

Carolina Power & Light Company

Robinson Nuclear Plant 3581 West Entrance Road Hartsville SC 29550

Serial: RNP-RA/00-0074

APR 2 5 2000

United States Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 DOCKET NO. 50-261/LICENSE NO. DPR-23

TRANSMITTAL OF COPIES OF AMENDMENT 186 TO THE OPERATING LICENSE AND TECHNICAL SPECIFICATIONS AND REVISION 14 TO THE TECHNICAL SPECIFICATIONS BASES

Ladies and Gentlemen:

This letter transmits copies of replacement pages to the Operating License (OL), Technical Specifications (TSs), TSs Bases, and associated Lists of Effective Pages (LEPs) for the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. The attached copies incorporate Amendment No. 186 to the OL and TSs, and incorporate Revision 14 to the TSs Bases. Amendment No. 186 revised Surveillance Requirement (SR) 3.7.6.2 to require verification that each required Component Cooling Water pump starts automatically on an actual or simulated Loss of Power Diesel Generator Start Undervoltage Signal rather than on an Engineered Safety Feature Actuation Signal. This amendment was implemented on February 14, 2000. Revision 14 to the TSs Bases represents changes associated with Amendment No. 186.

One (1) copy is provided for Mr. B. R. Bonser at NRC Region II, one (1) copy is provided for the NRC Resident Inspector, and three (3) copies are provided for Mr. R. Subbaratnam at NRC Headquarters. The recipients are requested to remove and insert pages in accordance with the instructions provided.

The attachment provides copies of replacement pages to the LEP for the OL and TSs, the facility OL, the TSs, the LEP for the TSs Bases, and the TSs Bases, and includes instructions for removing and inserting the pages on the cover page.

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If you have any questions concerning this matter, please contact Mr. H. K. Chernoff.

Sincerely,

RL. Warden

Manager - Regulatory Affairs

EBS/ebs Attachment

c: L. A. Reyes, NRC, Region II (w/o attachment)

B. R. Bonser, NRC, Region II

R. Subbaratnam, NRC, NRR (3 copies attachment)

NRC Resident Inspector, HBRSEP

United States Nuclear Regulatory Commission Attachment to Serial: RNP-RA/00-0074 9 Pages

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 REPLACEMENT PAGES FOR THE OPERATING LICENSE, TECHNICAL SPECIFICATIONS, BASES, AND LISTS OF EFFECTIVE PAGES

Instructions for Removal and Insertion of Pages to the Lists of Effective Pages, Operating License, Technical Specifications, and Bases

Replace the following pages of the Lists of Effective Pages (LEPs), Operating License, Technical Specifications, and Bases with the enclosed pages. The revised areas are indicated by margin lines.

<u>Remove</u>	<u>Insert</u>
LEP for Operating License and Technical Specifications	
Page 1 Page 6	Page 1 Page 6
Facility Operating License	
Page 3 Page 4d	Page 3 Page 4d
Technical Specifications	
Page 3.7-17	Page 3.7-17
LEP for Technical Specifications Bases	
Page i.1 Page i.11	Page i.1 Page i.11
Technical Specifications Bases	
Page B 3.7-39	Page B 3.7-39

OPERATING LICENSE EFFECTIVE PAGES

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Title Page List of Effective Pages 1 2 3 4 5 6 7	- 186 181 181 181 182 186 181 185
Technical Specifications Interpretations LEP	1
Facility Operating License DPR-23	Amendment No.
1 2 3 4 4a 4b 4c	186
4d	186

TECHNICAL SPECIFICATIONS EFFECTIVE PAGES

<u>Appendix A</u>

<u>Page</u>	Amendment No.
3.6-13 3.6-14 3.6-15 3.6-16 3.6-17 3.6-18 3.6-19 3.6-20 3.6-21	176 176 176 176 176 176 176 176
3.7-1 3.7-2 3.7-3 3.7-4 3.7-5 3.7-6 3.7-7 3.7-8 3.7-9 3.7-10 3.7-11 3.7-12 3.7-15 3.7-16 3.7-17 3.7-18 3.7-20 3.7-21 3.7-21 3.7-21 3.7-22 3.7-23 3.7-24 3.7-25 3.7-25 3.7-26 3.7-27 3.7-28 3.7-29 3.7-30 3.7-31 3.7-32 3.7-33	176 176 176 176 176 176 176 176 176 176
3.8-1 3.8-2 3.8-3 3.8-4 3.8-5 3.8-6	176 176 176 176 176 176

3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20. Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Section 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at a steady state reactor core power level not in excess of 2300 megawatts thermal.

B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 186 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(1) For Surveillance Requirements (SRs) that are new in Amendment 176 to Final Operating License DPR-23, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 176. For SRs that existed prior to Amendment 176, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 176.

C. Reports

Carolina Power & Light Company shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

Carolina Power & Light Company shall keep facility operating records in accordance with the requirements of the Technical Specifications.

3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20. Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Section 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at a steady state reactor core power level not in excess of 2300 megawatts thermal.

B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 186 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(1) For Surveillance Requirements (SRs) that are new in Amendment 176 to Final Operating License DPR-23, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 176. For SRs that existed prior to Amendment 176, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 176.

C. Reports

Carolina Power & Light Company shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

Carolina Power & Light Company shall keep facility operating records in accordance with the requirements of the Technical Specifications.

4. Additional Conditions

The Additional Conditions contained in Appendix B, as revised through Amendment No. 186, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the additional conditions.

5. This license is effective as of the date of issuance and shall expire at midnight July 31, 2010.

Attachment Appendix A - Technical Specifications

Date of Issuance: JUL 31 1970

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.7.6.1	Isolation of CCW flow to individual components does not render the CCW System inoperable. Verify each required CCW manual, power operated, and automatic valve in the flow path servicing safety related equipment, that is not locked, sealed, or otherwise secured in position, is in the correct position.	31 days
SR 3.7.6.2	Verify each required CCW pump starts automatically on an actual or simulated LOP DG Start undervoltage signal.	18 months

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

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i i.1 i.2 i.3 i.4 i.5 i.6 i.7 i.8 i.9 i.10 i.11 i.12 i.13 ii iii iv B 2.0-1 B 2.0-2 B 2.0-3 B 2.0-4 B 2.0-5 B 2.0-7 B 2.0-9 B 3.0-1 B 3.1-2 B 3.1-3 B 3.1-6 B 3.1-7 B 3.1-7 B 3.1-8 B 3.1-7 B 3.1-7 B 3.1-8 B 3.1-7	0 14 7 E1 7 E1 7 E1 7 E1 7 E1 11 10 7 E1 14 13 7 E1 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0

TECHNICAL SPECIFICATIONS BASES EFFECTIVE PAGES

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B 3.7-39 B 3.7-40 B 3.7-41 B 3.7-42 B 3.7-43 B 3.7-45 B 3.7-45 B 3.7-47 B 3.7-49 B 3.7-50 B 3.7-51 B 3.7-52 B 3.7-53 B 3.7-55 B 3.7-55 B 3.7-56 B 3.7-60 B 3.7-61 B 3.7-62 B 3.7-63 B 3.7-63 B 3.7-64 B 3.7-65 B 3.7-65 B 3.7-67 B 3.7-68 B 3.7-69	14 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0
B 3.7-73 B 3.7-74 B 3.7-75 B 3.7-76 B 3.7-77	0 0 0 0 0 0 0
B 3.7-78 B 3.7-79 B 3.7-80 B 3.8-1 B 3.8-2 B 3.8-3 B 3.8-4	0
B 3.7-71 B 3.7-72 B 3.7-73 B 3.7-74 B 3.7-75 B 3.7-76 B 3.7-77 B 3.7-78 B 3.7-79 B 3.7-80 B 3.8-1 B 3.8-2 B 3.8-3 B 3.8-4 B 3.8-5 B 3.8-7 B 3.8-7	0 0 0 0 0

ACTIONS

B.1 and B.2 (continued)

allowed Completion Times are reasonable, based on operating experience, to reach the required unit conditions from full power conditions in an orderly manner and without challenging unit systems.

SURVEILLANCE REQUIREMENTS

SR 3.7.6.1

This SR is modified by a Note indicating that the isolation of the CCW flow to individual components may render those components inoperable but does not affect the OPERABILITY of the CCW System.

Verifying the correct alignment for manual, power operated, and automatic valves in the required CCW flow path provides assurance that the proper flow paths exist for CCW operation. This SR does not apply to valves that are locked, sealed, or otherwise secured in position, since these valves are verified to be in the correct position prior to locking, sealing, or securing. This SR also does not apply to valves that cannot be inadvertently misaligned, such as check valves. This Surveillance does not require any testing or valve manipulation; rather, it involves verification that those valves capable of being mispositioned are in the correct position.

The 31 day Frequency is based on engineering judgment, is consistent with the procedural controls governing valve operation, and ensures correct valve positions.

SR 3.7.6.2

This SR verifies proper automatic operation of the required CCW pumps on an actual or simulated LOP DG Start undervoltage signal. The CCW System is a normally operating system that cannot be fully actuated as part of routine testing during normal operation. The 18 month Frequency is based on the need to perform this Surveillance under the conditions that apply during a unit outage and the potential for an unplanned transient if the Surveillance were performed with the reactor at power. Operating experience has shown that these components usually pass the Surveillance when performed at

(continued)