

April 28, 2000 LIC-00-0042

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Mail Station P1-137 Washington, DC 20555

References:

- 1. Docket No. 50-285
- 2. WCAP-13027-P, "Westinghouse ECCS Evaluation Model for Analysis of CE-NSSS," dated July 1991
- 3. Letter from OPPD (R. L. Phelps) to NRC (Document Control Desk), dated April 28, 1999 (LIC-99-0036)
- 4. Letter from OPPD (S. K. Gambhir) to NRC (Document Control Desk), dated October 29, 1999 (LIC-99-0099)

SUBJECT: Annual Report for 1999 Loss of Coolant Accident (LOCA)/Emergency Core Cooling System (ECCS) Models Pursuant to 10 CFR 50.46

In accordance with 10 CFR 50.46(a)(3)(ii), the Omaha Public Power District (OPPD) is submitting the annual 10 CFR 50.46 summary report for 1999. This summary report updates all identified changes or errors in the LOCA/ECCS codes and methods used by Westinghouse Electric Company (\underline{W}) to model Fort Calhoun Station Unit No. 1 (FCS). Reference 2 describes the methodology used by \underline{W} to model Combustion Engineering plants, such as FCS.

OPPD has received the \underline{W} "10 CFR 50.46 Annual Notification and Reporting for 1999." There were no new reportable Peak Clad Temperature (PCT) impacts for the NOTRUMP (small break) evaluation model analyses, and the PCT margin utilization summary contained in Reference 3 remains valid (i.e., PCT=1446° F). There were also no new reportable PCT impacts for the BART (large break) evaluation model analyses, and the PCT margin utilization summary contained in Reference 4 remains valid (i.e., PCT=2094° F). In summary, the FCS PCT values for small and large break LOCAs remain less than the 10 CFR 50.46(b)(1) acceptance criterion of 2200° F.



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Please contact me if you have any questions.

Sincerely,

S. K. Gambhir Division Manager Nuclear Operations

SKG/brh

c: E. W. Merschoff, NRC Regional Administrator, Region IV
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