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SEPTEMBER 30-OCTOBER 2, 1999

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REPORTS, LETTERS, AND MEMORANDA

REPORTS

- Draft Commission Paper Regarding Proposed Guidelines for Applying Risk-Informed Decisionmaking In License Amendment Reviews (Report to Greta Joy Dicus, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated October 8, 1999)
- Combustion Engineering Owners Group (CEOG) Application to Eliminate the Post-Accident Sampling System from the Plant Design Bases for CEOG Utilities (Report to Greta Joy Dicus, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated October 8, 1999)
- Proposed Plans for Developing Risk-Informed Revisions to 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities" (Report to Greta Joy Dicus, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated October 12, 1999)

LETTERS

- Proposed Resolution of Generic Safety Issue B-55, "Improved Reliability of Target Rock Safety Relief Valves" (Letter to William D. Travers, Executive Director for Operations, NRC, from Dana A. Powers, Chairman, ACRS, dated October 8, 1999)
- Proposed Resolution of Generic Safety Issue 23 (GSI-23), "Reactor Coolant Pump Seal Failure" (Letter to William D. Travers, Executive Director for Operations, NRC, from Dana A. Powers, Chairman, ACRS, dated October 8, 1999)

MEMORANDUM

- Proposed Final Revision 3 to Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants" (Memorandum to William D. Travers, Executive Director for Operations, NRC, from John T. Larkins, Executive Director, ACRS, dated October 5, 1999)

APPENDICES

- I. Federal Register Notice
- II. Meeting Schedule and Outline
- III. Attendees
- IV. Future Agenda and Subcommittee Activities
- V. List of Documents Provided to the Committee

MINUTES OF THE 466TH MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
SEPTEMBER 30-OCTOBER 2, 1999
ROCKVILLE, MARYLAND

The 466th meeting of the Advisory Committee on Reactor Safeguards was held in Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on September 30-October 2, 1999. The purpose of this meeting was to discuss and take appropriate action on the items listed in the following meeting minutes. The meeting was open to public attendance. There were no written statements nor requests for time to make oral statements from members of the public regarding the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the Gelman Building, 2120 L Street, N.W., Washington, D.C. [Copies of the transcript are available for purchase from Ann Riley & Associates, Ltd., 1025 Connecticut Avenue, N.W., Suite 1014, Washington, D.C. 20036, and on the ACRS/ACNW Web Page at www.NRC.gov/ACRS/ACNW.]

ATTENDEES

ACRS Members: Dr. Dana A. Powers (Chairman), Dr. George Apostolakis (Vice-Chairman), Mr. John Barton, Dr. Mario V. Bonaca, Dr. Thomas S. Kress, Dr. William J. Shack, Dr. Robert L. Seale, Mr. John D. Sieber, Dr. Robert E. Uhrig, and Dr. Graham B. Wallis. [For a list of other attendees, see Appendix III.]

I. Chairman's Report (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Dana A. Powers, Committee Chairman, convened the meeting at 8:30 a.m. and reviewed the schedule for the meeting. He summarized the agenda topics for this meeting and discussed the administrative items for consideration by the full Committee.

II. NRC Safety Research Program (Open)

[Note: Dr. Medhat El-Zeftawy was the Designated Federal Official for this portion of the meeting.]

The Committee heard a briefing from representatives of the Office of Nuclear Regulatory Research (RES) regarding a vision for RES in accomplishing the statutory mission of the NRC. The vision states that RES conducts independent analyses and develops technical bases for realistic regulatory decisions that protect public health and safety and prepare the agency for the future by evaluating safety issues involving current and new designs and technologies.

The RES vision has three critical aspects: it is independent, exhibits realism in decisionmaking, and it is forward-looking.

RES representatives described the role of RES as (1) the performance of activities that complement the NRC's front-line regulatory activities and (2) independent examination of evolving technology and anticipated issues. In addition, RES will maintain a center of excellence for technical expertise and for developing and maintaining regulatory tools and databases.

The Committee also discussed certain options in preparing the Year 2000 (Y2K) Advisory Committee on Reactor Safeguards (ACRS) report to the Commission regarding the RES safety research program. Dr. Wallis presented four options for the report as follows:

- Option 1** - In Option 1, the report would be like previous reports
- A. It would review performance (1998 and 1999 plans)
 - B. It would review intent (2000 and 2001 plans)

Option 2 - In Option 2, the report would be like previous reports but would also address a few key areas.

Option 3 - In option 3, the report would restructure "areas" to reflect Commission concerns, for example, license renewal, risk-informing regulation, fuels.

Option 4 - In Option 4, the report would be a much shorter high-level report designed for the Commission, would avoid reviewing details, and would be more like an extended ACRS letter than a NUREG. Also, it would make recommendations that had a possibility of being implemented.

The Committee discussed these four options and voted to use Option 4.

Conclusion

This briefing was presented for information only.

III. Combustion Engineering Owners Group (CEOG) Proposal to Eliminate the Post-Accident Sampling System Requirements (Open)

[Note: Mr. Paul A. Boehnert was the Designated Federal Official for this portion of the meeting.]

Dr. Kress, cognizant AGRS member for this issue, introduced this topic to the Committee. He noted that the Severe Accident Management (SAM) Subcommittee reviewed this matter during its meeting on September 16-17, 1999. Dr. Kress said that the Committee had previously reviewed a similar proposal from the Westinghouse Owners Group (WOG) for elimination of the Post-Accident Sampling System (PASS) requirements from Westinghouse plants. The Committee wrote two letters relating to the WOG review expressing concern about the issues of monitoring the pH of containment sump water and the monitoring of radiation in the containment atmosphere given a severe accident. He suggested that the Committee consider two questions: Is the information provided by the PASS needed and Is the PASS appropriately configured to obtain the needed information?

Presentation by the Combustion Engineering Owners Group

Mr. R. Schneider, ABB/CE, discussed the Combustion Engineering Owners Group (CEOG) proposal for relaxation of the PASS requirements. The CEOG is proposing to eliminate all PASS requirements from the licensing basis of CEOG plants and discontinuation of operation and maintenance activities on PASS components. The following arguments have been posed for supporting this action:

- Information from PASS is not needed for accident management (alternative in-plant instruments and guidance are more appropriate for taking actions and making decisions)
- The use of PASS is not risk-beneficial (it diverts resources away from accident management; it affects as low as reasonably achievable [ALARA])
- System maintenance costs are high and are rapidly increasing ("Y2K" costs are substantial; the systems are aging)

The issue of monitoring of containment sump pH water was extensively discussed. Dr. Kress noted that the CEOG was granted relief from this requirement in 1993 through NRC approval of a CEOG license amendment request. Drs. Powers and Wallis questioned how the licensee would be able to ensure that the pH value in the sump water would be kept high enough (>7) to prevent revolitization of iodine, given that the amount of acids entering the sump is not known. Mr. Schneider said that analysis has shown that the amount of tri-sodium phosphate located in the sump provides adequate margin. Dr. Powers said he was perplexed as to how a licensee would ensure adequate pH control absent actual measurement, particularly after the accident had progressed for several days.

Regarding radiation monitoring of the containment atmosphere, the CEOG said that the central concern is the escape of iodine from containment. To address this concern, field monitoring teams would track any release plume.

NRC Presentations

Messrs. J. O'Brien (Office of Nuclear Reactor Regulation [NRR]) discussed the status of the staff's review of the CEOG's proposed PASS license amendment request. The review team, background, and preliminary review findings were noted. The staff's findings are that the CEOG's proposal to eliminate the PASS requirements from CEOG plant licensing bases is acceptable. Two issues are still being reviewed however: (1) the need for sampling of radionuclides in the reactor coolant system, the containment atmosphere, and the sump and (2) the need for measurement of containment sump water pH. Regarding issue 2, NRR noted that it met separately with Drs. Powers and Kress to gain further understanding of their concerns, and additional staff action is being reviewed. For issue 1, the staff has four options being considered, ranging from maintenance of existing criteria to the elimination of this requirement. The staff also intends to inform affected State emergency response organizations about the above four options being considered.

Mr. R. Hasselberg (Incident Response Operations) posed a hypothetical accident situation that could result in a significant release of radioactive materials. He then posited a series of questions from the President on down to local officials that by implication argued for obtaining as much sampling information as possible. Dr. Kress said that this matter would ultimately rest on the staff's position with regard to issue 1 above. Dr. Powers said that most of the questions posited can be reliably answered with the tools currently at the licensee's command (assuming no PASS),

with the exception of issue 2 above. Dr. Bonaca said that he is concerned about the loss of measurement capability if the PASS is abandoned.

Conclusion

The Committee issued a report on this matter to Chairman Dicus, dated October 8, 1999.

IV. Rulemaking and Review Plans Associated with the Development of Risk-Informed Revisions to 10 CFR Part 50 (Open)

[Note: Mr. Michael T. Markley was the Designated Federal Official for this portion of the meeting.]

Dr. Apostolakis, Chairman of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment (RPRA), introduced this topic to the Committee and stated that the purpose of this meeting was to discuss proposed plans for developing risk-informed revisions to 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities." He noted that the Subcommittees on RPRA and on Regulatory Policies and Practices met with representatives of the NRC staff and the Nuclear Energy Institute (NEI) on July 13 and September 24, 1999, to discuss these matters. The Committee also met with a representative of Public Citizen, Critical Mass Energy Project, during the Subcommittee meeting on September 24, 1999. Dr. Apostolakis stated that the Subcommittees discussed the staff's proposal to develop a new regulatory section 10 CFR 50.69 and an associated Appendix T to implement the Option 2 rulemaking (changing the special treatment rules of 10 CFR Part 50) and the staff's plan for implementing the Option 3 study (changing specific requirements in the body of 10 CFR Part 50) of SECY-98-300. Dr. Apostolakis also noted that he had made a brief presentation to the Subcommittees on the proper use of importance measures.

NRC Staff Presentation

Messrs. Thomas Bergman and Timothy Reed, NRR, led the staff discussions of the proposed plan for implementing Option 2. Messrs. Scott Newberry, Michael Cheok, and Ms. Cynthia Carpenter, NRR, provided supporting discussion. Mr. Thomas King and Ms. Mary Drouin discussed the proposed plan for implementing Option 3. Significant points made during the presentations include the following:

- For Option 2, the staff proposes to develop a new rule 10 CFR 50.69 and an associated Appendix T. The new rule will allow the use of a new risk-informed scope and Appendix T will provide criteria for the categorization process.
- The staff proposes to preserve the current terminology of safety-related structures, systems, and components (SSCs) and to use additional terminology, that is, high-safety significance and low-safety significance. The staff proposes to use a 2X2 matrix in which SSCs are to be placed in one of four categories according to their safety significance and their safety-related status.
- For Option 3, the staff plans to (1) identify candidate requirements and design-basis accidents (DBAs) to be considered, (2) define the bases for improving current requirements and DBAs, (3) prioritize candidate requirements and DBAs, and (4) identify proposed changes to the regulations.
- Application of 10 CFR 50.69 and Appendix T by the licensee is voluntary.
- The staff plans to develop goals, objectives, and criteria for the pilots. Proposed pilots include South Texas, San Onofre, Arkansas Nuclear One, and Fermi.

NEI Presentation

Messrs. Steven Floyd and Adrian Heymer, NEI, discussed the industry's views regarding proposed revisions to 10 CFR Part 50. Significant points made during the presentation include the following:

- The primary industry objective for revising 10 CFR Part 50 is to reduce "unnecessary regulatory burden."
- The industry is overly constrained by measures, that is., collecting and developing plant performance measures rather than using risk information to improve decisionmaking.
- NEI plans to produce a guidance document incorporating many of the features previously planned for submittal in a petition for rulemaking. NEI

proposes a reevaluation of the scope of SSCs on a system basis and a recategorization of SSCs without additional regulatory treatment, that is, SSCs that are risk significant but not safety related would be controlled by licensee-defined controls and commitments rather than by regulatory requirements.

- NEI agrees with the staff's approach to 10 CFR Part 50 with respect to revising design bases under Option 3. NEI provided a list of candidate regulations for the staff to consider as priority items under Option 3, including 10 CFR 50.34 content of applications (e.g., Three Mile Island [TMI] requirements), 10 CFR 50.36 Technical Specifications (e.g., limiting conditions for operation, operability versus functionality, etc.), and 10 CFR 50.44 for hydrogen recombiners.

Public Citizen Presentation

Mr. Jim Riccio, staff attorney for Public Citizen's Critical Mass Energy Project, made a brief presentation to the Committee. He gave his views on NRC and industry efforts pertaining to risk-informed regulation and a recent report issued by Public Citizen entitled "Amnesty Irrational - How the Nuclear Regulatory Commission Fails to Hold Nuclear Reactors Accountable for Violations of Its Own Safety Regulations." Significant points made during the presentation include the following:

- Risk-informing of the regulations in 10 CFR Part 50 is another attempt by the NRC and the nuclear power industry to deregulate safety standards not on the basis of safety but on cost. The NRC and the industry appear to have reverted to a pre-TMI production mind-set (as opposed to a safety mind-set), in part, based on the manipulation of performance indicators.
- PRAs are not valid because nuclear power plants have not been designed, constructed, operated, or maintained in accordance with the design basis. The absence of any recent reactor core meltdown does not preclude one from happening, and the use of PRA masks the potential for core-melt accidents. PRA is being misused to save industry costs. NRC and NEI efforts to make 10 CFR Part 50 risk informed are "short sighted and will place the public at greater risk."
- The NRC should stay with the present version of 10 CFR Part 50. Making 10 CFR Part 50 risk informed will result in a less safe industry.

Dr. Powers questioned how the staff would consider different levels of risk that are of neither high nor low safety significance (i.e., some medium category of risk). He also questioned how the staff would consider the various operating modes other than full-power operation (e.g., low-power and shutdown operations). The staff stated that it had not fully decided on a final set of categories and noted that the South Texas Nuclear Operating Company had selected four categories. The staff stated that it would be appropriate to consider the critical event sequences for low-power and shutdown operations risk but that it would be difficult to model all possible configurations.

Dr. Apostolakis noted that the determination of safety significance of SSCs relies heavily on the use of importance measures (e.g., Fussell-Vesely and Risk Achievement Worth). He noted the limitations of importance measures and suggested that the guidance in Appendix T be modified to clarify the proper roles of (a) importance measures, (b) sensitivity and uncertainty analysis, (c) baseline core damage frequency (CDF) and large, early release frequency (LERF), and changes in CDF and LERF (i.e., Δ CDF and Δ LERF). He also suggested that the staff should include guidance for conducting expert panel sessions and training on the use of importance measures. The staff agreed to consider Dr. Apostolakis' comments in the draft Commission paper and associated materials.

Dr. Wallis questioned Public Citizen's criticism of performance indicators as not being a representative sample of performance and as being manipulated by the industry to alter perceived performance. Dr. Wallis suggested that Public Citizen appears to be guilty of extrapolating limited data or operating experience for the purpose of criticizing the industry. Mr. Riccio stated that the use of PRAs by the NRC and the industry is deplorable because the plant design bases are not maintained. Dr. Wallis suggested that PRA may be the best tool for reducing the probability of a core meltdown.

Conclusion

The Committee issued a report on this matter to Chairman Dicus dated October 12, 1999.

- V. Proposed Resolution of Generic Safety Issue (GSI)-148, "Smoke Control and Manual Fire Fighting Effectiveness" (Open)

[Note: Mr. Amarjit Singh was the Designated Federal Official for this portion of the meeting.]

Mr. John J. Barton, Acting Chairman of the Fire Protection Subcommittee, introduced this topic to the Committee. He stated that the purpose of this session was to discuss with representatives of the NRC staff the proposed resolution of Generic Safety Issue (GSI) 148, "Smoke Control and Manual Fire Fighting Effectiveness."

NRC Staff Presentations

Mr. Alan Rubin led the discussions for the staff. He presented a brief background and the proposed resolution of GSI 148. He stated that GSI 148 has been classified as a "licensing issue," with the resolution being handled through plant-specific reviews as requested as a part of the individual plant examination of external events (IPEEE) program. Since smoke is one of the major contributions to increased fire brigade response times and can hamper Operations' ability to safely shut down the plant, some of the IPEEE participants have discussed how fire brigade members are trained in handling smoke. To date, the NRC staff has completed its review of 26 licensee IPEEE submittals out of 70 and has documented the results in safety evaluation reports. The staff stated that 22 licensees responded to the smoke management issue. The staff has proposed a document, "Review Guidance for Generic Safety Issue 148, which will also include guidance for smoke control and effectiveness. The staff also stated that additional recent efforts by RES are underway to fully address the risk impact of smoke on safety-related equipment.

The staff concluded that with the review of IPEEE submittals to date, the guidance document, and the research activities in the area of smoke propagation, it has proposed to close out this GSI and consider it resolved without recommending any safety enhancements or other regulatory action.

Conclusion

The Committee deferred the letter to the Executive Director for Operations on this matter to the November 1999 ACRS meeting.

VI. Proposed Resolution of Generic Safety Issue (GSI)-23, "Reactor Coolant Pump Seal Failures" (Open)

[Note: Mr. Paul A. Boehnert was the Designated Federal Official for this portion of the meeting.]

Dr. Wallis, cognizant ACRS member for this issue, introduced this topic to the Committee. He said that GSI-23 is almost 20 years old and was initiated following a large number of reactor coolant pump (RCP) seal failures during normal operation. A major seal failure can result in a several hundred gpm leak rate, which can lead to core uncover if no action is taken. Improvements have been made by the industry with the use of better seal material and more reliable seal cooling systems. No major seal failures have been noted in the last 10 years. NRC argues that given this improvement, this issue is no longer a GSI and can be closed out. Questions for consideration include whether an adequate amount of improvement in seal reliability has been obtained and how many plants need additional attention.

NRC Staff Presentations

Mr. J. Craig gave the opening remarks, noting that in 1995 the Commission rejected rulemaking to resolve this issue because, among other reasons, this matter is a plant-specific concern. The staff has been examining this issue for the station blackout (SBO) and the loss of component cooling water and essential service water (CCW/ESW) events.

Details of the staff's resolution of GSI-23 were presented by Messrs. M. Cunningham, J. Jackson, and A. Buslik. Key points included the following:

- The scope of GSI-23 includes consideration of SBO (10 CFR 50.63), and loss of CCW/ESW events. The staff's evaluation showed that all plants meet the intent of the SBO rule (i.e., the CDF industry average of 10-5/year). For the analysis of CCW/ESW, a few plants have been identified as outliers and when its reviews are complete, the staff expects that about 10 plants will be subject to plant-specific reviews and/or backfit analysis.
- The staff developed a model authored by D. Rhodes, AECL, for evaluation of seal failures for Westinghouse RCPs. The staff's assumptions for two types of seal failure events (loss of O-ring integrity and seal "pop open") were noted. The pop-open event results in the highest leakage rate (up to 490 gpm). This model has also been applied to non-Westinghouse pumps, given a lack of any alternatives.

- Given that the Commission has rejected issuance of a rule to address this matter and that changes made to plants (e.g., the SBO rule, IPE plant-specific modifications, improved RCP seal performance, etc.) have lowered the likelihood of seal failure, and given the results of the staff's analyses, it is argued that GSI-23 should be closed out.
- NRC will develop a task action plan to address the remaining plant-specific issues that need to be addressed.

Dr. Wallis said that a seal leakage model is needed that is applicable to the non-Westinghouse RCPs in use. In response to Dr. Apostolakis, the staff said that they did not perform an uncertainty analysis in support of its evaluations of the SBO and CCW/ESW events. The staff also noted in response to Dr. Apostolakis that it was necessary to combine the aleatory and epistemic uncertainties, given the lack of necessary information for non-Westinghouse pump seals.

WOG Presentation

Messrs. D. Lounsbury and T. Timmons, representing the WOG, commented on this issue. Their key points noted were as follows:

- WOG supports closure of GSI-23
- Conservative analysis determined that the maximum leak rate for Westinghouse RCPs is about 21 gpm/pump. Plant emergency procedures will ensure that this leak rate will be further reduced upon depressurization and cooldown.
- Installation of improved high-temperature O-ring material provides a long-term passive solution to this issue.
- WOG believes that the NRC analyses cited above are overly conservative, and NRC's assumptions are based on non-prototypical testing.
- Operating experience and test data do not support a high probability of excessive RCP seal leak rates.

In response to Dr. Wallis, Mr. Lounsbury said that he did not know if all plants with Westinghouse RCPs will install the improved O-ring material. Currently, 75 percent of Westinghouse plants have done so.

Conclusion

The Committee issued a letter on this matter to the Executive Director for Operations, dated October 8, 1999.

VII. Status of the Proposed Final Amendment to 10 CFR 50.55a, "Codes and Standards" (Open)

[Note: Mr. Noel F. Dudley was the Designated Federal Official for this portion of the meeting.]

Introduction

Dr. William Shack, Chairman of the Materials and Metallurgy Subcommittee, stated that the staff would provide updates on the proposal for eliminating the requirement for licensees to update their inservice inspection (ISI) and inservice testing (IST) programs every 120 months and on the proposal to add a requirement to perform volumetric inspections of small-bore high-pressure safety injection piping. He noted that the NEI would comment on the ACRS letter dated April 19, 1999, concerning the elimination of the 120-month update.

Staff Presentation

Mr. Thomas G. Scarbrough, NRR, presented historical background for the proposed amendment to 10 CFR 50.55a, that would eliminate the 120-month update requirement. He explained that the staff had reviewed the widely varying public comments on the proposed amendment and was considering possible options for resolving this issue. He stated that the staff planned to brief the ACRS on this matter at the December 1999 ACRS meeting.

Mr. Scarbrough presented historical background concerning volumetric inspection of small-bore Class 1 piping. He explained that at a meeting on August 20, 1999, with NEI, the industry presented the principal technical constraints associated with conducting volumetric inspections of small-bore Class 1 piping. Mr. Scarbrough stated that the staff deferred any action on this issue until the industry completes

its assessment of possible options. He noted that the staff plans to work with the American Society of Mechanical Engineers (ASME) to develop an ASME Code case concerning volumetric inspection of small-bore Class 1 piping. The ACRS members and the staff discussed alternative inspection options, the benefits of volumetric inspections, operating experience, and inconsistencies in the ASME Code.

Nuclear Energy Institute Presentation

Mr. Alex Marion, NEI, summarized NEI's letter to the NRC concerning the elimination of the 120-month update requirement. He stated that an industry evaluation of the ASME Code changes made between 1989 and 1992 concluded that these changes had no safety significance. Mr. Marion presented the following points:

- The ASME Code should capture current practices and support new technologies as a voluntary consensus code.
- Section 50.55a mandates the use of a voluntary code.
- Licensees' costs for updating the latest edition of the ASME Code every 10 years ranges from \$550,000 to \$1,500,000 per facility.,
- Government agencies are required to endorse consensus standards; however, the standards development process should not become an extension of the regulatory process.
- Rulemaking is too rigid and not related to safety.
- The NRC staff should expedite its endorsement of ASME Code cases.

The ACRS members, the staff, and NEI discussed whether future ASME Code changes will be safety significant, the ability of the ASME Code development process to create meaningful consensus codes, and how NEI derived the costs for updating ISI and IST programs.

Conclusion

This briefing was for information only. The Committee plans to review the issue of eliminating the 120-month update requirement when the staff's position paper is available.

VIII. Proposed Regulatory Guide on Design Bases Information (Open)

[Note: Mr. Michael T. Markley was the Designated Federal Official for this portion of the meeting.]

Introduction

Mr. John J. Barton, Chairman of the Plant Operations Subcommittee, introduced this topic to the Committee. He stated that the purpose of this meeting was to discuss the staff's proposed draft regulatory guide on design basis information and associated industry guidance in NEI 97-04, "Design Bases Program Guidelines." Mr. Barton noted that the staff has been working with the NEI since 1990 to resolve issues associated with design basis information in accordance with 10 CFR 50.2, "Definitions." Mr. Barton added that the staff has prepared a draft regulatory guide, DG-1093, "Guidance and Examples for Identifying 10 CFR 50.2 Design Bases" (September 1999), in which the staff proposes to endorse industry guidance in Appendix B to NEI 97-04 as an acceptable method for meeting NRC requirements.

NRC Staff Presentation

Mr. Stuart Magruder, NRR, led the discussions for the staff. Messrs. Dave Mathews and Richard Wessman, NRR, provided supporting discussions.

Mr. Magruder emphasized that the fundamental issue at hand and the current focus of the efforts of the staff and the industry is to establish a clear definition of the term "design basis" as defined in 10 CFR Part 50.2. Since the term "design basis" has broad use in other regulations, its application has a direct impact on all design and licensing bases activities at NRC-licensed facilities. Mr. Magruder stated that although this issue has been a topic of contention between the NRC staff and the industry for many years, it became the subject of increased concern when the NRC led engineering team inspections in the late 1980s and more recently as a result of mid-1990s inspection findings at Millstone, Maine Yankee, and Nine Mile Point nuclear stations.

NEI Presentation

Mr. Russell Bell, NEI, led the discussions on behalf of the nuclear industry. Mr. Anthony Pietrangelo, NEI, who provided supporting discussion, also participated. Mr. Bell reiterated the importance of establishing a clear definition of the term "design basis" both from a regulatory as well as from a safety perspective. Mr. Bell stated that the industry recognizes and appreciates the significance of operating within the design basis but wants to ensure that design issues that may arise in the field are properly characterized. However, NEI believes that there is a fundamental difference between "design basis" and "supporting design information." NEI representatives stated that NEI 97-04, "Design Bases Program Guidelines," reflects the industry's position on this issue and has been revised to emphasize these differences. In particular, the revised industry guidance in Appendix B to NEI 97-04 (a) provides amplifying framework guidance that NEI 97-04 lacks, (b) highlights the basis in the regulations for 10 CFR 50.2 design basis functional requirements, and (c) includes examples of "supporting design information" to highlight its distinction from "design basis."

Although the NRC staff and NEI representatives are in agreement on most issues, several fundamental issues related to the exceptions being taken by the staff in DG-1093 remain to be resolved. Specifically, agreement is needed on whether "design basis" as defined in the regulations encompasses the following:

- Redundancy and diversity as specific design functions.
- All modes of operation, including full power, low power, shutdown, and accident conditions.
- Design basis values, such as piping design pressure, temperature, material, and so on.
- Testing and inspections

Mr. Barton questioned why only Appendix B was proposed for endorsement rather than NEI 97-04 in its entirety. NEI representatives stated that the overall document was not sufficiently well developed to provide adequate distinction between design bases associated with accident analyses and supporting design information. NEI also stated that revision of the overall document to make it consistent with Appendix

B will require approval by the industry task force and would likely adversely affect the schedule for endorsement of Appendix B by the staff.

Dr. Bonaca and Mr. Barton noted that the exceptions being taken by the staff were not trivial and questioned how NRC could endorse Appendix B of NEI 97-04 without some level of closure on these issues. Dr. Powers stated that neither the staff nor NEI had expressed the problem in a manner clearly sufficient for him to understand the issues. He noted that both stated that they were more in agreement than disagreement on three out of the four exceptions but noted that the stated level of agreement was not reflected in the written materials provided to the Committee. The staff and NEI stated that progress has been made on the exceptions and that resolution could be expected in the near future. The staff noted that a follow up meeting was scheduled for October 14, 1999, to continue discussion of these matters. The staff and NEI did, however, acknowledge that they may agree to disagree on the issues related to design basis values.

At the conclusion of the meeting, the Committee stated that it would be preferable for NEI and the NRC staff to continue working on these differences until a consensus can be reached. NEI and the NRC staff reiterated that their dialogue would continue to try to find a mutually acceptable position on these issues and offered to again brief the Committee once a consensus has been reached.

Conclusion

The Committee decided to continue its review of this matter during the ACRS meeting on November 4-6, 1999.

IX. Proposed Resolution of Generic Safety Issue B-55, "Improved Reliability of Target Rock Safety Relief Valves" (Open)

[Note: Mr. Amarjit Singh was the Designated Federal Official for this portion of the meeting.]

Mr. John D. Sieber, Chairman of the Plant Systems Subcommittee, introduced this topic to the Committee. He stated that the purpose of this session was to discuss with representatives of the NRC the proposed resolution of GSI B-55, "Improved Reliability of Target Rock Safety Relief Valves."

NRC Staff Presentation

Mr. Garry Hammer led the discussions for the staff. He presented a brief background and the proposed resolution of GSI B-55. The staff stated that the GSI B-55 boiling-water reactor (BWR) pressure relief system is designed to prevent overpressurization of the reactor coolant pressure boundary under the most severe abnormal operation transient. This design function is accomplished through the use of a plant-unique combination of safety valves, power-actuated relief valves, and dual-function safety relief valves (SRVs), which have both a mechanical self-actuating setpoint function and a power-actuated function. The majority of the valves in the older BWRs were manufactured by Target Rock.

He stated that the BWR Owners Group and the individual BWR licensees have improved the performance of the SRVs by installing ion-beam platinum disks, or Stellite 21 disks, and additional pressure switches to actuate these valves using pneumatic power. On the basis of recent performance data, the staff concluded that the Stellite 21 and the ion-beam implanted platinum disks are performing better than the former Stellite 6B disks, with a lower rate of occurrence of high setpoint drift beyond that allowed by plant Technical Specifications. Additionally, the staff stated that the affected BWR plants have sufficient margin to accommodate upward valve-setpoint drift as high as 10 percent. On the basis of the activities being pursued by the licensees under existing regulatory requirements, the staff does not plan to impose any additional regulatory requirements.

Conclusion

The Committee issued a letter on this matter to the Executive Director for Operations (EDO), dated October 8, 1999.

X. Subcommittee Chairman Reports (Open)

- Proposed Final Revision to Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants"

Dr. Robert Seale, Acting Chairman of the Human Factors Subcommittee, summarized the proposed Revision 3 of Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants." He explained that the revision would endorse a more recent version of American National Standards Institute/American Nuclear Society (ANSI/ANS) 3.1, "Selection, Qualification, and

Training of Personnel for Nuclear Power Plants," with additions, exceptions, and clarifications. The most significant exception is the endorsement of ANSI/ASME NQA-1-1983, "Quality Assurance Program Requirements for Nuclear Power Plants," as the acceptable standard for individuals working in Quality Control and the Quality Control organizations. Dr. Seale recommended that the Committee not review the proposed Revision 3 of RG 1.8.

- Control Room Habitability

The Committee received a report from the Chairman of the SAM Subcommittee regarding the Subcommittee's meeting on September 16-17, 1999, on the status of NRC staff and industry actions pertaining to control room habitability. Presentations were made by representatives of the NRC staff; the Nuclear HVAC [Heating, Ventilation, and Air Conditioning] Utilities Group (NHUG); NEI; an independent contractor (P. Lagus) who performs tracer gas testing of control room leak rates; and two utilities, Commonwealth Edison Company and Northern States Power Company. The NRC staff and NEI representatives are working to address closure of this issue. NEI has prepared a draft guidance document on how licensees can reconstitute their design basis for this issue. The staff reviewed this document and identified issues for further evaluation. The Committee plans to review the NRC staff and industry resolution during the ACRS meeting in February or March 2000.

XI. EXECUTIVE SESSION (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

A. Reconciliation of ACRS Comments and Recommendations

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

The Committee discussed the response from the NRC Executive Director for Operations (EDO) to ACRS comments and recommendations included in recent ACRS reports:

- The Committee discussed the response from the NRC Executive Director for Operations (EDO) dated August 9, 1999, to ACRS comments and recommendations in the ACRS report dated July 21, 1999, concerning the proposed final Regulatory Guide 1.181, "Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e)"

The Committee decided that it was satisfied with the EDO's response.

- The Committee discussed the EDO's response dated August 9, 1999, to ACRS comments and recommendations in the ACRS report dated July 23, 1999, concerning the Proposed Maintenance Rule Guidance, DG-1082, "Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants."

The Committee decided that it was satisfied with the EDO's response.

- The Committee discussed the EDO's response dated August 9, 1999, to ACRS comments and recommendations in the ACRS letter dated June 10, 1999, concerning pilot application of the revised inspection and assessment programs, risk-based performance indicators, and performance-based initiatives, and related matters.

The Committee decided that it was not satisfied with the EDO's response. In accordance with the Staff Requirements Memorandum dated August 13, 1999, the Committee decided to discontinue its review of these matters.

- The Committee discussed the EDO's response dated August 2, 1999, to the ACRS comments and recommendations included in the ACRS report dated July 23, 1999, concerning a proposed final amendment to 10 CFR 50.55a, "Codes and Standards."

The Committee decided to continue its discussion of this matter during future meetings.

- The Committee discussed the EDO's response dated August 18, 1999, to ACRS comments and recommendations included in the ACRS report dated July 22, 1999, concerning the revision of Appendix K, "ECCS Evaluation Models," to 10 CFR Part 50.

The Committee decided it was satisfied with the EDO's response.

B. Report on the Meeting of the Planning and Procedures Subcommittee (Open)

The Committee heard a report from Dr. Powers and the Executive Director, ACRS, on the Planning and Procedures Subcommittee meeting held on September 29, 1999. The following items were discussed:

- Member Assignments and Priorities for ACRS Reports and Letters for the October ACRS Meeting

Member assignments and priorities for ACRS reports and letters for the October ACRS meeting were discussed. Letters that can be deferred or would benefit from additional consideration at a future ACRS meeting were selected. Lessons learned from the September ACRS meeting were also discussed and incorporated into the discussions of the ACRS self-assessment.

- Unanticipated Workload for ACRS Members

The anticipated workload of the ACRS members through December 1999 was discussed. The objectives were: (1) to review the reasons for the scheduling of each activity and the expected work product and to make changes, as appropriate, (2) to manage the members' workload for these meetings, and (3) to plan and schedule items for ACRS discussion of topical and emerging issues. The Subcommittee discussed and selected several items that could be deferred to a future meeting, discussed at a Subcommittee meeting, or scheduled for discussion at the ACRS retreat.

- Topics for ACRS Meeting with the Commissioners

The ACRS met with the NRC Commissioners on Thursday, November 4, 1999, to discuss items of mutual interest. Topics approved by the Commission were:

- Risk-Informing to 10 CFR Part 50
- Risk-Informing 10 CFR Part 50.59 (Changes, Tests and Experiments)

- Relationship and Balance Between PRA Results and Defense-in-Depth
 - Strategy for Reviewing License Renewal Applications
 - NRC Safety Research Program
 - Multiple SSCs Out of Service During Maintenance
 - High Burnup Fuel Phenomena Identification and Ranking
 - Low-Power and Shutdown Operations Risk
- Planning for the ACRS Retreat

During the September 1999 full Committee meeting, it was decided to have a retreat outside of the Washington, D.C. area. A list of topics for the retreat will be discussed and modified as appropriate.

- ACRS Self-Assessment

The ACRS and ACNW received a Staff Requirements Memorandum on August 6, 1999 related to the Self-Assessment report for CY 1998. In its August 6, 1999 SRM the Commission stated that:

- The periodic self-assessment report and the ACRS/ACNW Operating Plan can be combined into one annual report.
- The reports should include self-assessment summary "matrixes."
- The ACRS and ACNW should identify areas for improvement and planned actions and should highlight the degree of improvement observed in areas identified in previous assessments.

- ACRS Fellows Program

The ACRS Vice Chairman discussed lessons learned from the operation of the program and provided his suggestions for improvements in this program.

- ACRS Review of the Pilot Program and the NRC Inspection and Assessment Programs

In response to the Commission's guidance in the FY 2001 Budget/Performance Plan (SRM dated August 13, 1999), the NRC staff

deleted the ACRS review and comment on the new NRC Inspection and Assessment Programs. This item had previously been scheduled for discussion at the February 2000 full Committee meeting.

- Member Issues

Dr. Robert Uhrig received a letter from Dr. Arnold Graf, Siemens at KWU NLL 5, Erlanger, Germany with suggested meeting dates for a technical exchange with selected ACRS members.

C. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 467th ACRS Meeting, November 4-6, 1999.

The 466th ACRS meeting was adjourned at 1:25 p.m. on October 2, 1999.

Dated at Rockville, Maryland, this 14th day of September 1999.

For the Nuclear Regulatory Commission.
Frank Rinaldi,
Project Manager, Section 1, Project Directorate II, Division of Licensing and Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 99-24381 Filed 9-17-99; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-245, 50-336, and 50-423]

Northeast Nuclear Energy Company (NNECO), et al., Millstone Nuclear Power Station, Unit Nos. 1, 2, and 3, Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of exemptions from Facility Operating License Nos. DPR-21, DPR-65, and NPF-49, issued to Northeast Nuclear Energy Company (NNECO/the licensee), for operation of Millstone Nuclear Power Station, Units 1, 2, and 3 (Millstone), located in New London County, Connecticut.

Environmental Assessment

Identification of the Proposed Action

The proposed action would exempt the licensee from the requirements of 10 CFR Part 50, Appendix E, Item IV.F.2.c regarding conduct of a full participation exercise of the offsite emergency plan every 2 years. Under the proposed exemption, the licensee would reschedule the Federally-observed full-participation emergency exercise from September 1999 to March 2000 and all future Nuclear Regulatory Commission (NRC)—and Federal Emergency Management Agency (FEMA)—evaluated exercises would occur biennially from the year 2000.

The proposed action is in accordance with the licensee's application for exemption dated August 3, 1999.

The Need for the Proposed Action

Title 10 of the Code of Federal Regulation, (10 CFR) Part 50, Appendix E, Item IV.F.2.c requires each licensee at each site to conduct an exercise of its offsite emergency plan biennially. The NRC and FEMA observe these exercises and evaluate the performance of the licensee, State and local authorities having a role under the emergency plan.

The licensee had initially planned to conduct an exercise of its onsite and offsite emergency plans in September 1999, which is at the end of the required interval. To support the efficient and

effective use of Federal resources, as discussed during the annual NRC Region I and FEMA (Regions I, II, and III) exercise scheduling meeting held in White Plains, New York, in December 1998, the planned September 1999 exercise for Millstone was shifted to March 2000, which is beyond the required interval.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and concludes that the proposed action involves an administrative activity (a scheduler change in conducting an exercise) unrelated to plant operations.

The proposed action will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not involve any historic sites. It does not affect non-radiological plant effluents and has no other environmental impact. Therefore, there are no significant non-radiological environmental impacts associated with the proposed action.

Accordingly, the Commission concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the "Final Environmental Statement Related to the Continuation of Construction of Unit 2 and the Operation of Units 1 and 2, Millstone Nuclear Power Station, Millstone Point Company," dated June 1973 or "Final Environmental Statement Related to the Operation of Millstone Nuclear Power Station, Unit No. 3," dated December 1984 (NUREG-1064).

Agencies and Persons Consulted

In accordance with its stated policy, on August 5, 1999, the staff consulted with the Connecticut State official, Mr. Fred Scheuritzel of the Department of Environmental Protection, regarding the environmental impact of the proposed action. The State official had no comments. In addition, by letter dated July 14, 1999, from Ms. Vanessa Quinn, the Federal Emergency Management Agency indicated support for rescheduling the exercise.

Finding of No Significant Impact

On the basis of the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated August 3, 1999, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document rooms located at the Learning Resources Center, Three Rivers Community-Technical College, 574 New London Turnpike, Norwich, Connecticut, and the Waterford Public Library, 49 Rope Ferry Road, Waterford, Connecticut.

Dated at Rockville, Maryland, this 10th day of September 1999.

For the Nuclear Regulatory Commission.
John A. Nakoski,
Senior Project Manager, Section 2, Project Directorate I, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 99-24380 Filed 9-17-99; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Meeting Notice

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on September 30–October 2, 1999, in Conference Room T-2B3, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the Federal Register on Wednesday, November 18, 1998 (63 FR 64105).

Thursday, September 30, 1999

8:30 a.m.-8:45 a.m.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:45 a.m.-10:15 a.m.: NRC Safety Research Programs (Open)—The Committee will discuss the proposed ACRS report to the Commission on the NRC Safety Research Program and related matters. The Committee will hear presentations by and hold discussions with representatives of the NRC staff on the proposed Mission Statement for the NRC's Office of Research.

10:30 a.m.-12:00 Noon: Combustion Engineering Owners Group (CEOG) Proposal to Eliminate the Post-Accident Sampling System Requirements (Open)—The Committee will hear presentations by and hold discussions with representatives of CEOG and the NRC staff regarding CEOG proposal to eliminate the post-accident sampling system requirements.

1:30 p.m.-2:30 p.m.: Rulemaking and Review Plans Associated with the Development of Risk-Informed Revisions to 10 CFR Part 50 (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and the Nuclear Energy Institute (NEI) regarding the proposed rulemaking and review plans associated with the development of risk-informed revisions to 10 CFR part 50, "Domestic Licensing of Production and Utilization Facilities."

2:45 p.m.-3:45 p.m.: Proposed Resolution of Generic Safety Issue (GSI)-148, "Smoke Control and Manual Fire Fighting Effectiveness" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed resolution of GSI-148.

3:45 p.m.-4:30 p.m.: Discussion of Proposed ACRS Reports (Open)—The Committee will discuss the proposed guidelines for applying risk-informed decisionmaking in the licensing process.

4:30 p.m.-5:30 p.m.: Break and Preparation of Draft ACRS Reports (Open)—Cognizant ACRS members will prepare draft reports for consideration by the full Committee.

5:30 p.m.-6:30 p.m.: Joint ACRS/ACNW Report on the Proposed Framework for Risk-Informed Regulation in NMSS (Open)—The Committee will discuss the Joint ACRS/ACNW report.

6:30 p.m.-7:30 p.m.: Discussion of Proposed ACRS Reports (Open)—The Committee will discuss proposed ACRS

reports on matters considered during this meeting.

Friday, October 1, 1999

8:30 a.m.-8:35 a.m.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 a.m.-10:00 a.m.: Proposed Resolution of Generic Safety Issue (GSI)-23, "Reactor Coolant Pump Seal Failures" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed resolution of GSI-23.

10:15 a.m.-12:15 p.m.: Proposed Final Amendment to 10 CFR 50.55a, "Codes and Standards" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff, NEI and the American Society for Mechanical Engineers regarding the proposed final amendment to 10 CFR 50.55a related to elimination of the requirement to update the inservice inspection and inservice testing programs every 120 months, and the status of resolution of the issues associated with the volumetric examination of the small-bore piping of the high-pressure safety injection system.

1:15 p.m.-2:45 p.m.: Proposed Regulatory Guide on Design Bases Information (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and NEI regarding the proposed regulatory guide, which endorses NEI 97-04 document, "Design Bases Program Guidelines," and related matters.

3:00 p.m.-4:00 p.m.: Proposed Resolution of Generic Safety Issue B-55, "Improved Reliability of Target Rock Safety Relief Valves" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding proposed resolution of GSI B-55.

4:00 p.m.-5:15 p.m.: Break and Preparation of Draft ACRS Reports (Open)—Cognizant ACRS members will prepare draft reports for consideration by the full Committee.

5:15 p.m.-7:15 p.m.: Discussion of Proposed ACRS Reports (Open)—The Committee will discuss proposed ACRS reports on matters considered during this meeting.

Saturday October 2, 1999

8:30 a.m.-10:15 a.m.: Discussion of ACRS Meeting with the Commission (Open)—The Committee will discuss

the topics for the meeting with the NRC Commissioners on November 4, 1999.

10:15 a.m.-11:00 a.m.: Report of the Planning and Procedures Subcommittee (Open)—The Committee will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.

11:00 a.m.-11:15 a.m.: Future ACRS Activities (Open)—The Committee will discuss the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings.

11:15 a.m.-12:00 Noon: Subcommittee Chairman Reports (Open)—The Committee will hear reports by ACRS members on the proposed final revision to Regulatory Guide 1.8 on the training and qualification of personnel at nuclear power plants, possible modifications to the NRC PRA Implementation plan, and control room habitability issues.

12:00 Noon-12:45 p.m.: Reconciliation of ACRS Comments and Recommendations (Open)—The Committee will discuss the responses from the EDO to comments and recommendations included in recent ACRS reports and letters. The EDO responses are expected to be made available to the Committee prior to the meeting.

12:45 p.m.-5:00 p.m.: Discussion of Proposed ACRS Reports (Open)—The Committee will continue its discussion of proposed ACRS reports on matters considered during this meeting.

5:00 p.m.-5:30 p.m.: Miscellaneous (Open)—The Committee will discuss matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.

Procedures for the conduct of and participation in ACRS meetings were published in the Federal Register on September 29, 1998 (63 FR 51968). In accordance with these procedures, oral or written views may be presented by members of the public, including representatives of the nuclear industry. Electronic recordings will be permitted only during the open portions of the meeting and questions may be asked only by members of the Committee, its consultants, and staff. Persons desiring to make oral statements should notify Mr. Sam Duraiswamy, ACRS, five days before the meeting, if possible, so that appropriate arrangements can be made to allow necessary time during the meeting for such statements. Use of still, motion picture, and television cameras

during this meeting may be limited to selected portions of the meeting as determined by the Chairman. Information regarding the time to be set aside for this purpose may be obtained by contacting Mr. Sam Duraiswamy prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with Mr. Sam Duraiswamy if such rescheduling would result in major inconvenience.

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting Mr. Sam Duraiswamy (telephone 301/415-7364), between 7:30 a.m. and 4:15 p.m. EDT.

ACRS meeting agenda, meeting transcripts, and letter reports are available for downloading or viewing on the internet at <http://www.nrc.gov/ACRSACNW>.

Videoteleconferencing service is available for observing open sessions of ACRS meetings. Those wishing to use this service for observing ACRS meetings should contact Mr. Theron Brown, ACRS Audio Visual Technician (301-415-8066), between 7:30 a.m. and 5 p.m. EDT at least 10 days before the meeting to ensure the availability of this service. Individuals or organizations requesting this service will be responsible for telephone line charges and for providing the equipment facilities that they use to establish the videoteleconferencing link. The availability of videoteleconferencing services is not guaranteed.

Dated: September 14, 1999.

Andrew L. Bates,

Advisory Committee Management Officer.

[FR Doc. 99-24384 Filed 9-17-99; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Twenty-Seventh Water Reactor Safety Information Meeting

AGENCY: Nuclear Regulatory Commission

ACTION: Notice of meeting.

SUMMARY: The Twenty-Seventh Water Reactor Safety Information Meeting will be held on October 25-27, 1999, 9:00 a.m. to 5:00 p.m. at the Bethesda Marriott Hotel, 5151 Pooks Hill Road, Bethesda, Maryland.

The Water Reactor Safety Information Meeting will be opened by the NRC Office of Nuclear Regulatory Research Office Director, Ashok C. Thadani. Commissioner Jeffrey S. Merrifield will be the keynote speaker for the plenary session on Monday, October 25, 1999 at 9:00 a.m. and will be followed by an expert panel addressing the issue of the impact of emerging technologies on nuclear safety research. Panel members include NRC Commissioner Edward McGaffigan, Jr.; Sigma Xi Director and Former NRC Chairman, John Ahearne; EPRI Vice President, S&T Development & Chief Nuclear Officer, Robin Jones; NEI President & CEO, Joe Colvin; Framatome's Vice President of Corporate Research, Technology & Quality, M. A. Vallee; and the Department of Energy's Director of Nuclear Energy Science, & Technology, Bill Magwood. Technical sessions will be held in the afternoon. Chairman Greta J. Dicus will be the guest speaker on Tuesday morning at 8:30 a.m. and will be followed by technical sessions that will last until 5:00 p.m. On Wednesday, technical sessions will be held in the morning beginning at 8:30 a.m. and a plenary session featuring an expert panel addressing the issue of how best to focus both on safety and unnecessary burden reduction—the research role, will be held in the afternoon. Panel members include former NRC Office of Nuclear Reactor Regulation Office Director, Thomas Murley; NRC ACRS Chairman, Dana Powers; and representatives from the NEI and utility.

This meeting is international in scope and includes presentations by personnel from the NRC, U.S. Government, laboratories, private contractors, universities, the Electric Power Research Institute, reactor vendors, and a number of foreign organizations. This meeting is sponsored by the NRC and conducted by the Brookhaven National Laboratory.

The preliminary agenda for this year's meeting includes 8 sessions, along with discussions, on the following topics: Risk-Informed Regulation; Integrity of Primary Coolant Pressure Boundary; Regulatory Effectiveness Assessments; High Burnup and Mixed Fuel Oxide Fuel Under Reactivity Accident Conditions; Fuel Burnup Credit; Fission Product, Source Terms, and Applications; and Research Supporting New Technology.

Those who wish to attend are encouraged to register in advance on the newly established WRSM website (www.wrsm.bnl.gov) or by contacting Susan Monteleone, Brookhaven National Laboratory, Department of

Nuclear Energy, Building 130, Upton, NY 11973, telephone (516) 344-7235; or Sandra Nesmith (301) 415-6437, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

Dated at Rockville, Maryland, this 8th Day of September, 1999.

For the Nuclear Regulatory Commission.

Charles E. Ader,

Director, Program Management, Policy Development, & Analysis Staff, Office of Nuclear Regulatory Research.

[FR Doc. 99-24379 Filed 9-17-99; 8:45 am]

BILLING CODE 7590-01-P

SECURITIES AND EXCHANGE COMMISSION

Submission for OMB Review; Comment Request

Upon Written Request; Copies Available From: Securities and Exchange Commission, Office of Filings and Information Services, Washington, DC 20549.

Extension:

Regulations 13D and 13G; Schedules 13D and 13G; SEC File No. 270-137; OMB Control No. 3235-0145.

Notice is hereby given that, pursuant to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.), the Securities and Exchange Commission ("Commission") has submitted to the Office of Management and Budget a request for extension of the previously approved collection of information discussed below.

Schedules 13D and 13G are filed pursuant to Sections 13(d) and 13(g) of the Securities Exchange Act of 1934 ("Exchange Act") and Regulations 13D and 13G thereunder to report beneficial ownership of equity securities registered under Section 12 of the Exchange Act. Regulations 13D and 13G are intended to provide investors and the subject issuers with information about accumulations of securities that may have the potential to change or influence control of the issuer. Schedules 13D and 13G are used by persons including small entities to report their ownership of more than 5% of a class equity securities registered under Section 12. It is estimated that approximately 10,690 respondents file Schedules 13D and 13G for a total burden of 114,718 hours. Respondents file Schedule 13D or Schedule 13G only when necessary.

The information provided by respondents is mandatory. All information provided to the Commission is public. However, Rules 0-6 and 24b-2 under the Exchange Act



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

September 24, 1999 REVISED

SCHEDULE AND OUTLINE FOR DISCUSSION
466TH ACRS MEETING
SEPTEMBER 30 - OCTOBER 2, 1999

THURSDAY, SEPTEMBER 30, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND

- 1) 8:30 - 8:45 A.M. Opening Remarks by the ACRS Chairman (Open)
 1.1) - Opening statement (DAP/JTL/SD)
 1.2) Items of current interest (DAP/NFD/SD)
 1.3) Priorities for preparation of ACRS reports (DAP/JTL/SD)
- 2) 8:45 - ^{10:20}~~10:15~~ A.M. NRC Safety Research Program (Open) (GBW/MME)
 2.1) Remarks by the Subcommittee Chairman
 2.2) Briefing by and discussions with representatives of NRC staff on the Research Mission.
 2.3) Discussion of the proposed ACRS report to the Commission on the NRC Safety Research Program and related matters.
- ^{10:20 - 10:35}
~~10:45~~ - ~~10:30~~ A.M. *****BREAK*****
- 3) ^{10:35 - 12:30}
~~10:30~~ - ~~12:00~~ Noon Combustion Engineering Owners Group (CEOG) Proposal to Eliminate the Post-Accident Sampling System Requirements (Open) (TSK/PAB)
 3.1) Remarks by the Subcommittee Chairman
 3.2) Briefing by and discussions with representatives of CEOG and the NRC staff regarding CEOG proposal to eliminate the post-accident sampling system requirements.
- ^{12:30 - 1:30}
~~12:00~~ - ~~1:00~~ P.M. *****LUNCH*****
- 4) ^{1:30 - 3:04}
~~1:00~~ - ~~2:30~~ P.M. Rulemaking and Review Plans Associated with the Development of Risk-Informed Revisions to 10 CFR Part 50 (Open) (GA/MTM)
 4.1) Remarks by the Subcommittee Chairman
 4.2) Briefing by and discussions with representatives of the NRC staff and the Nuclear Energy Institute (NEI) regarding the proposed rulemaking and review plans associated with the development of risk-informed revisions to 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities."
- ^{3:04 - 3:15}
~~2:30~~ - ~~2:45~~ P.M. *****BREAK*****

- 5) ~~3:15 - 4:05~~
2:45 - 3:45 P.M. Proposed Resolution of Generic Safety Issue (GSI)-148, "Smoke Control and Manual Fire Fighting Effectiveness" (Open) (JJB/AS)
5.1) Remarks by the Subcommittee Chairman
5.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed resolution of GSI-148.
- Representatives of the nuclear industry will provide their views, as appropriate.
- 6) ~~4:05 - 4:25~~
~~3:45 - 4:30 P.M.~~ Subcommittee Report on Control Room Habitability (TSK/PAB)
Discussion of Proposed ACRS Report (Open) (TSK/NFD/JDP)
Discussion of the ACRS report on the draft Commission Paper regarding Proposed Guidelines for Applying Risk-Informed Decisionmaking in the Licensing Process.
- 7) ~~4:40 -~~
~~4:30 -~~ 5:30 P.M. Break and Preparation of Draft ACRS Reports
Cognizant ACRS members will prepare draft reports for consideration by the full Committee.
- 8) ~~5:30 - 5:35~~
~~5:30 - 6:30 P.M.~~
5:35 - 6:35 Subcommittee Report, RG 1.8, Dual & Training of Personnel @ NPPs (RLS/NFD)
Joint ACRS/ACNW Report on the Proposed Framework for Risk-Informed Regulation in NMSS (Open) (TSK/GA/MTM/JS)
8.1) Remarks by the ACRS members of the Joint ACRS/ACNW Subcommittee
8.2) Discussion of Joint ACRS/ACNW Report
- 9) ~~6:35 - 7:15~~
~~6:30 - 7:30 P.M.~~ Discussion of Proposed ACRS Reports (Open)
Discussion of proposed ACRS reports on:
6:35-6:50 9.1) Combustion Engineering Owners Group Proposal to Eliminate the Post-Accident Sampling System Requirements (TSK/PAB)
9.2) Proposed Resolution of GSI-148, "Smoke Control and Manual Fire Fighting Effectiveness" (JJB/AS)
4:30-4:40 Risk-Informed Decisionmaking in the Licensing Process
5:30-6:35

FRIDAY, OCTOBER 1, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 10) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (DAP/SD)
- 11) 8:35 - ~~10:00~~^{10:10} A.M. Proposed Resolution of Generic Safety Issue (GSI)-23, "Reactor Coolant Pump Seal Failures" (Open) (GBW/PAB)
11.1) Remarks by the Subcommittee Chairman
11.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed resolution of GSI-23

Representatives of the nuclear industry will provide their views, as appropriate.

~~10:10 - 10:20~~
10:00 - 10:15 A.M. *****BREAK*****

Joint ACRS/ACNW
Report 6:50-7:10

- 12) ^{10:30 - 11:20}
~~10:15 - 11:00 A.M.~~ Status of the Proposed Final Amendment to 10 CFR 50.55a, "Codes and Standards" (Open) (WJS/NFD)
12.1) Remarks by the Subcommittee Chairman
12.2) Briefing by and discussions with representatives of the NRC staff and Nuclear Energy Institute (NEI) regarding the status of the proposed final amendment to 10 CFR 50.55a related to elimination of the requirement to update the inservice inspection and inservice testing programs every 120 months, as well as the status of resolution of the issues associated with the volumetric examination of the small-bore piping of the high-pressure safety injection system.

Representatives of the American Society of Mechanical Engineers will provide their views, as appropriate.

- 13) ^{11:20}
~~11:00 - 11:30 A.M.~~ Reconciliation of the ACRS Comments and Recommendations (Open) (DAP, et al./SD, et al.)
Discussion of the response from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.

- 14) ^{12:25}
11:30 - ~~12:00~~ Noon Strategy for Reviewing License Renewal Applications (Open) (MVB/NFD)
14.1) Remarks by the Subcommittee Chairman
14.2) Discussion of the proposed strategy for ACRS review of the license renewal applications and related matters.

- ^{12:35 - 1:30}
~~12:00 - 1:00 P.M.~~ *****LUNCH****
- 15) ^{1:30 - 3:05}
~~1:00 - 2:30 P.M.~~ Proposed Regulatory Guide on Design Bases Information (Open) (JJB/MTM/JDP)
15.1) Remarks by the Subcommittee Chairman
15.2) Briefing by and discussions with representatives of the NRC staff and NEI regarding the proposed regulatory guide, which endorses NEI 97-04 document, "Design Bases Program Guidelines," and related matters.

~~2:30 - 2:45 P.M. ***BREAK***~~

- 16) ^{3:05 - 4:20}
~~2:45 - 3:45 P.M.~~ Proposed Resolution of Generic Safety Issue B-55, "Improved Reliability of Target Rock Safety Relief Valves" (Open) (JDS/AS)
16.1) Remarks by the Cognizant ACRS Member
16.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed resolution of GSI B-55.

Representatives of the nuclear industry will provide their views, as appropriate.

- 17) ~~3:45 - 4:45 P.M.~~ ^{4:30 - 5:00} Break and Preparation of Draft ACRS Reports
Cognizant ACRS members will prepare draft reports for consideration by the full Committee.
- 18) ~~4:45 - 7:15 P.M.~~ ^{5:00 - 7:10} Discussion of Proposed ACRS Reports (Open)
Discussion of proposed ACRS reports on:
- ^{6:10-6:45} 18.1) Proposed Resolution of GSI-23, "Reactor Coolant Pump Seal Failures" (GBW/PAB)
 - ^{6:45-7:10} 18.2) Combustion Engineering Owners Group Proposal to Eliminate the Post Accident Sampling System Requirements (TSK/PAB)
 - 18.3) Proposed Resolution of GSI-148, "Smoke Control and Manual Fire Fighting Effectiveness" (JJB/AS)
 - ^{5:00-6:10} 18.4) Draft Commission Paper Regarding Proposed Guidelines for Applying Risk-Informed Decisionmaking in the Licensing Process (TSK/NFD/JDP)
 - 18.5) Joint ACRS/ACNW Report on the Proposed Framework for Risk-Informed Regulation in NMSS (TSK/GA/MTM/JS)
 - 18.6) Proposed Regulatory Guide on Design Bases Information (JJB/MTM/JDP)
 - 18.7) Proposed Resolution of GSI B-55, "Improved Reliability of Target Rock Safety Relief Valves" (JDS/AS)
 - 18.8) Rulemaking and Review Plans Associated with the Development of Risk-Informed Revisions to 10 CFR Part 50 (GA/MTM)

SATURDAY, OCTOBER 2, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 19) 8:30 - 10:15 A.M. Discussion of Topics for ACRS Meeting with the Commission (Open)
(DAP/JTL)
Discussion of the following topics for meeting with the NRC Commissioners on November 4, 1999:
- ^{8:30-9:20 +} 19.1) Risk-Informing 10 CFR Part 50 (GA/MTM)
 - ^{10:30-11:30} 19.2) Risk-Informing 10 CFR Part 50.59 (Changes, Tests and Experiments) (GA/MTM)
 - 19.3) Relationship and Balance Between PRA Results and Defense-in-Depth (TSK/MTM/JNS)
 - 19.4) Strategy for Reviewing License Renewal Applications (MVB/NFD)
 - 19.5) NRC Safety Research Program (GBW/MME)
 - 19.6) Multiple SSCs Out of Service During Maintenance (JJB/AS)
 - 19.7) High Burnup Fuel Phenomena Identification and Ranking (DAP/MME)
 - 19.8) Low-Power and Shutdown Operations Risk (MVB/MME)

GSI-23, 9:20-9:55

PASS, 10:05-10:27

Joint ACRS/ACNW, 11:45-12:05

~~10:15 - 10:30 A.M. ***BREAK***~~

- 20) ^{12:10 - 12:35}
~~10:30 - 11:00 A.M.~~ Report of the Planning and Procedures Subcommittee (Open)
(DAP/JTL)
Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business.
- 21) ^{12:15 - 12:35}
~~11:00 - 11:30 A.M.~~ Future ACRS Activities (Open) (DAP/JTL/RPS/SD)
Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee.
- 22) ^{12:35 - 1:00 Discuss Research Report}
~~11:30 - 12:15 P.M.~~ Subcommittee Chairman Reports (Open)
22.1) ~~Proposed Final Revision to Regulatory Guide 1.8, Qualification and Training of Personnel at NPPs (RLS/NFD)~~
22.2) ~~Possible Modifications to the PRA Implementation Plan (GA/MTM)~~
22.3) ~~Control Room Habitability (TSK/PAB)~~
- ~~12:15 - 1:15 P.M. ***LUNCH**~~
- 23) ~~1:15 - 3:00 P.M.~~ Discussion of Proposed ACRS Reports (Open)
Discussion of proposed ACRS reports on:
23.1) Proposed Resolution of GSI-23, "Reactor Coolant Pump Seal Failures" (GBW/PAB)
23.2) Combustion Engineering Owners Group Proposal to Eliminate the Post Accident Sampling System Requirements (TSK/PAB)
23.3) Proposed Resolution of GSI-148, "Smoke Control and Manual Fire Fighting Effectiveness" (JJB/AS)
23.4) Draft Commission Paper Regarding Proposed Guidelines for Applying Risk-Informed Decisionmaking in the Licensing Process (TSK/NFD/JDP)
23.5) Joint ACRS/ACNW Report on the Proposed Framework for Risk-Informed Regulation in NMSS (TSK/GA/MTM/JS)
23.6) Proposed Regulatory Guide on Design Bases Information (JJB/MTM/JDP)
^{1:00 - 1:25} 23.7) Proposed Resolution of GSI B-55, "Improved Reliability of Target Rock Safety Relief Valves" (JDS/AS)
23.8) Rulemaking and Review Plans Associated with the Development of Risk-Informed Revisions to 10 CFR Part 50 (GA/MTM)
- 24) ~~3:00 - 3:30 P.M.~~ Miscellaneous (Open) (DAP/JTL/RPS)
Discussion of matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.

End Time: 1:25

NOTE:

- **Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.**
- **Number of copies of the presentation materials to be provided to the ACRS - 35.**

APPENDIX III: MEETING ATTENDEES

466TH ACRS MEETING
September 30-October 2, 1999

NRC STAFF (September 30, 1999)

W. Ott, OEDO B. Palla, NRR
J. Mitchell, RES S. West, NRR
T. King, RES
J. Rosenthal, RES
N. Kadambi, RES
F. Ovar, RES
J. Craig, RES
P. Lewis, RES
A. Rubin, RES
N. Siu, RES
H. VanderMolen, RES
J. Calvert, RES
R. Hasselberg, IRO
L. Lambros, NRR
J. Cushing, NRR
B. Boger, NRR
R. Gallo, NRR
J. O'Brien, NRR
S. Bloom, NRR
M. Snodderly, NRR
S. LaView, NRR
K. Parczewski, NRR
R. Pedersen, NRR
S. Rovdier, NRR
F. Kantor, NRR
T. Reed, NRR
S. Newberry, NRR
G. Bagchi, NRR
T. Bergman, NRR
J. Williams, NRR
M. Shewski, NRR
P. Balmain, NRR
G. Parry, NRR
C. Carpenter, NRR
M. Cheok, NRR
S. Magruder, NRR
A. Markley, NRR
S. Dinsmore, NRR

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

R. Schneider, ABB/CE
D. Bice, Entergy Operations, Inc.
S. Lurie, ABB/CE
R. Partridge, Entergy Operations Inc.
D. Raleigh, SERCH/Bechtel
C. Amoniso, NUSIS
S. Floyd, NEI
A. Meymer, NEI
J. Riccio, Public Citizen
T. Brooks, NEU
F. Emerson, NEU

NRC STAFF (October 1, 1999)

T. Boyce, OCM
J. Wermiel, NRR
D. Terao, NRR
M. Mitchell, NRR
M. Kotzalas, NRR
K. Wichman, NRR
T. Scarborough, NRR
G. Imbro, NRR
D. Wessman, NRR
S. Magruder, NRR
E. McKenna, NRR
F. Akstulewicz, NRR
C. Carpenter, NRR
C. Jackson, NRR
O. Chopra, NRR
K. Parczewski, NRR
D. Terao, NRR
G. Hammer, NRR
D. Mathews, NRR
E. Marinos, NRR
S. Bahadur, RES
N. Chokshi, RES
J. Craig, NRR
J. Jackson, RES
H. VanderMolen, RES
M. Wegner, RES

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

S. Lurie, ABB/CE
A. Rudyl, ABB/CE
M. Sabadhi, BNL
A. Mario, NEI
T. Timmons, Westinghouse
C. Brinkman, ABB
B. Mrowca, BGE
C. Cave, McGraw Hill
K. Cozens, NEI
A. Wyche, SERCH/Bechtel
C. Amoruso, NUS
P. Negus, GE
T. Peitrangelo, NEI
R. Bell, NEI



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

October 7, 1999

**SCHEDULE AND OUTLINE FOR DISCUSSION
467TH ACRS MEETING
NOVEMBER 4-6, 1999**

**THURSDAY, NOVEMBER 4, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND**

- 1) 8:30 - 9:15 A.M. Preparation for Meeting with the Commission (Open) (DAP, et al./ JTL, et al.)
Discussion of the following topics for meeting with the NRC Commissioners:
- 1.1) Risk-Informing 10 CFR Part 50 (GA/MTM)
 - 1.2) Risk-Informing 10 CFR Part 50.59 (Changes, Tests and Experiments) (GA/MTM)
 - 1.3) Relationship and Balance Between PRA Results and Defense-in-Depth (TSK/MTM/JNS)
 - 1.4) Strategy for Reviewing License Renewal Applications (MVB/NFD)
 - 1.5) NRC Safety Research Program (GBW/MME)
 - 1.6) Multiple SSCs Out of Service During Maintenance (JJB/AS)
 - 1.7) High Burnup Fuel Phenomena Identification and Ranking (DAP/MME)
 - 1.8) Low-Power and Shutdown Operations Risk (RLS/MME)
- 9:15 - 9:30 A.M. *****BREAK*****
- 2) 9:30 - 11:30 A.M. Meeting with the Commission (Open)
Meeting with the NRC Commissioners at the Commissioners' Conference Room, One White Flint North, to discuss matters listed under Item 1.
- 11:30 - 1:00 P.M. *****LUNCH*****
- 3) 1:00 - 1:15 P.M. Opening Remarks by the ACRS Chairman (Open)
- 3.1) Opening statement (DAP/JTL/SD)
 - 3.2) Items of current interest (DAP/NFD/SD)
 - 3.3) Priorities for preparation of ACRS reports (DAP/JTL/SD)
- 4) 1:15 - 2:45 P.M. Proposed Revision to Section 11 of NUMARC 93-01, Assessment of Risk Resulting from Performance of Maintenance Activities (Open) (JJB/JDS/AS)
- 4.1) Remarks by the Subcommittee Chairman
 - 4.2) Briefing by and discussions with representatives of the Nuclear Energy Institute and NRC staff regarding proposed revision to Section 11 of NUMARC 93-01.

- 2:45 - 3:00 P.M. *****BREAK*****
- 5) 3:00 - 5:00 P.M. NRC Safety Research Program Report to the Commission (Open)
(GBW/MME)
5.1) Remarks by the Subcommittee Chairman
5.2) Discussion of proposed ACRS report to the Commission on the NRC Safety Research Program.
- Representatives of the NRC staff will provide their views, as appropriate.
- 6) 5:00 - 6:00 P.M. Break and Preparation of Draft ACRS Reports
Cognizant ACRS members will prepare draft reports for consideration by the full Committee.
- 7) 6:00 - 7:00 P.M. Discussion of Proposed ACRS Reports (Open)
Discussion of proposed ACRS reports on:
7.1) Proposed Revision to Section 11 of NUMARC 93-01 (JJB/JDS/AS)
7.2) Proposed Resolution of GSI-148, Smoke Control and Manual Fire Fighting Effectiveness (JJB/AS)
7.3) Joint ACRS/ACNW Report on the Proposed Framework for Risk-Informed Regulation in NMSS (TSK/GA/MTM/JS)
- FRIDAY, NOVEMBER 5, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND**
- 8) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (DAP/SD)
- 9) 8:35 - 10:00 A.M. Proposed Changes to the Design Control Document Associated with the AP600 Design (Open) (JJB/NFD)
9.1) Remarks by the Subcommittee Chairman
9.2) Briefing by and discussions with representatives of the NRC staff and Westinghouse Electric Company regarding proposed changes to the Design Control Document related to the AP600 design and the associated NRC staff evaluation.
- 10:00 - 10:15 A.M. *****BREAK*****
- 10) 10:15 - 11:45 A.M. Spent Fuel Fire Risk Associated with Decommissioning (Open)
(TSK/DAP/MME)
10.1) Remarks by the Subcommittee Chairman
10.2) Briefing by and discussions with representatives of the NRC staff regarding the status of staff activities related to assessing the spent fuel fire risk associated with decommissioning, spent fuel pool risk assessment study, and related matters.

Representatives of the nuclear industry will provide their views, as appropriate.

- 11:45 - 12:45 P.M.** *****LUNCH*****
- 11) 12:45 - 1:45 P.M. Status of Resolution of Issues Associated with the Design Bases Information (Open) (JJB/MTM)
 11.1) Remarks by the Subcommittee Chairman
 11.2) Briefing by and discussions with representatives of the NRC staff and Nuclear Energy Institute (NEI) regarding the status of resolution of the differences between the NRC staff and NEI related to NEI 97-04 document, "Design Bases Program Guidelines."
- 12) 1:45 - 2:15 P.M. Future ACRS Activities (Open) (DAP/JTL/SD)
 Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee.
- 2:15 - 2:30 P.M.** *****BREAK*****
- 13) 2:30 - 3:00 P.M. Report of the Planning and Procedures Subcommittee (Open) (DAP/JTL)
 Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business.
- 14) 3:00 - 3:15 P.M. Reconciliation of the ACRS Comments and Recommendations (Open) (DAP, et al./SD, et al.)
 Discussion of the response from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.
- 15) 3:15 - 4:00 P.M. Break and Preparation of Draft ACRS Reports
 Cognizant ACRS members will prepare draft reports for consideration by the full Committee.
- 16) 4:15 - 7:00 P.M. Discussion of Proposed ACRS Reports (Open)
 Discussion of proposed ACRS reports on:
 16.1) Proposed Changes to the Design Control Document Associated with the AP600 Design (JJB/TSK/NFD)
 16.2) Proposed Revision to Section 11 of NUMARC 93-01 (JDS/JJB/AS)
 16.3) NRC Safety Research Program (GBW/PAB)
 16.4) Proposed Regulatory Guide on Design Bases Information (JJB/MTM)
 16.5) Proposed Resolution of GSI-148, Smoke Control and Manual Fire Fighting Effectiveness (JJB/AS)
 16.6) Joint ACRS/ACNW Report on the Proposed Framework for Risk-Informed Regulation in NMSS (TSK/GA/MTM/JS)

**SATURDAY, NOVEMBER 6, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND**

- 17) 8:30 - 2:00 P.M. Discussion of Proposed ACRS Reports (Open)
(12:00-1:00 P.M. - LUNCH) Discussion of proposed ACRS reports on:
- 17.1) NRC Safety Research Program (GBW/MME)
 - 17.2) Proposed Changes to the Design Control Document
Associated with the AP600 Design (JJB/TSK/NFD)
 - 17.3) Proposed Revision to Section 11 of NUMARC 93-01.
(JDS/JJB/AS)
 - 17.4) Proposed Regulatory Guide on Design Bases Information
(JJB/MTM)
 - 17.5) Proposed Resolution of GSI-148, Smoke Control and Manual
Fire Fighting Effectiveness (JJB/AS)
 - 17.6) Joint ACRS/ACNW Report on the Proposed Framework for
Risk-Informed Regulation in NMSS (TSK/GA/MTM/JS)
- 18) 2:00 - 2:30 P.M. Miscellaneous (Open) (DAP/JTL/RPS)
Discussion of matters related to the conduct of Committee activities
and matters and specific issues that were not completed during
previous meetings, as time and availability of information permit.

NOTE:

- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
- Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX V
LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE
466th ACRS MEETING
September 30-October 2, 1999

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA
ITEM NO.

DOCUMENTS

- 1 Opening Remarks by the ACRS Chairman
 1. Items of Interest, dated September 30-October 2, 1999

- 2 NRC Safety Research Program
 2. Vision Statement for Office of Nuclear Regulatory Research, presentation by A. Thadani, RES [Viewgraphs]

- 3 Combustion Engineering Owners Group (CEOG) Proposal to Eliminate the Post-Accident Sampling System Requirements
 3. Relaxation of NUREG-0737 Requirements for Post Accident Sampling System, presentation by R. Schneider, ABB/CEOG [Viewgraphs]
 4. Post Accident Sampling System, presentation by the NRR [Viewgraphs]
 5. US NRC Incident Response Program, presentation by R. Hasselberg, IRO [Viewgraphs]

- 4 Rulemaking and Review Plans Associated with the Development of Risk-Informed Revisions 10 CFR Part 50
 6. Risk-Informed Treatment of Structures, Systems and Components, presentation by NRR [Viewgraphs]
 7. Risk-Informing Part 50, Option 3, Plan, presented by T. King, RES [Viewgraphs]
 8. Risk-Informed, Performance-Based Improvements to NRC Regulations, presentation by NEI [Viewgraphs]
 9. Statement of James P. Riccio, Staff Attorney, Public Citizen's Critical Mass Energy Project [Handout]

- 5 Proposed Resolution of Generic Safety Issue (GSI)-148, "Smoke Control and Manual Fire Fighting Effectiveness"
 10. Generic Issue 148, "Smoke Control and Manual Fire Fighting Effectiveness", presentation by A. Rubin, RES

- 6 Subcommittee Chairman Reports
 11. Subcommittee Report, SAM Subcommittee Meeting, September 16-17, 1999, Control Room Habitability [Handout]

- 11 Proposed Resolution of Generic Safety Issue (GSI)-23, "Reactor Coolant Pump Seal Failures"
 12. Report from ACRS Consultant V. Shrock on 9/15-16, 1999 T/H Phenomena Subcommittee Meeting; presentation schedule, ACRS review of proposed resolution of GSI-23
 13. Resolution of Generic Issue 23 "Reactor Coolant Pump Seal Failure," presentation by RES [Viewgraphs]
 14. Reactor Coolant Pump Seal Integrity, presentation by Westinghouse Owners Group [Viewgraphs]

- 12 Status of the Proposed Final Amendment to 10 CFR 50.55a, "Codes and Standards"
 15. Status of 120-Month ISI/IST Update Requirement and HPSI Class 1 Piping Weld Examinations, presentation by T. Scarbrough, NRR
 16. Letter from NEI Administrative Points of Contact, Subject: Comment on Proposed Rule, Industry Codes and Standards; Amended Requirements (10 CFR 50.55a), dated June 25, 1999 [Handout]

- 13 Reconciliation of the ACRS Comments and Recommendations
 17. Reconciliation of ACRS Comments and Recommendations [Handout 13.1]

- 15 Proposed Regulatory Guide on Design Bases Information
 18. Clarifying the Definition of Design Bases, presentation by NRR [Viewgraphs]
 19. Design Bases Interpretation, presentation by NEI [Viewgraphs]
 20. Revised NEI 97-04, Appendix B [Handout]

- 16 Proposed Resolution of Generic Safety Issue B-55, "Improved Reliability of Target Rock Safety Relief Valves"
 21. Resolution of Generic Safety Issue B-55 Reliability of Target Rock Safety Relief Valves, presented by NRR [Viewgraphs]

- 20 Report of the Planning and Procedures Subcommittee
 22. Final Draft Minutes of Planning and Procedures Subcommittee Meeting - September 29, 1998 [Handout #20.1]

- 21 Future ACRS Activities
 23. Future ACRS Activities - 467th ACRS Meeting, November 4-6, 1999 [Handout #21-1]

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

- 2 NRC Safety Research Program
 1. Table of Contents
 2. Presentation Schedule
 3. Status Report dated September 30, 1999
 4. Note from Dr. Wallis to ACRS Members dated September 3, 1999

- 3 Combustion Engineering Owners Group Proposal to Eliminate the Post-Accident Sampling System Requirements
 5. Table of Contents
 6. Presentation Schedule
 7. Project Status Report dated September 30, 1999
 8. ACRS letter to W. D. Travers, EDO, from D. M. Powers, ACRS Chairman, Subject: Modifications Proposed by the Westinghouse Owners Group to the Core Damage Assessment Guidelines and Post Accident Sampling System (PASS) Requirements, dated September 17, 1999
 9. NRC Safety Evaluation Report by the Office of Nuclear Reactor Regulation Related to the Technical Basis for Allowing Combustion Engineering Pressurized Water Reactors to Change Commitments Related to the Post Accident Sampling System
 10. NRC Safety Evaluation Report by the Office of Nuclear Reactor Regulation Related to the Technical Basis for Allowing Westinghouse Pressurized Water Reactors to Change Commitments Related to the Post Accident Sampling

- 4 Rulemaking and Review Plans Associated with the Development of Risk-Informed Revisions to 10 CFR Part 50
 11. Table of Contents
 12. Proposed Schedule
 13. Status Report dated September 30, 1999
 14. Staff Requirements Memorandum - SECY-98-300 - Options for Risk-Informed Revisions to 10 CFR Part 50 - "Domestic Licensing of Production and Utilization Facilities."
 15. Letter dated December 14, 1998, from R. L. Seale, Chairman, ACRS, to William D. Travers, Executive Director for Operations, NRC, Subject: Proposed Commission Paper Concerning Options for Risk-Informed Revisions to 10 CFR Part 50 - "Domestic Licensing of Production and Utilization Facilities."
 16. Letter dated January 8, 1999, from William D. Travers, Executive Director for Operations, NRC, to Dana A. Powers, Chairman, ACRS, Subject:

- Commission Paper Concerning Options for Risk-Informed Revisions to 10 CFR Part 50.”
17. Letter dated December 11, 1998, from R. L. Seale, Chairman, ACRS, to William D. Travers, Executive Director for Operations, Subject: Options for Incorporating Risk Insights into the 10 CFR 50.59 Process.”
 18. Letter dated January 29, 1999, from William D. Travers, Executive Director for Operations, NRC, to Dana A. Powers, Chairman, ACRS, Subject: Response to ACRS letter to Dr. W. D. Travers, dated December 11, 1998, “Options for Incorporating Risk Insights into the 10 CFR 50.59 Process.”
- 5 Proposed Resolution of Generic Safety Issue (GSI)-148, “Smoke Control and Manual Fire Fighting Effectiveness”
19. Table of Contents
 20. Proposed Schedule
 21. Status Report dated September 30, 1999
 22. Memorandum to Ashok C. Thadani, Director, Office of Nuclear Regulatory Research, from Thomas L. King, Director, Division of Risk Analysis and Applications, Subject: Staff Review Guidance for Generic Safety Issue 148, “Smoke Control and Manual Fire-Fighting Effectiveness,” dated July 22, 1999
- 11 Proposed Resolution of Generic Safety Issue (GSI)-23, “Reactor Coolant Pump Seal Failures”
23. Table of Contents
 24. Project Status Report dated October 1, 1999
 25. Memorandum to W. D. Travers, EDO, from A. C. Thadani, RES, Subject: Closeout of Generic Safety Issue 23, “Reactor Coolant Pump Seal Failures,” undated
 26. Excerpted Portion, Working Copy of Minutes of Thermal-Hydraulic Phenomena Subcommittee Meeting, September 15-16, 1999 (Internal Committee Use)
 27. Presentation to T.H Phenomena Subcommittee, September 15, 1999: Contribution to CDF from RCP Seal LOCA Sequences on Station Blackout and Contribution to CDF from RCP Seal LOCA Sequences Induced by Loss of Component Cooling Water or Essential Service Water, A. Buslik, RES
- 12 Status of the Proposed Final Amendment to 10 CFR 50.55a, “Codes and Standards”
28. Table of Contents
 29. Proposed Schedule
 30. Status Report dated October 1, 1999
 31. Letter dated April 19, 1999, from Dana A. Powers, Chairman, ACRS, to William C. Travers, Executive Director for Operations, NRC, Subject: SECY-

- 99-017, "Proposed Amendment to 10 CFR 50.55a"
32. Letter dated June 16, 1999, from James A. Perry, Vice President, ASME, to the Secretary of the Commission, Subject: Federal Register Notice Solicitation of Public Comments Regarding Proposed Supplement to the Proposed Rule Published on December 3, 1997 (62 FR 63892) that Would Eliminate the 120 Month Requirement for Licensees to Update Their Inservice and Inservice Testing Programs
 33. Letter dated June 25, 1999, from David J. Modeen, Nuclear Energy Institute, to Annette L. Vietti-Cook, Secretary, NRC, Subject: Comment on Proposed Rule, *Industry Codes and Standards; Amended Requirements* (64 Fed. Reg. 22580)
- 15 Proposed Regulatory Guide on Design Bases Information
34. Proposed Schedule
 35. Status Report dated October 1, 1999
 36. Draft Regulatory Guide DG-1093, "Guidance and Examples for Identifying 10 CFR 50.2 Design Bases," September 1999
 37. Letter dated August 17, 1999, from A. R. Pietrangelo (NEI), to D. B. Matthews (NRC), Subject: Submittal of Appendix B of NEI 97-04, "Design Bases Program Guidelines," for NRC Review
- 16 Proposed Resolution of Generic Safety Issue B-55, "Improved Reliability of Target Rock Safety Relief Valves"
38. Table of Contents
 39. Proposed Schedule
 40. Status Report dated October 1, 1999
 41. Note to John T. Larkins from Charles Hammer, Subject: ACRS Briefing of Generic Safety Issue B-55, "Improved Reliability of Target Rock Safety Relief Valves," dated September 2, 1999.