AOIs), Emergency Operating Procedures (EOPs), Set Point Verification documents, Chemistry Technical documents (CTDs), and the Final Safety Analysis Report (FSAR) are also referenced in Appendix C of the Radiological Emergency Plan.

ICS Operator's Manual

EPPOS #2, "NRC EP Position on Timeliness of Classification of Emergency Conditions

EPRI Report 6695 Guidelines for Nuclear Power Plant Response to Earthquakes.



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FIS 1.1 1.2 1.3	SION PRODUCT BARRIER MATRIX (Modes 1-4) Fuel Clad RCS Containment	1		
<b>SY</b> 2.1 2.2 2.3 2.4 2.5	STEM DEGRADATIONLoss of Instrumentation2.6Loss of Function/Communication2.7Failure of Reactor Protection2.8Fuel Clad Degradation2.9RCS Unidentified Leakage2.10Safety Limit	2		
LO 3.1 3.2 3.3	SS OF POWER Loss of AC (Power Ops) Loss of AC (Shutdown) Loss of DC	3		
HA 4.1 4.2	ZARDS and SED JUDGMENTFire4.3Flammable Gas4.5Control Room EvacuationExplosion4.4Toxic Gas4.6SecurityTable 4-1Table 4-24.7SED JudgmentFigure 4-AFigure 4-BTable 4-3	4		
DE 5.1 5.2 5.3	STRUCTIVE PHENOMENONEarthquake5.4River Level HighTornado5.5River Level LowAircraft/Projectile5.6Watercraft CrashCrashFigure 5-ATable 5-1	5		
SH 6.1 6.2 6.3 6.4	6.2Loss of AC (Shutdown)06.3Loss of DC (Shutdown)0			
<b>R</b> A 7.1 7.2	DIOLOGICALGaseous Effluent7.3Radiation LevelsLiquid Effluent7.4Fuel HandlingTable 7-1Table 7-2Figure 7-A	7		

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### **DEFINITIONS/ACRONYMS**

UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4.7).

BOMB: An explosive device (See EXPLOSION).

**CIVIL DISTURBANCE:** A group of twenty (20) or more persons violently protesting station operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs: Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the conditions associated with the event exist. Implicit in this definition is the need for timely assessment, i.e. within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): The demarcation of the area surrounding the WBN units in which postulated FSAR accidents will not result in population doses exceeding the criteria of 10 CFR Part 100. Refer to Figure 7-A.

EXPLOSION: A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation.

EXTORTION: An attempt to cause an action at the station by threat of force.

FAULTED: (Steam Generator) Existence of secondary side leakage (i.e., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition.

HOSTAGE: A person(s) held as leverage against the station to ensure that demands will be met by the station.

**INEFFECTIVE:** The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH. A reduction in the level of severity is an improvement in the applicable parameters, e.g., Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization.

ODCM: Offsite Dose Calculation Manual.

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge.

**PROJECTILE:** An object ejected, thrown, or launched towards a plant structure. The source of the projectile may be onsite or offsite. Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein.

**PROTECTED AREA:** Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

**RED PATH:** Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge, prompt operator action is required.

**RUPTURED:** (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity.

**SABOTAGE:** Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load; (3) Reactor Trip or (4) Safety Injection System Activation.

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-Aand 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine).

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i.e., within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e.g., paint chipping, scratches) should NOT be included.

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

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1.1 Fuel C	Clad Barrier	
1. Critical Safety Fun	ction Status	1. Critic
LOSS	Potential LOSS	
Core Cooling Red	Core Cooling Orange	Not Applic
	<u>OR</u> Heat Sink Red	
	(RHR <u>Not</u> in Service)	
	OR-	
2. Primary Coolant Ad	stivity Level	
LOSS	Potential LOSS	2. RCS
RCS sample activity is Greater Than 300 μCi/gm	Not applicable	
dose equivalent iodine-131		RCS Leak
-	OR-	, (<65°F In
		[85°F AD)
3 Incore TCs Hi Qua Loss	d Average Potential LOSS	
Greater Than 1200°F	Greater Than 727°F	
-	OR-	
4 Reactor Vessel Wa	ater Level	3. Stea
LOSS	Potential LOSS	SGTR that
Not Applicable	VALID RVLIS level <33% (No RCP running)	safety inje
	,	Entry into
-	OR-	
5. Containment Radia	ation Manitorn	4. Read
LOSS	Potential LOSS	
VALID reading increase of Greater Than:	Not Applicable	VALID RV <33%
		(No RCP
9x10 <sup>1</sup> R/hr On 1-RE-90- 271 and 272		
<u>OR</u> 7x10 <sup>1</sup> R/hr On 1-RE-90-		
273 and 274		
	OR-	
6. Site Emergency Di Any condition that, in the		5. Site
Indicates Loss or Potentia	Loss of the Fuel Clad	Any cond Indicates
Barrier Comparable to the	Conditions Listed Above.	Compara
		·

1.2 RC	S Barrier
1. Critical Safety Fun	ction Status
LOSS	Potential LOSS
Not Applicable	Pressurized Thermal Shock Red
	OR
	Heat Sink Red (RHR <u>Not</u> in Service)
-	OR-
2 PCS Lookago/I OC	۰۸
2 RCS Leakage/LOC	Potential LOSS
RCS Leak results in	Non Isolatable RCS Leak Exceeding The Capacity of
Loss of subcooling (<65°F Indicated),	One Charging Pump (CCP)
[85°F ADV]	In the Normal Charging Alignment.
	<u>OR</u> RCS Leakage Results In
	Entry Into E-1
-	OR-
3. Steam Generator	Fube Rupture
LOSS	Potential LOSS
SGTR that results in a safety injection actuation	Not Applicable
OR Entry into E-3	
	OR-
4. Reactor Vessel W	atas Lavel
LOSS	Potential LOSS
VALID RVLIS level <33%	Not Applicable
(No RCP Running)	
	1
•	-OR-
5. Site Emergency D	
	Judgment of the SM/SED, I Loss of the RCS Barrier
Comparable to the Condit	

1.3 CNTM	T Barrier
Critical Safety Function	
LOSS	Potential LOSS
Not Applicable	Containment (FR-Z.1) <u>Red</u> OR
	Actions of FR-C.1 (Red Path) are INEFFECTIVE
-	
-OF 2. Containment Pressure	
LOSS	Potential LOSS
Rapid unexplained decrease	Containment Hydrogen
ollowing initial increase	Increases to >4% by volume
OR Containment pressure or	<u>OR</u> Pressure >2.8 PSIG (Phase
Sump level <u>Not</u> increasing	B) with < One full train of
with LOCA in progress)	Containment spray
-01	<u>،</u>
<ol> <li>Containment Isolation</li> </ol>	
LOSS	Potential LOSS
Containment Isolation is	Not Applicable
ncomplete <u>AND</u> a Release Path to the Environment Exists	
-01	र-
4. Containment Bypass LOSS	D-4
RUPTURED S/G is also	Potential LOSS Unexplained VALID increase
FAULTED outside CNTMT	in area or ventilation RAD
OR	monitors in areas adjacent to
_	
<u>OR</u> Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G	monitors in areas adjacent to CNTMT (with LOCA in
<u>OR</u> Prolonged (>4 Hours) Secondary Side release	monitors in areas adjacent to CNTMT (with LOCA in
<u>OR</u> Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G	monitors in areas adjacent to CNTMT (with LOCA in progress)
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -OI 5. Significant Radioactiv	monitors in areas adjacent to CNTMT (with LOCA in progress) <b>R-</b>
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -O 5. Significant Radioactiv LOSS	monitors in areas adjacent to CNTMT (with LOCA in progress) R- vity in Containment Potential LOSS
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -OI 5. Significant Radioactiv	monitors in areas adjacent to CNTMT (with LOCA in progress) R- vity in Containment
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -O 5. Significant Radioactiv LOSS	monitors in areas adjacent to CNTMT (with LOCA in progress) R- vity in Containment Potential LOSS VALID Reading increase of
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -O 5. Significant Radioactiv LOSS	monitors in areas adjacent to CNTMT (with LOCA in progress) R- /ity in Containment Potential LOSS VALID Reading increase of Greater Than: 3.6 x 10 <sup>2</sup> R/hr on 1-RE-90-271 and 1-RE-90-272 2.8 x 10 <sup>2</sup> R/hr on 1-RE-90-273 and
<u>OR</u> Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -O 5. Significant Radioactiv LOSS Not Applicable	monitors in areas adjacent to CNTMT (with LOCA in progress) R- /ity in Containment Potential LOSS VALID Reading increase of Greater Than: 3.6 x 10 <sup>2</sup> R/hr on 1-RE-90-271 and 1-RE-90-272 <u>OR</u> 2.8 x 10 <sup>2</sup> R/hr on 1-RE-90-273 and 1-RE-90-274
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -O 5. Significant Radioactiv LOSS	monitors in areas adjacent to CNTMT (with LOCA in progress) R- /ity in Containment Potential LOSS VALID Reading increase of Greater Than: 3.6 x 10 <sup>2</sup> R/hr on 1-RE-90-271 and 1-RE-90-272 <u>OR</u> 2.8 x 10 <sup>2</sup> R/hr on 1-RE-90-273 and 1-RE-90-274
<u>OR</u> Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -OI 5. Significant Radioactiv LOSS Not Applicable	monitors in areas adjacent to CNTMT (with LOCA in progress) R- /ity in Containment Potential LOSS VALID Reading increase of Greater Than: 3.6 x 10 <sup>2</sup> R/hr on 1-RE-90-271 and 1-RE-90-272 <u>OR</u> 2.8 x 10 <sup>2</sup> R/hr on 1-RE-90-273 and 1-RE-90-274 R-
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -OI 5. Significant Radioactiv LOSS Not Applicable -O 5. Site Emergency Direct	monitors in areas adjacent to CNTMT (with LOCA in progress) R- /ity in Containment Potential LOSS VALID Reading increase of Greater Than: 3.6 x 10 <sup>2</sup> R/hr on 1-RE-90-271 and 1-RE-90-272 <u>OR</u> 2.8 x 10 <sup>2</sup> R/hr on 1-RE-90-273 and 1-RE-90-274 R-
OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -OI 5. Significant Radioactiv LOSS Not Applicable -O 5. Site Emergency Direct Any condition that, in the Judgm Loss or Potential Loss of the CN	monitors in areas adjacent to CNTMT (with LOCA in progress) R- /ity in Containment Potential LOSS VALID Reading increase of Greater Than: 3.6 x 10 <sup>2</sup> R/hr on 1-RE-90-271 and 1-RE-90-272 <u>OR</u> 2.8 x 10 <sup>2</sup> R/hr on 1-RE-90-273 and 1-RE-90-274 R- ctor Judgment ient of the SM/SED, Indicates
<u>OR</u> Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits -OI 5. Significant Radioactiv LOSS Not Applicable	monitors in areas adjacent to CNTMT (with LOCA in progress) R- /ity in Containment Potential LOSS VALID Reading increase of Greater Than: 3.6 x 10 <sup>2</sup> R/hr on 1-RE-90-271 and 1-RE-90-272 <u>OR</u> 2.8 x 10 <sup>2</sup> R/hr on 1-RE-90-273 and 1-RE-90-274 R- ctor Judgment ient of the SM/SED, Indicates

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## Modes: 1, 2, 3, 4

## INSTRUCTIONS

NOTE: A condition is considered to be MET if, in the judgment of the Site Emergency Director, the condition will be MET imminently (i.e., within 1 to 2 hours, in the absence of a <u>viable success path</u>). The classification shall be made a soon as this determination is made.

- 1. In the matrix to the left, review the INITIATING CONDITIONS in all columns and identify which, if any, INITIATING CONDITIONS are MET. Circle these CONDITIONS.
- 2. For each of the three barriers, identify if any LOSS or Potential LOSS INITIATING CONDITIONS have been MET.
- 3. If a CSF is listed as an INITIATING CONDITION; the respective status tree criteria will be monitored and used to determine the EVENT classification for the Modes listed on the classification flowchart.
- 4. Compare the barrier losses and potential losses to the EVENTS below and make the appropriate declaration.

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F I

#### EVENTS

UNUSUAL EVENT	ALERT
Loss <u>or</u> Potential LOSS of Containment Barrier	Any LOSS <u>or</u> Potential LOSS of Fuel Clad barrier
	OR
	Any LOSS <u>or</u> Potential LOSS of RCS barrier
SITE AREA EMERGENCY	GENERAL EMERGENCY
LOSS <u>or</u> Potential LOSS of any two barriers	LOSS of any two barriers <u>and</u> Potential LOSS of third barrier

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## **DEFINITIONS/ACRONYMS**

## UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4.7).

BOMB: An explosive device (See EXPLOSION).

**CIVIL DISTURBANCE:** A group of twenty (20) or more persons violently protesting station operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs: Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the conditions associated with the event exist. Implicit in this definition is the need for timely assessment, i.e. within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): The demarcation of the area surrounding the WBN units in which postulated FSAR accidents will not result in population doses exceeding the criteria of 10 CFR Part 100. Refer to Figure 7-A.

**EXPLOSION:** A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation.

**EXTORTION:** An attempt to cause an action at the station by threat of force.

FAULTED: (Steam Generator) Existence of secondary side leakage (i.e., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition.

**HOSTAGE:** A person(s) held as leverage against the station to ensure that demands will be met by the station.

INEFFECTIVE: The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH. A reduction in the level of severity is an improvement in the applicable parameters, e.g., Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization.

**ODCM:** Offsite Dose Calculation Manual.

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge. PROJECTILE: An object ejected, thrown, or launched towards a

plant structure. The source of the projectile may be onsite or offsite.

Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein.

**PROTECTED AREA:** Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

**RED PATH:** Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

**RUPTURED:** (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity.

**SABOTAGE:** Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load; (3) Reactor Trip or (4) Safety Injection System Activation.

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-A and 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine).

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i.e., within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e.g., paint chipping, scratches) should NOT be included.

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

Re	EPIP-1 Revision 14 Page 14 of 49					
		1 Loss of Instrumentation		2.2 Loss of Function		
	Mode	Initiating/Condition	Mode	Initiating/Condition		
G E N E R A L		Refer to "Fission Product Barrier Matrix" and "Radiological Effluents" (Section 7)		Refer to "Fission Product Barrier Matrix"		
S - T E	1,2 3,4	<ul> <li>Inability to monitor a SIGNIFICANT TRANSIENT in progress (1 and 2 and 3 and 4)</li> <li>1. Loss of most (&gt;75%) of MCR annunciators (and Annunciator Printer) or indications</li> <li>2. SIGNIFICANT TRANSIENT in progress</li> <li>3. Loss of ICS Computer and SPDS</li> <li>4. Inability to directly monitor any of the following CSFs:</li> </ul>	1,2 3,4	<ul> <li>Complete loss of function needed to achieve or maintain Hot Shutdown (1 or 2)</li> <li>1. CSF status tree indicates Core Cooling Red</li> <li>2. CSF status tree indicates Heat Sink Red (RHR not in service)</li> <li>Note: Also Refer to "Failure of Rx Protection" (2.3) and "Fission Product Barrier Matrix"</li> </ul>		
		Sub-criticality PTS Core Cooling Containment Heat Sink Inventory				
ALERT	1,2 3,4	<ul> <li>UNPLANNED loss of most (&gt;75%) MCR annunciators (and Annunciator Printer) or indications for &gt;15 minutes with either a SIGNIFICANT TRANSIENT in progress or ICS computer and SPDS Unavailable (1 and 2 and 3)</li> <li>1. UNPLANNED loss of most (&gt;75%) MCR annunciators (and Annunciator Printer) or indications for &gt;15 minutes.</li> <li>2. SM/SED Judgment that increased surveillance is required to Safely operate the unit (beyond Shift compliment)</li> <li>3. (a or b) a. SIGNIFICANT TRANSIENT in Progress b. Loss of ICS Computer and SPDS</li> </ul>	4	Complete loss of function needed to achieve Cold Shutdown when Shutdown required by Tech Specs (1 and 2 and 3) 1. Shutdown is required 2. Loss of RHR capability 3. Loss of secondary heat sink and condenser		
JZJSJAL UVER	1,2 3,4	<ul> <li>UNPLANNED loss of most <u>or</u> All Safety System annunciators <u>or</u> indications in the Control Room for &gt;15 Minutes (1 and 2 and 3)</li> <li>1. UNPLANNED loss of most (&gt;75%) MCR annunciators (<u>and</u> Annunciator Printer) <u>or</u> indications for &gt;15 minutes.</li> <li>2. SM/SED Judgment that increased surveillance is required to Safely operate the unit (beyond Shift compliment)</li> <li>3. ICS Computer <u>or</u> SPDS is in service and capable of displaying data requested.</li> </ul>	ALL	<ul> <li>A. Unplanned loss of all In-Plant Communication capability (1 and 2 and 3)</li> <li>1. UNPLANNED loss of EPABX (PAX) phones</li> <li>2. UNPLANNED loss of all sound powered phones</li> <li>3. UNPLANNED loss of all radios <ul> <li>or</li> </ul> </li> <li>B. UNPLANNED loss of all Offsite Communication capability (1 and 2 and 3 and 4 and 5)</li> <li>1. UNPLANNED loss of all EPABX (PAX) phones</li> <li>2. UNPLANNED loss of all EPABX (PAX) phones</li> <li>3. UNPLANNED loss of all OFFsite Communication capability (1 and 2 and 3 and 4 and 5)</li> <li>1. UNPLANNED loss of all PABX (PAX) phones</li> <li>2. UNPLANNED loss of all OPX (Microwave) system</li> <li>4. UNPLANNED loss of all 1 FB-Bell lines</li> <li>5. UNPLANNED loss of all FTS 2000 (NRC) system</li> </ul>		

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3 Failure of Rx Protection			
		2.4 Fuel Clad Degradation	ļ
Initiating/Condition	Mode	Initiating/Condition	
<ul> <li>did <u>not</u> result in a reduction of Rx power to &lt;5% and decreasing (1 and 2)</li> <li>1. (a or b) <ul> <li>a. CSF status tree indicates Core Cooling Red</li> <li>b. CSF status tree indicates Heat Sink Red</li> </ul> </li> <li>2. FR-S.1 entered and subsequent actions <u>Did Not</u> result in</li> </ul>		Refer to "Fission Product Barrier Matrix"	S Y S T E M
a Rx Power of <5% and decreasing Rx power Not <5% and decreasing after VALID Auto and Manual trip signals (1 and 2 and 3) 1. VALID Rx Auto Trip signal received or required 2. Manual Rx Trip from the MCR was Not successful.		Refer to "Fission Product Barrier Matrix"	D E G R A D
		D. J. J. K. K. F. J.	A   T   C   N
<ul> <li>Automatic Rx trip did not occur after VALID Trip signal and manual trip from MCR was successful (1 and 2)</li> <li>1. VALID Rx Auto Trip signal received or required</li> <li>2. Manual Rx Trip from the MCR was successful and power is &lt;5% and decreasing.</li> </ul>		Refer to "Fission Product Barrier Matrix"	
Not Applicable	1,2, 3,4, 5	<ul> <li>Reactor Coolant System specific activity exceeds LCO (Refer to WBN Tech. Spec. 3.4.16)</li> <li>1. Radiochemistry analysis indicates (a or b)</li> <li>a. Dose equivalent iodine (1-131) &gt;1.0 μCi/gm for &gt;48 Hours or in excess of T/S Figure 3.4.16-1</li> <li>b. Specific activity &gt;100/E μCi/gm</li> </ul>	
	<ul> <li>Loss of Core cooling capability and VALID Trip Signals did <u>not</u> result in a reduction of Rx power to &lt;5% and decreasing (1 and 2)</li> <li>1. (a or b) <ul> <li>a. CSF status tree indicates Core Cooling Red</li> <li>b. CSF status tree indicates Heat Sink Red</li> </ul> </li> <li>2. FR-S.1 entered <u>and</u> subsequent actions <u>Did Not</u> result in a Rx Power of &lt;5% and decreasing after VALID Auto and Manual trip signals (1 and 2 and 3)</li> <li>1. VALID Rx Auto Trip signal received or required</li> <li>2. Manual Rx Trip from the MCR was <u>Not</u> successful.</li> <li>3. FR-S.1 has been entered.</li> </ul> Automatic Rx trip did not occur after VALID Trip signal and manual trip from MCR was successful (1 and 2) <ol> <li>VALID Rx Auto Trip signal received or required</li> <li>2. Manual Rx Trip from the MCR was successful and power is &lt;5% and decreasing.</li> </ol>	Automatic Rx trip did not occur after VALID Trip signal and manual trip signal received or required         2. Manual Rx Trip from the MCR was successful and power is <5% and decreasing.	Loss of Core cooling capability and VALID Trip Signals       Refer to "Fission Product Barrier Matrix"         1. (a or b)       a. CSF status tree indicates Core Cooling Red       Refer to "Fission Product Barrier Matrix"         b. CSF status tree indicates Heat Sink Red       Refer to "Fission Product Barrier Matrix"         2. FR-S.1 entered and subsequent actions Did Not result in a Rx Power Of <5% and decreasing

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	RCS Unidentified Leakage	Mode	2.6 RCS Identified Leakage	
Mode	Initiating/Condition	Mode	Initiating/Condition	
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"	
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"	
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"	
J J J J J J J J J J J J J J J J J J J	<ul> <li>Unidentified or pressure boundary RCS leakage &gt;10 GPM</li> <li>1. Unidentified or pressure boundary leakage (as defined by Tech. Spec.) &gt;10 GPM as indicated below (a or b)</li> <li>a. 1-SI-68-32 results</li> <li>b. With RCS Temperature and PZR Level Stable, VCT Level Dropping at a Rate &gt;10 GPM</li> </ul>	1,2, 3,4, *5	Identified RCS leakage >25 GPM 1. Identified RCS leakage (as defined by Tech. Spec.) >25 GPM (a or b) a. 1-SI-68-32 results b. Level rise in excess of 25 GPM total into PRT, RCDT or CVCS Holdup Tank	
	*Note: Applies to Mode 5 if RCS Pressurized		*Note: Applies to Mode 5 if RCS Pressurized	

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]	2.7 Uncontrolled Cooldown 2.8 Turbine Failure				
$\square$	Mode	Initiating/Condition	Mode	Initiating/Condition	<b></b>
GENERA-		Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"	S Y S T E M
с S-те		Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"	D E G R A D A T I O N
LERT		Refer to "Fission Product Barrier Matrix"	1,2,3	Turbine Failure has generated PROJECTILES that cause VISIBLE DAMAGE to any area containing Safety Related equipment         1. Turbine PROJECTILES has resulted in VISIBLE DAMAGE in any of the following areas:         Control Building       Diesel Generator Bldg. Auxiliary Building         RWST       Unit #1 Containment         Intake Pumping Station CST	U 1
UNUSUAL EVE		<ul> <li>UNPLANNED rapid depressurization of the Main Steam System resulting in a rapid RCS cooldown and Safety Injection Initiation (1 and 2)</li> <li>1. Rapid depressurization of Main Steam System (&lt;675 psig)</li> <li>2. Safety Injection has initiated or is required</li> </ul>	1,2,3	<ul> <li>Turbine Failure results in Casing penetration</li> <li>1. Turbine Failure which results in penetration of the Turbine Casing or Damage to Main Generator Seals</li> </ul>	
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	2.	9 Technical Specification
	Mode	Initiating/Condition
G E N E R A L		Not Applicable
S – F E		Not Applicable
ALERT		Not Applicable
UNUSUAL EX NT	1,2 3,4	<ul> <li>Inability to reach required Shutdown within Tech. Spec. limits (1 and 2)</li> <li>1. Any Tech. Spec. LCO Statement, requiring a Mode reduction, has been entered</li> <li>2. The Unit has not been placed in the required Mode within the time prescribed by the LCO Action Statement</li> </ul>

	2.10 Safety Limit
Mode	Initiating/Condition
	Not Applicable
i	
	Not Applicable
	Not Applicable
	Safety Limits have been Exceeded (1 or 2)
1,2, 3,4,	1. The combination of thermal power, RCS temperature, and RCS pressure > safety limits as indicated by WBN Tech.
5	Spec. Figure 2.1.1-1 "Reactor Core Safety Limits"
	<ol> <li>RCS/Pressurizer pressure exceeds safety limit (&gt;2735 psig)</li> </ol>
	-

Page 19 of 49 **FISSION PRODUCT BARRIER MATRIX (Modes 1-4)** 1.1 Fuel Clad RCS 1.2 1.3 Containment SYSTEM DEGRADATION **RCS** Identified Leakage Loss of Instrumentation 2.6 2.12.7 Uncontrolled Cool Down Loss of Function/Communication 2.2 **Turbine** Failure 2.8 Failure of Reactor Protection 2.3 2.9 **Technical Specification** 2.4 Fuel Clad Degradation 2.10 Safety Limit 2.5 RCS Unidentified Leakage LOSS OF POWER Loss of AC (Power Ops) 3.1Loss of AC (Shutdown) 3.2 3.3 Loss of DC **HAZARDS and SED JUDGMENT Control Room Evacuation** Flammable Gas 4.1 Fire 4.3 4.5 Security Explosion 4.4 Toxic Gas 4.6 4.2 4.7 SED Judgment Table 4-1 Table 4-2 Table 4-3 Figure 4-B Figure 4-A **DESTRUCTIVE PHENOMENON** River Level High 5.4 5.1 Earthquake 5.5 River Level Low 5.2 Tornado Watercraft Crash 5.3 Aircraft/Projectile 5.6 Figure 5-A Crash Table 5-1 SHUTDOWN SYSTEM DEGRADATION Loss of Shutdown Systems 6.1 Loss of AC (Shutdown) 6.2 6.3 Loss of DC (Shutdown) Fuel Handling 6.4 RADIOLOGICAL 7.3 Radiation Levels 7.1 Gaseous Effluent Fuel Handling Liquid Effluent 7.4 7.2 Table 7-2 Table 7-1 Figure 7-A

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### **DEFINITIONS/ACRONYMS**

#### UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4.7).

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**ODCM:** Offsite Dose Calculation Manual.

**ORANGE PATH:** Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge.

**PROJECTILE:** An object ejected, thrown, or launched towards a plant structure. The source of the projectile may be onsite or offsite. Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein.

**PROTECTED AREA:** Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

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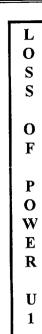
VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e.g., paint chipping, scratches) should NOT be included.

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	3.1	Loss of AC (Power Ops)		3.2 Loss of AC (Shutdown)	
	Mode	Initiating/Condition	Mode	Initiating/Condition	
	1,2, 3,4	<ul> <li>Prolonged loss of Offsite and Onsite AC power (1 and 2)</li> <li>1. 1A and 1B 6.9KV Shutdown Bds de-energized for &gt;15 minutes</li> <li>2. (a or b) <ul> <li>a. Core Cooling Red or Orange</li> <li>b. Restoration of Either 1A or 1B 6.9KV Shutdown Bds is not likely within 4 hours of loss.</li> </ul> </li> </ul>		Not Applicable	
5	1,2, 3,4	Loss of Offsite <u>and</u> Onsite AC Power > 15 minutes 1. 1A and 1B 6.9KV Shutdown Bds de-energized for >15 minutes		Not Applicable	
	1,2, 3,4	Loss of Offsite Power for >15 minutes (1 and 2) 1. C and D CSSTs not available for >15 minutes 2. 1A or 1B Diesel Generator not available	5,6, or De- fuel	UNPLANNED loss of Offsite and Onsite AC power for >15 minutes 1. 1A and 1B 6.9KV Shutdown Bds de-energized for >15 minutes Also Refer to "Loss of Shutdown Systems" (6.1)	
UNUSUAL EVE	1,2 3,4	Loss of Offsite Power for >15 minutes (1 and 2) 1. C and D CSSTs not available for >15 minutes 2. Each Diesel Generator is supplying power to its respective Shutdown Board	5,6, or De- fuel	<ul> <li>UNPLANNED loss of Offsite Power for &gt;15 minutes (1 and 2)</li> <li>1. C and D CSSTs not available for &gt;15 minutes</li> <li>2. Either Diesel Generator is supplying power to its respective Shutdown Board</li> </ul>	

TRANSPORT OF T

3.		.3 Loss of DC Power			
	Mode	Initiating/Condition			
G E N E R A L		Refer to "Fission Product Barrier Matrix" and "Loss of Function" (2.2)			
S – T E	1,2, 3,4	Loss of All Vital DC Power for >15 minutes 1. Voltage <105V DC on 125V DC Vital Battery Buses 1-I and 1-II and 1-III and 1-IV for >15 minutes Also Refer to "Fission Product Barrier Matrix", "Loss of Function" (2.2), and "Loss of Instrumentation" (2.1)			
A L E R T		Also Refer to "Fission Product Barrier Matrix", "Loss of Function" (2.2), and "Loss of Instrumentation" (2.1)			
UNUSUAL EV	5,6, or De-fuel	<ul> <li>UNPLANNED Loss of the Required Train of DC power for &gt;15 minutes (1 or 2)</li> <li>1. Voltage &lt;105V DC on 125V DC Vital Battery Buses 1-I and 1-III for &gt;15 minutes</li> <li>2. Voltage &lt;105V DC on 125V DC Vital Battery Buses 1-II and 1-IV for &gt;15 minutes</li> </ul>			



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## **DEFINITIONS/ACRONYMS**

#### UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4.7).

BOMB: An explosive device (see EXPLOSION).

CIVIL DISTURBANCE: A group of twenty (20) or more persons violently protesting station operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs: Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

**EVENT:** Assessment of an **EVENT** commences when recognition is made that one or more of the conditions associated with the event exist. Implicit in this definition is the need for timely assessment, i.e. within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): The demarcation of the area surrounding the WBN units in which postulated FSAR accidents will not result in population doses exceeding the criteria of 10 CFR Part 100. Refer to Figure 7-A.

**EXPLOSION:** A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation.

EXTORTION: An attempt to cause an action at the station by threat of force.

FAULTED: (Steam Generator) Existence of secondary side leakage (i.e., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition.

HOSTAGE: A person(s) held as leverage against the station to ensure that demands will be met by the station.

INEFFECTIVE: The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH. A reduction in the level of severity is an improvement in the applicable parameters, e.g., Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization.

**ODCM:** Offsite Dose Calculation Manual.

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge. PROJECTILE: An object ejected, thrown, or launched towards a plant structure. The source of the projectile may be onsite or offsite. Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein.

PROTECTED AREA: Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

**RED PATH:** Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

**RUPTURED:** (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity.

**SABOTAGE:** Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load; (3) Reactor Trip or (4) Safety Injection System Activation.

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-Aand 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine).

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

**VALID:** An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i.e., within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e.g., paint chipping, scratches) should NOT be included.

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

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	4.1 FIRE		4.2 Explosions
Mode	Initiating/Condition	Mode	Initiating/Condition
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
4 L	Refer to "Control Room Evacuation," (4.5) or		Refer to "Fission Product Barrier Matrix"
S I E	Fission Product Barrier Matrix"		
A L E All R T	<ul> <li>FIRE in any of the areas listed in Table 4-1 that is affecting Safety Related equipment (1 and 2)</li> <li>1. FIRE in any of the areas listed in Table 4-1</li> <li>2. (a or b)</li> <li>a. VISIBLE DAMAGE to permanent structure <u>or</u> Safety Related equipment in the specified area is observed due to the FIRE</li> <li>b. Control Room indication of degraded Safety System <u>or</u> component response due to the FIRE</li> </ul>	All	<ul> <li>EXPLOSION in any of the areas listed in Table 4-1 that is affecting Safety Related equipment (1 and 2)</li> <li>1. EXPLOSION in any of the areas listed in Table 4-1</li> <li>2. (a or b) <ul> <li>a. An EXPLOSION has caused VISIBLE DAMAGE to Safety Related equipment</li> <li>b. Control Room indication of degraded Safety System or component response due to the EXPLOSION</li> </ul> </li> </ul>
U N U S U All A	FIRE in the PROTECTED AREA threatening any of the areas listed in Table 4-1 that is <u>Not</u> extinguished within 15 minutes from the Time of Control Room notification <u>or</u> verification of Control Room Alarm (Figure 4-A)	All	Refer to "Security" (4.6) UNPLANNED EXPLOSION within the PROTECTED AREA resulting in VISIBLE DAMAGE to any permanen structure <u>or</u> equipment (Figure 4-A) Refer to "Security" (4.6)

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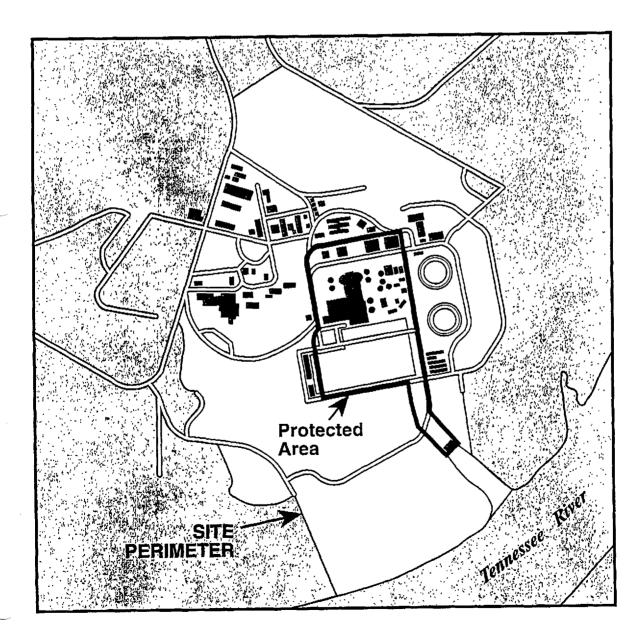
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## TABLE 4-1 PLANT AREAS ASSOCIATED WITH FIRE AND EXPLOSION EALS

Unit #1 Reactor Building Auxiliary Building Control Building Diesel Generator Building CST Additional Diesel Generator Building Intake Pumping Station Additional Equipment Buildings (Unit 1&2) RWST

## Figure 4-A PROTECTED AREA/SITE PERIMETER



Re	PIP-1 evision 14			
Pa	ge 27 of 49	4.3 Flammable Gas		4.4 Toxic Gas
	Mode	Initiating/Condition	Mode	Initiating/Condition
G E N E R A L		Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
L S I T E		Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
ALERT	All	<ul> <li>UNPLANNED release of Flammable Gas within a facility structure containing Safety Related equipment <u>or</u> associated with Power production</li> <li>Plant personnel report the average of three readings taken in a ~10ft triangular Area is &gt;25% (LEL) Lower Explosive Limit, as indicated on the monitoring instrument within any building listed in Table 4-2.</li> </ul>	All	<ul> <li>Release of TOXIC GAS within a facility structure which Prohibits Safe Operation of systems required to establish or maintain Cold S/D (1 and 2 and 3)</li> <li>1. Plant personnel report TOXIC GAS within any building listed in Table 4-2</li> <li>2. (a or b) <ul> <li>a. Plant personnel report Severe Adverse Health Reactions due to TOXIC GAS (i.e., burning eyes, nose, throat, dizziness)</li> <li>b. Sampling indications &gt; (PEL) Permissible Exposure Limit</li> </ul> </li> <li>3. Plant personnel would be unable to perform actions necessary to establish and maintain Cold Shutdown while utilizing appropriate personnel protection equipment.</li> </ul>
UNUSUAL EVENT	All	<ul> <li>A. UNPLANNED release of Flammable Gas within the SITE PERIMETER</li> <li>1. Plant personnel report the average of three readings taken in a ~10ft Triangular Area is &gt;25% (LEL) Lower Explosive Limit, as indicated on the monitoring instrument within the SITE PERIMETER (Refer to Figure 4-B) <ul> <li><u>OR</u></li> </ul> </li> <li>B. Confirmed report by Local, County, or State Officials that a Large Offsite Flammable Gas release has occurred within One Mile of the Site with potential to enter the SITE PERIMETER in concentrations &gt;25% of LEL Lower Explosive Limit (Refer to Figure 4-B)</li> </ul>	All	<ul> <li>A. Normal Operations impeded due to access restrictions caused by TOXIC GAS concentrations within a Facility Structure listed in Table 4-2 <ul> <li>OR</li> </ul> </li> <li>B. Confirmed report by Local, County, or State Officials that a Large Offsite TOXIC GAS release has occurred within One Mile of the Site with potential to enter the Site Perimeter in concentrations &gt;than the (PEL) Permissible Exposure Limit thus causing an Evacuation (Figure 4-B)</li> </ul>

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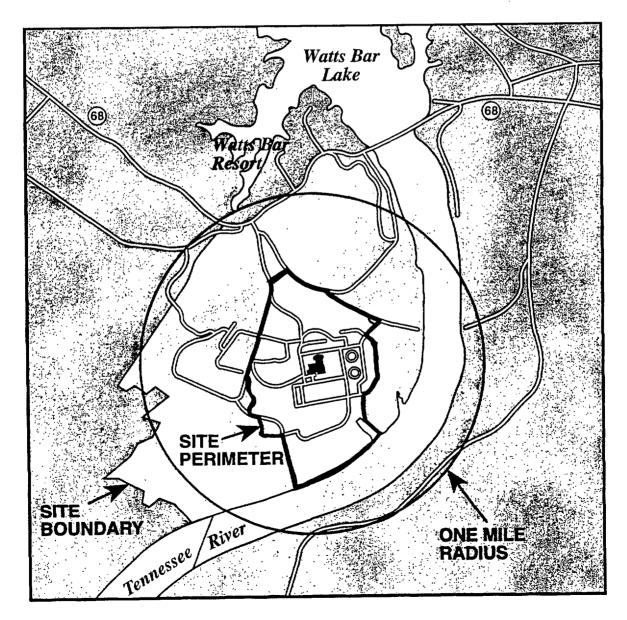
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# TABLE 4-2 Plant Structures Associated With TOXIC or Flammable Gas EALs

Unit #1 & 2 Reactor Buildings Auxiliary Building Control Building Diesel Generator Building Additional Diesel Generator Building Intake Pumping Station Additional Equipment Bldgs (Unit 1&2) CDWE Building Turbine Building

Figure 4-B ONE MILE RADIUS/SITE PERIMETER



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	4.5 C	ontrol Room Evacuation		4.6 Security
7	Mode	Initiating/Condition	Mode	Initiating/Condition
G E N E R A		Refer to "Fission Product Barrier Matrix"	All	<ol> <li>Security Event resulting in loss of Control of the Plant</li> <li>Hostile Armed Force has taken Control of the Plant, Control Room, or Remote shutdown capability</li> </ol>
L S I F E	All	<ul> <li>Evacuation of the Control Room has been initiated and Control of all necessary equipment <u>Has Not</u> been established within 15 minutes of manning the Auxiliary Control Room (1 and 2 and 3)</li> <li>1. (a or b) <ul> <li>a. AOI-30.2 "Fire Safety Shutdown" entered</li> <li>b. AOI-27 "Main Control Room Inaccessibility" entered</li> </ul> </li> <li>2. SM/SED Orders Control Room evacuation</li> <li>3. Control has <u>Not</u> been established at the Remote Shutdown Panel within 15 minutes of manning the Auxiliary Control Room and transfer of switches on Panels L11A and L11B</li> </ul>	All	<ul> <li>Security Event has <u>or</u> is occurring which results in Actual <u>or</u> Likely Failures of Plant Functions needed to Protect the Public</li> <li>1. VITAL AREA, other than the Control Room, has been penetrated by a Hostile Armed Force</li> </ul>
A L E R T	All	<ul> <li>Evacuation of the Control Room is Required (1 and 2)</li> <li>1. (a or b) <ul> <li>a. AOI-30.2 "Fire Safe Shutdown" entered</li> <li>b. AOI-27 "Main Control Room Inaccessibility" entered</li> </ul> </li> <li>2. SM/SED Orders Control Room evacuation</li> </ul>	All	<ul> <li>Confirmed Security Event which indicates an Actual or Potential Substantial Degradation in the level of Safety of the Plant (1 or 2 or 3)</li> <li>1. BOMB discovered within a VITAL AREA</li> <li>2. CIVIL DISTURBANCE ongoing within the PROTECTED AREA</li> <li>3. PROTECTED AREA has been penetrated by a Hostile Armed Force</li> <li>Refer to Figure 4-A For a Drawing of Protected Area and Site Perimeter</li> </ul>
U N U S U A L E V E N T		Not Applicable	All	<ul> <li>Confirmed Security Event which indicates a Potential Degradation in the level of Safety of the Plant (1 or 2)</li> <li>1. BOMB discovered within the PROTECTED AREA</li> <li>2. Security Shift Supervisor reports one or more of the events listed in Table 4-3</li> </ul>

-4	.7 Em	ergency Director Judgment
<b>آ</b> بر	Mode	Initiating/Condition
G E N E R A L	All	Events are in progress <u>or</u> have occurred which involve Actual <u>or</u> Imminent Substantial Core Degradation <u>or</u> Melting With Potential for Loss of Containment Integrity. Releases can be reasonable expected to exceed EPA Plume Protective Action Guidelines Exposure Levels outside the EXCLUSION AREA BOUNDARY, Refer to Figure 7-A.
S I T E	All	Events are in progress <u>or</u> have occurred which involve Actual <u>or</u> Likely Major Failures of Plant Functions needed for the Protection of the Public. Any releases are not expected to result in Exposure Levels which Exceed EPA Plume Protective Action Guidelines Exposure Levels outside the EXCLUSION AREA BOUNDARY, Refer to Figure 7-A.
LERT	All	Events are in progress <u>or</u> have occurred which involve Actual <u>or</u> Potential Substantial Degradation of the Level of Safety of the Plant. Any releases are expected to be limited to small fractions of the EPA Plume Protective Action Guidelines Exposure Levels.
	All	Unusual Events are in Progress <u>or</u> have occurred which indicate a Potential Degradation of the Level of Safety of the Plant. No releases of Radioactive Material requiring Offsite Response <u>or</u> Monitoring are expected unless further degradation of Safety Systems occurs.

## Table 4-3 SECURITY EVENTS

- a. SABOTAGE/INTRUSION has occurred <u>or</u> is occurring within the PROTECTED AREA
- b. HOSTAGE/EXTORTION Situation that Threatens to interrupt Plant Operations
- c. CIVIL DISTURBANCE ongoing between the SITE PERIMETER and PROTECTED AREA
- d. Hostile STRIKE ACTION within the PROTECTED AREA which threatens to interrupt Normal Plant Operations (Judgment Based on behavior of Strikers and/or Intelligence received)

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1.1 1.2 1.3	SION PRODUCT BARRIER MATRIX (Modes 1-4) Fuel Clad RCS Containment	1
<b>SYS</b> 2.1 2.2 2.3 2.4 2.5	STEM DEGRADATIONLoss of Instrumentation2.6Loss of Function/Communication2.7Failure of Reactor Protection2.8Fuel Clad Degradation2.9RCS Unidentified Leakage2.10Safety Limit	2
LO 3.1 3.2 3.3	SS OF POWER Loss of AC (Power Ops) Loss of AC (Shutdown) Loss of DC	3
<b>HA</b> 4.1 4.2	ZARDS and SED JUDGMENTFire4.3Flammable Gas4.5Control Room EvacuationExplosion4.4Toxic Gas4.6SecurityTable 4-1Table 4-24.7SED JudgmentFigure 4-AFigure 4-BTable 4-3	4
DE 5.1 5.2 5.3	STRUCTIVE PHENOMENON         Earthquake       5.4       River Level High         Tornado       5.5       River Level Low         Aircraft/Projectile       5.6       Watercraft Crash         Crash       Figure 5-A         Table 5-1       Figure 5-A	5
-	UTDOWN SYSTEM DEGRADATION Loss of Shutdown Systems Loss of AC (Shutdown) Loss of DC (Shutdown) Fuel Handling	6
<b>RA</b> 7.1 7.2	DIOLOGICALGaseous Effluent7.3Radiation LevelsLiquid Effluent7.4Fuel HandlingTable 7-1Table 7-2Figure 7-A	7

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## **DEFINITIONS/ACRONYMS**

## UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4.7).

BOMB: An explosive device (See EXPLOSION).

**CIVIL DISTURBANCE:** A group of twenty (20) or more persons violently protesting station operations or activities at the site.

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs: Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

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**EXPLOSION:** A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation.

EXTORTION: An attempt to cause an action at the station by threat of force.

FAULTED: (Steam Generator) Existence of secondary side leakage (i.e., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition.

**HOSTAGE:** A person(s) held as leverage against the station to ensure that demands will be met by the station.

**INEFFECTIVE:** The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH. A reduction in the level of severity is an improvement in the applicable parameters, e.g., Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization.

ODCM: Offsite Dose Calculation Manual.

**ORANGE PATH:** Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge.

**PROJECTILE:** An object ejected, thrown, or launched towards a plant structure. The source of the projectile may be onsite or offsite. Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein.

**PROTECTED AREA:** Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

**RED PATH:** Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

**RUPTURED:** (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity.

**SABOTAGE:** Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback > 15% thermal reactor power, (2) Electrical load rejection > 25% full electrical load; (3) Reactor Trip or (4) Safety Injection System Activation.

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-A and 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine).

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i.e., within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e.g., paint chipping, scratches) should NOT be included.

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

	5.1 Earthquake		5.2 Tornado
Mode	Initiating/Condition	Mode	Initiating/Condition
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
All	<ul> <li>Earthquake detected by site seismic instrumentation (1 and 2)</li> <li>1. (a and b) <ul> <li>a. Ann.166 D indicates "OBE Spectra Exceeded"</li> <li>b. Ann.166 E indicates "Seismic Recording Initiated"</li> </ul> </li> <li>2. (a or b) <ul> <li>a. Ground motion sensed by Plant personnel</li> <li>b. National Earthquake Information Center at 1-(303) 273-8500 can confirm the event.</li> </ul> </li> </ul>	All	<ul> <li>Tornado or High Winds strikes any structure listed Table 5-1 and results in VISIBLE DAMAGE (1 and 2)</li> <li>1. Tornado or High Winds (Sustained &gt;80 mph &gt; one minute) strikes any structure listed in Table 5-1</li> <li>2. (a or b) <ul> <li>a. Confirmed report of any VISIBLE DAMAGE</li> <li>b. Control Room indications of degraded Safety System or component response due to event</li> </ul> </li> <li>Note: Site Met Data Instrumentation fails to 0 at &gt;100 m National Weather Service Morristown 1-(423) 586-8400</li> </ul>
All	<ul> <li>Earthquake detected by site seismic instrumentation (1 and 2)</li> <li>1. Ann. 166 E indicator "Seismic Recording Initiated"</li> <li>2. (a or b) <ul> <li>a. Ground motion sensed by Plant personnel</li> <li>b. National Earthquake Information Center at 1-(303) 273-8500 can confirm the event.</li> </ul> </li> </ul>	All	<ul> <li>provide additional information if needed.</li> <li>Tornado within the SITE PERIMETER</li> <li>1. Plant personnel report a Tornado has been sighted withe SITE PERIMETER (Refer to Figure 5-A)</li> </ul>
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		Aircraft/Projectile Crash
	Mode	Initiating/Condition
G E N E R A L		Refer to "Fission Product Barrier Matrix"
S I T E		Refer to "Fission Product Barrier Matrix"
		Aircraft or PROJECTILE impacts (Strikes) any
A L E R T	All	<ul> <li>Plant structure listed in Table 5-1 resulting in VISIBLE DAMAGE (1 and 2)</li> <li>1. Plant personnel report aircraft or PROJECTILE has impacted any structure listed in Table 5-1</li> <li>2. (a or b) <ul> <li>a. Confirmed report of any VISIBLE DAMAGE</li> <li>b. Control Room indications of degraded Safety System or component response due to the event within the specified areas</li> </ul> </li> </ul>
U		Aircraft crash or PROJECTILE impact within
		the SITE PERIMETER  1. Plant personnel report a Aircraft Crash <u>or</u> PROJECTILE impact within the SITE PERIMETER (Refer to Figure 5-A)

#### Table 5-1 Plant Structures Associated With Tornado/Hi Wind and Aircraft EALs

Unit #1 and 2 Reactor Buildings Auxiliary Building Control Building Diesel Generator Building Additional Diesel Generator Building Intake Pumping Station Additional Equipment Buildings (Units 1 & 2) CDWE Building Turbine Building RWST CST

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	5.4 River Level HIGH		5.5 River Level LOW
Mode	Initiating/Condition	Mode	Initiating/Condition
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
All	River Reservoir level is at Stage II Flood Warning (1 or 2)         1. River Reservoir level >727 Ft         2. Stage II Flood Warning (AOI-7) has been issued by River Systems Operations	All	River Reservoir level is <668 Ft (AOI-22) as reporte River Systems Operations
All	River Reservoir level is at Stage I Flood Warning (1 or 2 or 3)         1. River Reservoir level >726.5 Ft from April 16 thru September 30         2. River Reservoir level >714.5 Ft from October 1 thru April 15	All	River Reservoir level is ≤673 Ft (AOI-22) as report River Systems Operations
	3. Stage I Flood Warning (AOI-7) has been issued by River Systems Operations		

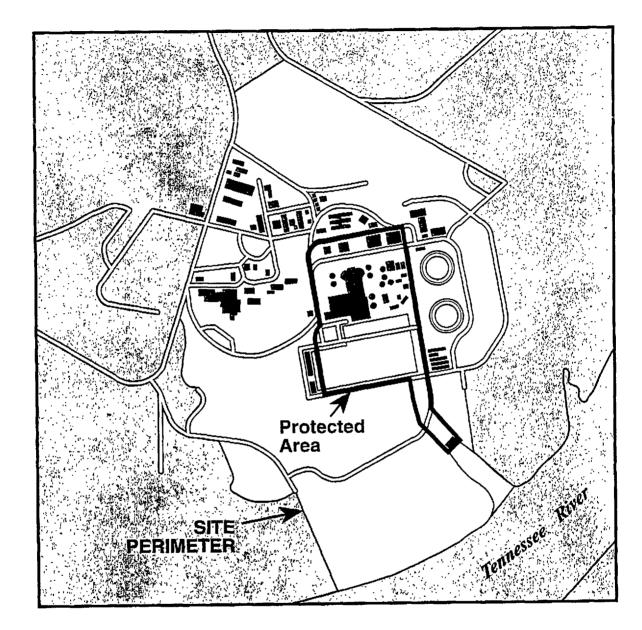
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	Mode	5.6 Watercraft Crash Initiating/Condition
GENERAL		Refer to "Fission Product Barrier Matrix"
S – F E		Refer to "Fission Product Barrier Matrix"
A L E R T		Refer to "Fission Product Barrier Matrix"
UNUSUAL EVENT	All	<ul> <li>Watercraft Strikes the Intake Pumping Station resulting in a reduction of Essential Raw Cooling Water (ERCW) or Raw Cooling Water (RCW) (1 and 2)</li> <li>Plant personnel report a Watercraft has struck the Intake Pumping Station</li> <li>(a or b or c) <ul> <li>a. ERCW Supply Header Pressure Train A</li> <li>O-PI-67-18A is &lt;15 psig</li> </ul> </li> <li>b. ERCW Supply Header Pressure Train B</li> <li>O-PI-67-17A is &lt;15 psig</li> <li>c. RCW Supply Header Pressure O-PI-24-22 is &lt;15 psig</li> </ul>



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Figure 5-A PROTECTED AREA/SITE PERIMETER



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HAZARDS and SED JUDGMENT4.1Fire4.3Flammable Gas4.5Control Room Evacuation4.2Explosion4.4Toxic Gas4.6SecurityTable 4-1Table 4-24.7SED JudgmentFigure 4-AFigure 4-BTable 4-3	<sup>m</sup> 4
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#### **DEFINITIONS/ACRONYMS**

#### UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4.7).

BOMB: An explosive device (See EXPLOSION).

**CIVIL DISTURBANCE:** A group of twenty (20) or more persons violently protesting station operations or activities at the site.

**CRITICAL-SAFETY FUNCTION (CSFs):** A plant safety function required to prevent significant release of core radioactivity to the environment. There are six **CSFs**: Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS).

**EVENT:** Assessment of an **EVENT** commences when recognition is made that one or more of the conditions associated with the event exist. Implicit in this definition is the need for timely assessment, i.e. within 15 minutes.

EXCLUSION AREA BOUNDARY (EAB): The demarcation of the area surrounding the WBN units in which postulated FSAR accidents will not result in population doses exceeding the criteria of 10 CFR Part 100. Refer to Figure 7-A.

EXPLOSION: A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation.

EXTORTION: An attempt to cause an action at the station by threat of force.

FAULTED: (Steam Generator) Existence of secondary side leakage (i.e., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition.

HOSTAGE: A person(s) held as leverage against the station to ensure that demands will be met by the station.

**INEFFECTIVE:** The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH. A reduction in the level of severity is an improvement in the applicable parameters, e.g., Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization.

**ODCM:** Offsite Dose Calculation Manual.

**ORANGE PATH:** Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge.

**PROJECTILE:** An object ejected, thrown, or launched towards a plant structure. The source of the projectile may be onsite or offsite. Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein.

**PROTECTED AREA:** Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

**RED PATH:** Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

**RUPTURED:** (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity.

**SABOTAGE:** Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load; (3) Reactor Trip or (4) Safety Injection System Activation.

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-Aand 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine).

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

**VALID:** An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i.e., within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e.g., paint chipping, scratches) should NOT be included.

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

	6.1 L	oss of Shutdown Systems		6.2 Loss of AC (Shutdown)
	Mode	Initiating/Condition	Mode	Initiating/Condition
G E N E R A L	5,6	Note: Additional information will be provided later pending NRC Guidance on Shutdown EALs Refer to "Gaseous Effluents" (7.1)		Not Applicable
S – T E	5,6	Loss of water level in the Rx vessel that has <u>or</u> will uncover fuel in the Rx vessel with CNTMT closure established (1 and 2 and 3 and 4 and 5) 1. Loss of RHR capability 2. Rx vessel water level < el. 718' 3. Incore TCs (if available) indicate RCS temp. >200° F 4. RCS is vented/open to CNTMT 5. CNTMT closure is established Note: If CNTMT open, refer to "Gaseous Effluents"		Not Applicable
ALERT	5,6	<ul> <li>(7.1)</li> <li>Inability to maintain Unit in Cold Shutdown</li> <li>(1 and 2 and 3)</li> <li>1. RHR capability is <u>not</u> available for RCS Cooling</li> <li>2. Incore TCs (if available) indicate RCS temp. &gt;200° F</li> <li>3. CNTMT closure is established</li> </ul>	5,6 or De- Fuel	<ul> <li>UNPLANNED loss of Offsite and Onsite AC Power for &gt;15 minutes</li> <li>1. 1A and 1B 6.9 KV Shutdown Bds de-energized for &gt;15 minutes</li> </ul>
UNUSUAL EVENT	5,6	Note: Additional information will be provided later pending NRC Guidance on Shutdown EALs	5,6 or De- Fuel	<ul> <li>UNPLANNED loss of All Offsite Power for &gt;15 minutes (1 and 2)</li> <li>1. C and D CSSTS not available For &gt;15 minutes.</li> <li>2. Either Diesel Generator is supplying power to its respective Shutdown Board</li> </ul>

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		oss of DC (Shutdown)
	Mode	Initiating/Condition
GENERAL		Not Applicable
SITE		Not Applicable
ALERT		Not Applicable
UNUSUAL EVENT	or De- fuel	<ul> <li>UNPLANNED loss of the required Train of DC Power for &gt;15 minutes (1 or 2)</li> <li>1. Voltage &lt;105V DC on 125V DC Vital Battery Buses 1-1 and 1-III for &gt;15 minutes</li> <li>2. Voltage &lt;105V DC on 125V DC Vital Battery Buses 1-II and 1-IV for &gt;15 minutes.</li> </ul>

Mode	uel Handling Initiating/Condition
	Refer to "Gaseous Effluents" (7.1)
	Refer to "Gaseous Effluents" (7.1)
	Major damage to Irradiated Fuel, <u>or</u> Loss
All	of water level that has <u>or</u> will uncover irradiated Fuel outside the Reactor Vessel (1 and 2) 1. VALID alarm on O-RE-90-101 <u>or</u> O-RE-90-102 <u>or</u> O-RE-90-103 <u>or</u> 1-RE-90-130/131 <u>or</u> 1-RE-90-112 <u>or</u> 1-RE-90-400 <u>or</u> 2-RE-90-400 2. (a or b) a. Plant personnel report damage of Irradiated Fuel sufficient to rupture Fuel Rods b. Plant personnel report water level drop has <u>or</u> will exceed makeup capability such that Irradiated Fuel will be
All	uncovered         UNPLANNED loss of water level in Spent Fuel Pool or Reactor Cavity or Transfer Canal with fuel remaining covered (1 and 2 and 3)         1. Plant personnel report water level drop in Spent Fuel Pool or Reactor Cavity, or Transfer Canal         2. VALID alarm on O-RE-90-102 or O-RE-90-103 or 1-RE-90-59 or 1-RE-90-60         3. Fuel remains covered with water
All	Transfer Canal 2. VALID alarm on O-RE-90-102 <u>or</u> O-RE-90-103 <u>or</u> 1-RE-90-59 <u>or</u> 1-RE-90-60

FISSION PRODUCT BARRIER MATRIX (Modes 1-4) 1.1 Fuel Clad 1.2 RCS 1.3 Containment	1
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DESTRUCTIVE PHENOMENON5.1Earthquake5.4River Level High5.2Tornado5.5River Level Low5.3Aircraft/Projectile5.6Watercraft Crash Figure 5-ACrashFigure 5-ATable 5-1	5
SHUTDOWN SYSTEM DEGRADATION6.1Loss of Shutdown Systems6.2Loss of AC (Shutdown)6.3Loss of DC (Shutdown)6.4Fuel Handling	6
RADIOLOGICAL         7.1       Gaseous Effluent       7.3       Radiation Levels         7.2       Liquid Effluent       7.4       Fuel Handling         Table 7-1       Table 7-2       Figure 7-A	7

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EPIP-1 Revision 14 Page 42 of 49 **DEFINITIONS/ACRONYMS** 

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SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-Aand 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine).

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i.e., within 15 minutes.

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VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

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		7.1 Gaseous Effluents
	Mode	Initiating/Condition
G E N E R A L	All	<ul> <li>EAB dose resulting from an actual <u>or</u> imminent release of Gaseous Radioactivity that exceeds 1000 mrem TEDE <u>or</u> 5000 mrem Thyroid CDE for the actual <u>or</u> projected duration of the release (1 or 2 or 3)</li> <li>A VALID rad monitor reading exceeds the values under General in Table 7-1 for &gt;15 minutes, unless assessment within this time period confirms that the Criterion is <u>Not</u> exceeded.</li> <li>Field survey results indicate &gt;1000 mrem/hr β-γ <u>or</u> an I-131 concentration of 3.9E-6 μ Ci/cc at SP</li> <li>EP dose assessment results indicate EAB dose &gt;1000 mrem TEDE <u>or</u> &gt;5000 mrem Thyroid CDE for the actual <u>or</u> projected duration of the release (Figure 7-A)</li> </ul>
S – T E	All	<ul> <li>EAB dose resulting from an actual <u>or</u> imminent release of Gaseous Radioactivity that exceeds 100 mrem TEDE <u>or</u> 500 mrem Thyroid CDE for the actual <u>or</u> projected duration of the release (1 or 2 or 3)</li> <li>A VALID rad monitor reading exceeds the values under Site in Table 7-1 for &gt;15 minutes, unless assessment within this time period confirms that the Criterion is <u>Not</u> exceeded</li> <li>Field survey results indicate &gt;100 mrem/hr β-γ <u>or</u> an I-131 concentration of 3.9E-7 μ Ci/cc at SP</li> <li>EP dose assessment results indicate EAB dose &gt;100 mrem TEDE <u>or</u> &gt;500 mrem Thyroid CDE for the actual <u>or</u> projected duration of the release (Figure 7-A)</li> </ul>
A L E R T	All	<ul> <li>Any UNPLANNED release of Gaseous Radioactivity that exceeds 200 times the ODCM Limit for &gt;15 minutes (1 or 2 or 3)</li> <li>1. A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for &gt;15 minutes, unless assessment within this time period confirms that the Criterion is <u>Not</u> exceeded</li> <li>2. Field survey results indicate &gt;10 mrem/hr β-γ at SP &gt;15 minutes</li> <li>3. EP dose assessment results indicate EAB dose &gt;10 mrem TEDE for the duration of the release (Figure 7-A)</li> </ul>
UNUSUAL EVENT		<ul> <li>Any UNPLANNED release of Gaseous Radioactivity that exceeds 2 times the ODCM Limit for &gt;60 minutes (1 or 2 or 3)</li> <li>1. A VALID rad monitor reading exceeds the values under UE in Table 7-1 for &gt;60 minutes, unless assessment within this time period confirms that the Criterion is <u>Not</u> exceeded</li> <li>2. Field survey results indicate &gt;0.1 mrem/hr β-γ at SP for &gt;60 minutes</li> <li>3. EP dose assessment results indicate EAB dose &gt;0.1 mrem TEDE for the duration of the release (Figure 7-A)</li> </ul>

Initiating/Condition
Not Applicable
Not Applicable
<ul> <li>Any UNPLANNED release of Liquid Radioactivity that exceeds 200 times the ODCM Limit for &gt;15 minutes (1 or 2)</li> <li>A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for &gt;15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded.</li> <li>Sample results exceed 200 times the ODCM limit value for an unmonitored release of liquid radioactivity &gt;15 minutes in duration</li> </ul>
<ul> <li>Any UNPLANNED release of Liquid Radioactivity to the Environment that exceeds 2 times the ODCM Limit for &gt;60 minutes (1 or 2)</li> <li>A VALID rad monitor reading exceeds the values under UE in Table 7-1 for &gt;60 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded.</li> <li>Sample results exceed 2 times the ODCM limit value for an unmonitored release of liquid radioactivity &gt;60 minutes in duration</li> </ul>

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# TABLE 7-1 EFFLUENT RADIATION MONITOR EALS<sup>(1)</sup>

NOTE:

The values below, **if exceeded**, indicate the need to perform the specified assessment. If the assessment can not be completed within 15 minutes (60 minutes for UE), the declaration shall be made based on the **VALID** reading. As used here, the radiation monitor indications as displayed on **ICS** are the primary indicators. If **ICS** is unavailable, utilize the radiation monitor readings in the control room or local indication as necessary.

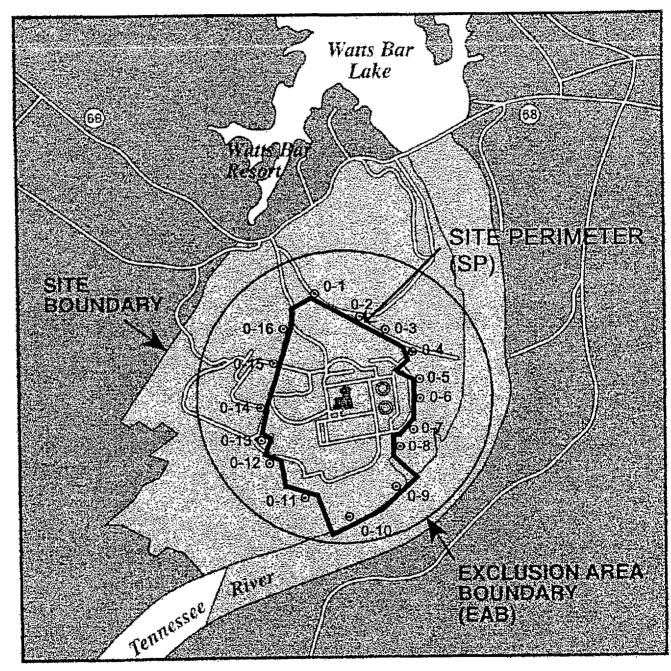
Monitor	ICS Screen	Units	UE	Alert	Site	General
Total Site	EFF1	μCi/s <sup>(2)</sup>	1.5E+05	1.5E+07	2.5E+08	2.5E+09
		T				1
U1 Shield Building 1-RE-90-400	EFF1	μCi/s	6.7E+04	6.7E+06	1.0E+08	1.0E+09
1-INE-20-400				1	· · · · · · · · · · · · · · · · · · ·	
U2 Shield Building				1		
2-RE-90-400	EFF1	μCi/s	1.5E+04	1.5E+06	2.5E+07	2.6E+08
				T I	1	1
Auxiliary Building				!	1	
0-RE-90-101B	4RM1	cpm	1.2E+04	1.2E+06	*****(1)	*****(])
				,	1 '	1
Service Building	(D) (1		4 215 1 02	125105	9.8E+06	*****{]}
0-RE-90-132B	4RM1	cpm	4.3E+03	4.3E+05	9.85700	1.1.1.1.1.1.1.1.1.1.1.1.1.1.1.1.1.1.1.
U1 Condenser Vacuum						
Exhaust	212 ላ አፈ	µCi/cc <sup>(3)</sup>	5.5E-02	5.5E+00	8.83E+01	8.83E+02
1-RE-90-404A	3PAM 3PAM	•	5.5E-02 5.5E-02	5.5E+00	8.83E+01	8.83E+02
1-RE-90-404B	3PAIvi	μCi/cc	5.515-02	5.515100	0.052.01	0.002 01
S/G Discharge						
Monitors	4RM2	mR/hr <sup>(4)</sup>	NA	3.5E+02	3.5E+03	3.5E+04
1-RE-90-421 thru 424 (B)	41/11/2	muvm	1477	5.52.02		
Liquid Monitors	n/a	µCi/ml <sup>(2)</sup>	1.8E-05	1.8E-03	N/A	N/A
0-RE-90-122	4RM2	cpm	1.1E+06	*****(1)	N/A	N/A
1-RE-90-120,121	4RM2	cpm	1.0E+06	*****(1)	N/A	N/A
0-RE-90-225	4RM2	cpm	9.2E+05	*****(1)	N/A	N/A
0-RE-90-212	4RM2	cpm	1.5E+04	1.5E+06	N/A	N/A
RELEASE DURATION			60	15	15	15

Note: (1) Table values are calculated values. The \*\*\*\*\* indicates the monitor is off scale.

- (2) These releases rate values in µCi/s and µCi/ml are provided on the gaseous and liquid release points for <u>Information Only</u>. Actual monitor readings are given in the table corresponding to the monitor for the **four** emergency classifications.
- (3) This eberline channel reads out in cpm in the MCR. Indications of a radioactivity release via this pathway would be S/G blowdown monitors or other indications of primary-to-secondary leakage such as S/G level increase or pressurizer level decrease. ICS calculates μCi/cc and has a visual indication of an alarm condition when the indications exceeds 5.5E-02μCi/cc. This channel was included in the table to provide a means to further assess a release detected by other indications and to provide a path for possible escalation.
- (4) These unit values are based on flow rates through one [1] PORV of 970,000 lb/hr at 1,185 psig, 600°F. Before using these values, ensure a release to the environment is ongoing (e.g. PORV).

## Figure 7-A EXCLUSION AREA, SITE BOUNDARY and SITE PERIMETER

NOTE: The Site Boundary used here is consistent with the definition in the Offsite Dose Calculation Manual. Do Not confuse this boundary with the SITE PERIMETER defined in these EALs, or with other definitions of "Site Boundary."



Note: Numbered points are [SP] radiological survey point for all sectors.

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		7.3 Radiation Levels		7.4 Fuel Handling
M	iode	Initiating/Condition	Mode	Initiating/Condition
G E N E R A L		Refer to "Fission Product Barrier Matrix" or "Gaseous Effluents" (7.1)		Refer to "Gaseous Effluents" (7.1)
L S I T E		Refer to "Fission Product Barrier Matrix" or "Gaseous Effluents" (7.1)		Refer to "Gaseous Effluents" (7.1)
A L E R T	All	<ul> <li>UNPLANNED increases in Radiation levels within the Facility that impedes Safe Operations <u>or</u> establishment <u>or</u> maintenance of Cold Shutdown (1 or 2)</li> <li>1. VALID area Radiation Monitor readings <u>or</u> survey results exceed 15 mrem/hr in the Control Room <u>or</u> CAS</li> <li>2. (a and b) <ul> <li>a. VALID area radiation monitor readings exceed values listed in Table 7-2</li> <li>b. Access restrictions impede operation of systems necessary for Safe Operation <u>or</u> the ability to establish Cold Shutdown</li> </ul> </li> <li>See UNUSUAL EVENT Note Below</li> </ul>	All	<ul> <li>Major damage to Irradiated Fuel, or Loss of water level that has or will uncover Irradiated Fuel outside the Reactor Vessel (1 and 2)</li> <li>1. VALID alarm on 0-RE-90-101 or 0-RE-90-102 or 0-RE-90-103 or 1-RE-90-130/131 or 1-RE-90-112 or 1-RE-90-400 or 2-RE-90-400</li> <li>2. (a or b) <ul> <li>a. Plant personnel report damage of Irradiated Fuel sufficient to rupture Fuel Rods</li> <li>b. Plant personnel report water level drop has or will exceed makeup capacity such that Irradiated Fuel will be uncovered</li> </ul> </li> </ul>
UNUSUAL EVENT	All	<ul> <li>UNPLANNED increase in Radiation levels within the Facility</li> <li>1. VALID area Radiation Monitor readings increase by a factor 1000 over normal levels</li> <li>Note: In Either the UE or ALERT EAL, the SED must determine the cause of Increase in Radiation Levels and Review Other INITIATING/CONDITIONS for Applicability (e.g., a dose rate of 15 mrem/hr in the Control Room could be caused by a release associated with a DBA).</li> </ul>	All	<ul> <li>UNPLANNED loss of water level in Spent Fuel Pool or Reactor Cavity or Transfer Canal with fuel remaining covered (1 and 2 and 3)</li> <li>1. Plant personnel report water level drop in Spent Fuel Pool, or Reactor Cavity, or Transfer Canal</li> <li>2. VALID alarm on 0-RE-90-102 or 0-RE-90-103 or 1-RE-90-59 or 1-RE-90-60</li> <li>3. Fuel remains covered with water.</li> </ul>

R A D I O L O G

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#### Table 7-2

## **ALERT - RADIATION LEVELS**

······································	Location	Monitor
Monitor No.	<b>Building and Elevation</b>	Reading *
1&2 RE-90-1	Auxiliary El. 757.0	$2.5 \times 10^3 \text{ mR/hr}$
	(spent fuel pool)	
1-RE-90-2	Auxiliary El. 757.0	2.5 x 10 <sup>0</sup> R/hr
	(personnel air lock)	
0-RE-90-3	Auxiliary El. 729.0	$2.5 \times 10^3 \text{ mR/hr}$
	(waste pac. area)	
0-RE-90-4	Auxiliary El. 713.0	$1.5 \ge 10^3 \text{ mR/hr}$
	(decon room)	
0-RE-90-5	Auxiliary El. 737.0	$1.5 \ge 10^3 \text{ mR/hr}$
	(spt. fuel pool pmp. ar.)	)
1&2-RE-90-6	Auxiliary El. 737.0	$1.5 \times 10^3 $ mR/hr
	(comp. cl. wtr. ht. ex. ar	
1&2-RE-90-7	Auxiliary El. 713.0	$2 \ge 10^3 \text{ mR/hr}$
	(sample room)	
1&2-RE-90-8	Auxiliary El. 713.0	$1.5 \ge 10^3 \text{ mR/hr}$
	(aux. feed pump area)	
0-RE-90-9	Auxiliary El. 692.0	$1.5 \times 10^3 $ mR/hr
	(wst. cond. evap. tk. ar.	
1&2-RE-90-10	Auxiliary El. 692.0	$1.5 \ge 10^3 \text{ mR/hr}$
	(cvcs area)	
0-RE-90-11	Auxiliary El. 676.0	$1.5 \ge 10^3 \text{ mR/hr}$
	(ctmt. spry. & rhr pmp a	
1-RE-90-61	Auxiliary El. 736.0	$2.5 \times 10^3 \text{ mR/hr}$
	(RB low. cmpt. inst. rm	
0-RE-90-230	Turbine El. 685.0	$1.5 \times 10^3 $ mR/hr
	(conden. demin.)	
0-RE-90-231	Turbine El. 685.0	$1.5 \text{ x } 10^3 \text{ mR/hr}$
	(conden. demin.)	

Note: \*These monitors read out in mR/hr. It is assumed that this is equivalent to mrem/hr.

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## SOURCE NOTES

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<ol> <li>NIR-0551, DV-847100 F00012, and MC- 850321 809004, MSC-00956, NCO 920030366.</li> </ol>	Monitor readings and challenges to barriers are provided in EPIP-1, Section 1 in (1.1 Fuel Clad 1.1.5 and 1.3 CNTMT Barrier 1.3.5), Section 7 (7.1 Gaseous Effluents, 7.2 Liquid Effluents, Table 7-1, 7.3 Radiation Levels, 7.4 Fuel Handling and Table 7-2). Barriers are covered in Section 1, Fission Product Barrier Matrix. Monitor readings are also provided in EPIP-5, App. B, Note 3.
2. MC-84 0827 005 035A, MCS-2400	SED duties that can not be delegated. Section 2.0 Responsibility.
3. MC-8407 1900 3003, MSC-00701, NCO- 920030222 CNTMT	Rad Monitors used in conjunction with a plant parameter to determine emergency classifications. Monitor readings are included with plant parameters for the purposes of emergency classifications. Section 1, Fission Product Barrier Matrix (1.1 Fuel Clad, 1.2 RCS, 1.3 Containment), Section 7 (7.1 Gaseous Effluent, 7.2 Liquid Effluent and 7.3 Radiation Levels and 7.4 Fuel Handling).
4. ANSI Standard N.18.7-1976 Subsection 5.3.9.3: 01 POI	EPIPs will contain the following elements.
<ol> <li>5. MSC-02401, NCO-920030998</li> <li>6. EPPOS #2</li> </ol>	Chemistry detection of failed fuel. Emergency Preparedness Position (EPPOS) on timeliness of classification of emergency conditions.