

CERTIFIED

Date Issued: 6/24/99
Date Signed: 7/8/99

ACRS-3159
PDR

TABLE OF CONTENTS MINUTES OF THE 462ND ACRS MEETING

MAY 5-8, 1999

	<u>Page</u>
I. <u>Chairman's Report (Open)</u>	1
II. <u>Electric Power Research Institute (EPRI) Application of Risk-Informed Methods to Inservice Inspection (Open)</u>	1
III. <u>Proposed Final Revision to 10 CFR 50.59 (Changes, Tests and Experiments) (Open)</u>	4
IV. <u>Safety Evaluation for the Calvert Cliffs Nuclear Power Plant (CCNPP) License Renewal Application (Open)</u>	6
V. <u>Proposed Resolution of Generic Safety Issue (GSI) 158, "Performance of Safety Related Power-Operated Valves Under Design Bases Conditions" (Open)</u>	8
VI. <u>Proposed Resolution of Generic Safety Issue (GSI) 165, "Spring-Actuated Safety Relief Valve Reliability" (Open)</u>	9
VII. <u>Fire Protection Functional Inspection Program (Open)</u>	10
VIII. <u>Westinghouse Owners Group (WOG) Proposal for Modification of Core Damage Assessment Guidelines (CDAG) and Post-Accident Sampling System (PASS) Requirements (Open)</u>	11
IX. <u>Executive Session (Open)</u>	12

Reports, Letters, and Memorandum

REPORTS

- Modified Proposed Final Revision to 10 CFR 50.65, "Requirements for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants (Report to Shirley Ann Jackson, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated May 11, 1999)

RSOI

- Use of Mixed Oxide Fuel in Commercial Nuclear Power Plants (Report to Shirley Ann Jackson, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated May 17, 1999)
- Proposed Final Rule - Revisions to 10 CFR Parts 50 and 72 Concerning Changes, Tests, and Experiments (Report to Shirley Ann Jackson, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated May 17, 1999)
- The Role of Defense In Depth in a Risk-Informed Regulatory System (Report to Shirley Ann Jackson, Chairman, NRC, from Dana A. Powers, Chairman, ACRS, dated May 19, 1999)

LETTERS

- Proposed Resolution of Generic Safety Issue-158, "Performance of Safety-Related Power-Operated Valves Under Design Basis Conditions" (Letter to William D. Travers, Executive Director for Operations, NRC, from Dana A. Powers, Chairman, ACRS, dated May 14, 1999)
- Interim Letter on the Safety Aspects of the Baltimore Gas and Electric Company's License Renewal Application for Calvert Cliffs Nuclear Power Plant, Units 1 and 2 (Letter to William D. Travers, Executive Director for Operations, NRC, from Dana A. Powers, Chairman, ACRS, dated May 19, 1999)
- Modifications Proposed by the Westinghouse Owners Group to the Core Damage Assessment Guidelines and Post Accident Sampling System Requirements (Letter to William D. Travers, Executive Director for Operations, NRC, from Dana A. Powers, Chairman, ACRS, dated May 19, 1999)

MEMORANDUM

- Draft Guide DG-1075, "Emergency Planning and Preparedness for Nuclear Power Reactors" (Memorandum to William D. Travers, Executive Director for Operations, NRC, from John T. Larkins, Executive Director, ACRS, dated June 3, 1999)

- A. Reconciliation of ACRS Comments and Recommendations
- B. Report on the Meeting of the Planning and Procedures Subcommittee Held on May 5, 1999 (Open)
- C. Future Meeting Agenda

APPENDICES

- I. Federal Register Notice
- II. Meeting Schedule and Outline
- III. Attendees
- IV. Future Agenda and Subcommittee Activities
- V. List of Documents Provided to the Committee

CERTIFIED

MINUTES OF THE 462ND MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
MAY 5-8, 1999
ROCKVILLE, MARYLAND

The 462nd meeting of the Advisory Committee on Reactor Safeguards was held in Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on May 5-8, 1999. The purpose of this meeting was to discuss and take appropriate action on the items listed in the following meeting minutes. The meeting was open to public attendance. There were no written statements nor requests for time to make oral statements from members of the public regarding the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the Gelman Building, 2120 L Street, N.W., Washington, D.C. [Copies of the transcript are available for purchase from Ann Riley & Associates, Ltd., 1025 Connecticut Avenue, N.W., Suite 1014 Washington, D.C. 20036.]

ATTENDEES

ACRS Members: Dr. Dana A. Powers (Chairman), Dr. George Apostolakis (Vice-Chairman), Mr. John Barton, Dr. Mario V. Bonaca, Dr. Mario H. Fontana, Dr. Thomas S. Kress, Dr. Don W. Miller, Dr. William J. Shack, Dr. Robert L. Seale, Dr. Robert E. Uhrig, and Dr. Graham B. Wallis. [For a list of other attendees, see Appendix III.]

I. CHAIRMAN'S REPORT (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Dana A. Powers, Committee Chairman, convened the meeting at 1:00 p.m. and reviewed the schedule for the meeting. He announced that a proposed new member to the Committee, Jack Siebert, was in the audience. Dr. Powers noted that the NRC approved restart of Millstone Unit 2 and that the Clinton plant is progressing toward restart following an extended shutdown.

II. Electric Power Research Institute (EPRI) Application of Risk-Informed Methods to Inservice Inspection (Open)

[Note: Mr. Michael T. Markley was the Designated Federal Official for this portion of the meeting.]

Dr. William Shack, Chairman of the Subcommittee Materials and Metallurgy, introduced the topic to the Committee. He stated that the purpose of this meeting was to review EPRI topical report, EPRI TR-11657, "Revised Risk-Informed Inservice Inspection Procedure" for piping and proposed NRC positions for the development of an associated safety evaluation report (SER). Dr. Shack noted that the ACRS Subcommittees on Materials and Metallurgy and on Reliability and Probabilistic Risk Assessment had met earlier on May 5, 1999, to discuss this matter.

EPRI Presentation

Mr. Jeffrey Mitman, EPRI project manager for risk-informed inservice inspection (ISI) led the discussions regarding EPRI TR-11657. He noted that the EPRI team was still available in the audience and could be called upon to answer questions, if needed. Mr. Karl Fleming, ERIN Engineering, provided supporting discussion. Significant points raised during the presentation include:

- EPRI expects to resolve issues in the staff's draft SER in July-August 1999 and issue the proposed final version of EPRI TR-11657 September 1999 after receiving the staff's final SER.
- Pilot plants include: Vermont Yankee, Arkansas Nuclear One Unit 1, River Bend, Waterford, South Texas Project and Braidwood (ASME Code Case N-560 for Class 1 piping) and Arkansas Nuclear One Unit 2 and Fitzpatrick (ASME Code Case N-578 for Class 1, 2, and 3 piping).
- The EPRI risk methodology utilizes a cross comparison matrix of failure potential assessment (degradation mechanism) versus consequence assessment (CCDP and CLERP potential).
- Pipe service experience including susceptible regions and pipe segments, degradation mechanisms, and consequence considerations (initiating event, mitigation capability, and containment effects) is considered in evaluating risk relative to consequence ranking criteria.
- EPRI TR-11657 is consistent with the guidance provided in Regulatory Guide 1.174 (General Guidance) and 1.178 (ISI) for risk-informed changes to the licensing basis.

NRC Staff Presentation

Messrs. Syed Ali and Steven Dinsmore, NRR, led the discussions for the NRC staff. Mr. Thomas Sullivan, NRR, provided supporting discussion. Significant points made during the presentation include:

- The staff has approved pilot applications for Vermont Yankee and Arkansas Nuclear One Unit 2 using the EPRI methodology. Arkansas Nuclear One Unit 1 is scheduled for completion in July 1999.
- The EPRI risk-informed ISI methodology can be applied on a system-by-system basis.
- Changes subsequent to the approved pilot applications include minor modifications to the safety-significant categories and changes to the Δ risk calculation and associated criteria.

Dr. Powers questioned whether water chemistry has a lot to do with pipe failure. He also questioned the likelihood of human error leading to unfavorable water chemistry and corrosion. EPRI representatives stated that the process for integrating operating experience feedback captures conditions that might lead to such degradation. However, they noted that no attempts had been made to identify acts of commission/omission related to human error. EPRI representatives also noted that PRA is not used to estimate the likelihood of pipe ruptures but to assess the consequences. EPRI representatives also stated that they also look more closely at root causes of problems with special attention to the potential for common-cause failures.

Conclusion

The Committee plans to review the proposed final topical report and associated SER when available.

III. Proposed Final Revision to 10 CFR 50.59 (Changes, Tests and Experiments)
(Open)

[Note: Mr. Michael T. Markley was the Designated Federal Official for this portion of the meeting.]

Mr. John Barton, Chairman of the ACRS Subcommittee on Plant Operations, introduced the topic to the Committee. He stated that the purpose of this meeting was to review the proposed final revision to 10 CFR 50.59. He noted that the Committee issued a report on SECY-99-054 on March 22, 1999, on this matter. He also noted that the staff had met with the Nuclear Energy Institute (NEI) several times and that these discussions appeared to have been beneficial in resolving differences. Mr. Barton stated that major changes in the proposed rule since the ACRS last met are the addition of two new criteria to the rule language.

NRC Staff Presentation

Mr. David Matthews, NRR, introduced the NRC staff in attendance and provided introductory remarks. Ms. Eileen McKenna, NRR, led the discussions for the staff. Mr. Frank Akstulewicz and Ms. Cynthia Carpenter, NRR, also participated. Significant points raised during the staff presentation include:

- The staff's Commission paper on proposed final revisions to 10 CFR 50.59 supplements the recommendations made in SECY-99-054. It incorporates the results of meetings with NEI as well as informal feedback from the individual Commissioners and Chairman Jackson.
- Proposed Criterion (vii) result in a design basis limit for a fission product barrier being exceeded or altered.
- Criterion (viii) result in a departure from a method of evaluation used in establishing the design bases or in the safety analysis.
- Supplemental guidance for the applicable regulations (Parts 71, 72, 74, etc.) to conform with the proposed final revision to 10 CFR 50.59 may be delayed to enable the staff and NEI to develop implementing guidance.
- The staff stated that they and NEI agree that an 18-month implementation period is reasonable.

NEI Presentation

Mr. Russell Bell provided a brief summary of industry perspectives regarding the proposed changes to 10 CFR 50.59 and related matters. Significant points made during the presentation include:

- NEI and the staff agree on most issues related to the proposed rulemaking. However, the full extent of the common understanding needs to be worked out in revising NEI 96-07, "Guidelines for 10 CFR 50.59 safety Evaluations," and an associated NRC regulatory guide.
- NEI supports issuance of the proposed rule and associated conforming changes to related regulations.

Dr. Apostolakis suggested that the staff modify the sentence regarding "likelihood" and "probability." He stated that these words mean essentially the same thing and that "clarity" is not provided as purported in the subject sentence. The staff agreed to make the change as recommended by Dr. Apostolakis.

Dr. Kress questioned the 10 percent criterion for "minimal." He noted that the uncertainties may be so large that there may not be a proper gauge on minimal. Drs. Kress and Wallis questioned why there was no justification for this value in the rulemaking package. The staff stated that 10 percent was applied as a criterion for evaluating consequences. Dr. Bonaca stated that he has some comfort with the 10 percent criterion. NEI representatives stated that a 10 percent increase in dose is still limited by the NRC acceptance guidelines. Dr. Wallis questioned how long it will take to use up the available margin if it is incrementally decreased by 10 percent. The staff stated that licensees will have to evaluate each change against the criteria and assess the consequences.

Dr. Powers questioned why the industry was not asking for more flexibility in Criterion (viii). NEI representatives stated that they hoped to build flexibility in the guidance using lots of examples in the revised guidance in NEI 96-07.

Conclusion

The Committee issued a report to Chairman Jackson dated May 17, 1999, on this matter.

IV. Safety Evaluation for the Calvert Cliffs Nuclear Power Plant (CCNPP) License Renewal Application (Open)

[Note: Mr. Noel F. Dudley was the Designated Federal Official for this portion of the meeting.]

Introduction

Dr. Mario Fontana, Chairman of the Subcommittee on Plant License Renewal, stated that the Subcommittee met with representatives of the staff and Baltimore Gas and Electric Company (BGE) on April 28-29, 1999. He stated that the Subcommittee had requested the staff and BGE to provide an overview of the ground rules for license renewal, the Calvert Cliffs license renewal application, and the NRC safety evaluation report (SER) at the May 5-8, 1999 ACRS meeting. Dr. Fontana noted that the ACRS planned to issue an interim letter concerning the Calvert Cliffs SER.

Overview of the License Renewal Rule

Mr. Stephen Hoffman, Office of Nuclear Reactor Regulation (NRR), presented the intent of 10 CFR Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants," the Commission determinations related to Part 54, and the principles of license renewal. He described the required contents of a license renewal application and identified the structures and components that are subject to review as part of the integrated plant assessment process. Mr. Hoffman described time-limiting aging analyses and the regulatory standards for issuance of a renewed license.

The ACRS Members and the staff discussed the following:

- whether increases in population density or probabilistic risk assessment (PRA) insights would be considered as part of the license renewal review process,
- how the license renewal process ensures identification of all applicable components,
- how time-limited aging analyses are performed,

- why existing aging management programs for time-independent processes needed to be reviewed, and
- how to identify and manage emerging aging-induced degradation mechanisms.

Calvert Cliffs License Renewal Application

Mr. Bath W. Doroshuck, BGE, provided an overview of the development, review, and implementation of the Calvert Cliffs license renewal application. He presented the goals and life cycle management of Calvert Cliffs. He explained the contents and construction of the license renewal application. In particular, Mr. Doroshuck described the integrated plant assessment (IPA) process, aging management reviews, and aging management programs. Mr. Doroshuck stated that of the 446 aging management activities identified in the license renewal application, 329 were existing programs, 107 were modified programs or analyses, and 10 were new programs. He concluded that no aging-related degradation mechanism unique to license renewal has been discovered, that inspections will be conducted to ensure the effects of aging are managed, and that planning for long term safety and reliability has been completed.

The ACRS Members and BGE discussed the balance between safety and cost competitiveness, use of formal decision analyses, areas for improvements or efficiencies in the license renewal process, planned capital projects during the license extension period, any needs for additional studies, and use of PRA insights in the license renewal process. Some ACRS Members noted that the change in core damage frequency (CDF) resulting from license extension is inconsistent with the proposed revision to 10 CFR 50.59, "Changes, tests and experiments," and guidance provided in Regulatory Guide 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis."

NRC Safety Evaluation Report

Mr. Samuel Lee, NRR, explained the process for reviewing the Calvert Cliffs application. He presented the staff evaluation of the process for identifying structures and components subject to aging management review and identified common aging management programs. Mr. Lee described the fatigue monitoring program, the age-related degradation inspection program, and the time-limited

aging analyses. He presented a summary of the SER open and confirmatory items and provided the schedule for completing the final SER.

The ACRS Members and the staff discussed preparing the SERs without a Standard Review Plan and developing a list of existing aging management programs that could be referenced in future applications.

Conclusion

The Committee issued a letter to the Executive Director for Operations (EDO) dated May 19, 1999, on this matter.

V. Proposed Resolution of Generic Safety Issue (GSI) 158, "Performance of Safety Related Power-Operated Valves Under Design Bases Conditions" (Open)

[Note: Mr. Amarjit Singh was the Designated Federal Official for this portion of the meeting.]

Introduction

Dr. Mario V. Bonaca, Acting Chairman of the Subcommittee on Plant Systems, introduced this topic to the Committee. He stated that the purpose of this briefing was to hear presentations by and hold discussions with the representatives of the NRC staff regarding the proposed resolution of Generic Safety Issue (GSI) 158, "Performance of Safety-Related Power-Operated Valves Under Design Basis Conditions."

NRC Staff Presentation

Mr. John Craig, RES, led the presentations for the NRC staff and stated that RES has full support for the proposed resolution of GSI-158. The staff presented the background information regarding GSI-158 and proposed the resolution of this issue. GSI-158 included the power-operated valves (POVs), solenoid operated valves (SOVs), and hydraulic operated valves (HOVs) and the potential for these valves to not be able to perform their intended functions under design basis conditions. This GSI was prioritized in 1994 as MEDIUM. The staff recommended to resolve the GSI-158 on the basis of the results of the analysis performed by the Idaho National Engineering and Environmental Laboratory (INEEL). The INEEL study concluded that the resulting core damage frequency for four typical plants to

be within the range of $2.5E^{-06}$ per reactor-year to $6.5E^{-05}$ per reactor-year. The staff concluded that imposing new requirements and regulations, in addition to those presently available, would not significantly improve the margin of protection to the public health and safety.

Dr. Powers expressed a concern that the staff has not adequately addressed whether POVs will perform their intended functions under design basis dynamic conditions.

Conclusion

The Committee issued a letter to the NRC Executive Director for Operations dated May 14, 1999, on this matter.

VI. Proposed Resolution of Generic Safety Issue (GSI) 165, "Spring-Actuated Safety Relief Valve Reliability" (Open)

[Note: Mr. Amarjit Singh was the Designated Federal Official for this portion of the meeting.]

Introduction

Dr. Mario V. Bonaca, Acting Chairman of the Subcommittee on Plant Systems, introduced this topic to the Committee. He stated that the purpose of this briefing was to hear presentations by and hold discussions with representatives of the NRC staff concerning the status of activities related to a proposed resolution of GSI-165, "Spring-Actuated Safety Relief Valve Reliability."

NRC Staff Presentation

Mr. Owen P. Gormley, Office of Nuclear Regulatory Research (RES), presented a brief summary and status of GSI-165. He stated that the actions taken to evaluate and resolve this GSI did not confirm the assumptions made in the initial prioritization. The review of related valve data only identified a single valve in one plant type that had the potential for failing the train. The safety relief valve (SRV) was analyzed for worst case conditions and it showed that the increase in core damage frequency for that SRV is acceptable (6×10^{-6} per reactor-year). The staff requested a follow-up briefing during the June 1999 ACRS meeting.

Conclusion

The Committee decided to defer its comments until the June ACRS meeting.

VII. Fire Protection Functional Inspection Program (Open)

[Note: Mr. Amarjit Singh was the Designated Federal Official for this portion of the meeting.]

Introduction

Dr. Dana A. Powers, Chairman of the Subcommittee on Fire Protection, introduced this topic to the Committee. He stated that the purpose of this session was to hear presentations and hold discussions with the representatives of the NRC staff and NEI regarding the fire protection functional inspection program (FPFI) and related fire protection issues.

NRC Staff Presentations

Mr. Steven West, NRR, presented a brief background and current status regarding the FPFI program and also discussed the development for assessing the risk-significance of the fire protection program deficiencies such as FPFI and other fire protection inspection findings. He stated that the objective of the FPFI program was to renew industry attention to reactor safety and determine if the licensees are maintaining licensing and design basis of the fire protection program. The staff plans to issue the final Commission paper with the recommendations in mid-May 1999. He also stated that the staff has developed a fire protection risk significant screening methodology to be used to detect the risk significance of fire protection weaknesses. The purpose of the screening methodology was to focus the inspectors on risk significant sets of inspection findings, and to screen the findings which have minimal or no risk significance.

NEI Presentation

Mr. Fred Emerson, NEI, presented the industry views on the current fire protection inspection programs. NEI agrees with the staff that the current fire protection inspection program should be subsumed into a new reactor oversight process and needs to be balanced between NRC inspections and industry self-assessments.

However, NEI disagreed with the staff's plan to include the triennial inspection module with the reactor oversight inspection process. NEI is developing performance indicators for industry use to assure that there is adequate balance and that the level of self-assessment and NRC oversight is appropriate. NEI plans to issue the performance indicators in mid-year 2000.

Conclusion

The Committee plans to continue its review of the matters related to the FPF program and related issues during future meetings.

VIII. Westinghouse Owners Group (WOG) Proposal for Modification of Core Damage Assessment Guidelines (CDAG) and Post-Accident Sampling System (PASS) Requirements (Open)

[Note: Mr. Paul A. Boehnert was the Designated Federal Official for this portion of the meeting.]

The Committee heard presentations by and held discussions with representatives of the NRC staff and the Westinghouse Owners Group (WOG) regarding the WOG's proposed modifications to the core damage assessment guidelines (CDAG) and the post accident sampling system (PASS) requirements. The WOG stated that the modifications to the CDAG are based on knowledge of severe accidents that is more state-of-the-art, is focused on plant safety, meets regulatory requirements, and will better support accident management needs. Specifically, reliance will be on use of fixed in-plant instrumentation (core exit thermocouples, containment high-range radiation monitors) for input to the CDAGs. Similarly, the PASS modifications were characterized as technically sound, comporting with regulatory requirements, an improvement to the effectiveness of the site emergency plan, and useful for planning severe accident recovery actions. The WOG proposes to remove most of the PASS sampling and accuracy requirements from the plant licensing basis, with the exception of a few select parameters (e.g., RCS boron and containment hydrogen), and to retain sampling capability for long-term cleanup and recovery planning.

Representatives of the NRR staff discussed the background, scope, status and results of their review of the WOG's proposed modifications. For the CDAG, the staff finds the revised guidelines result in more timely information and are an improvement over the existing methodology. Regarding the PASS modifications,

NRR finds most acceptable, a few acceptable with conditions, and one set (sampling of radionuclides from RCS and containment spaces) is still under review. NRR also has under review the issue of how the PASS capabilities will be maintained outside the licensing basis.

Conclusion

The Committee issued a report to the NRC Executive Director for Operations dated May 19, 1999 on this matter.

IX. EXECUTIVE SESSION (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

A. Reconciliation of ACRS Comments and Recommendations

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

The Committee discussed the response from the NRC Executive Director for Operations (EDO) to ACRS comments and recommendations included in recent ACRS reports:

- The Committee discussed the response from the NRC Executive Director for Operations (EDO) dated March 31, 1999, to the ACRS comments and recommendations included in the ACRS letter dated February 23, 1999 concerning proposed improvements to the NRC inspection and assessment programs.

The Committee had initially expressed concern with some aspects of the EDO's response. The Committee plans to continue its review of this matter during future meetings related to the pilot plant application projects.

- The Committee discussed the response from the NRC Executive Director for Operations dated April 8, 1999, to ACRS comments and recommendations in the ACRS report dated March 22, 1999, concerning SECY-99-054, "Plans for Final Rule - Revisions to 10 CFR

Parts 50, 52, and 72: Requirements Concerning Changes, Tests, and Experiments.”

The Committee decided to continue its review of this matter during future meetings related to the proposed final revision to 10 CFR 50.59.

- The Committee discussed the response from the EDO dated April 19, 1999, to the ACRS comments and recommendations included in the ACRS letter dated March 23, 1999, concerning the proposed rule making to modify the reactor event reporting requirements in 10 CFR 50.72 and 50.73.

The Committee decided that it was satisfied with the EDO's response and plans to continue its discussion of this matter during future meetings.

- The Committee discussed the response from the EDO dated April 23, 1999, to the ACRS comments and recommendations included in the ACRS letter dated March 24, 1999, concerning the guidance memorandum for implementation of the revised enforcement policy.

The Committee decided that it was satisfied with the EDO's response and plans to continue its discussion of this matter during future meetings.

B. Report on the Meeting of the Planning and Procedures Subcommittee (Open)

The Committee heard a report from Dr. Powers and the Executive Director, ACRS, on the Planning and Procedures Subcommittee meeting held on May 5, 1999. The following items were discussed:

- Review of the Priorities for ACRS Reports and Letters Scheduled for the May 1999 ACRS meeting

Priorities for ACRS reports and letters scheduled for the May ACRS meeting were addressed. Letters that can be deferred and would

benefit from additional consideration at a future ACRS meeting were selected.

- Review of Activities Scheduled for Future ACRS Meetings and Member assignments

The activities scheduled for June 1999 and later ACRS meetings along with member assignments are attached to the Future Activities. The objectives are: (1) to review the reasons for the scheduling of each activity and the expected work product and to make changes, as appropriate, (2) to manage the members' workload for these meetings, and (3) to plan and schedule items for ACRS discussion of topical and emerging issues.

- Draft ACRS Letter on the EDO's Response to the February 1999 ACRS Report on the NRC Inspection and Assessment Program

During the April 1999 meeting, the ACRS Members discussed a proposed letter to the EDO to document ACRS concerns regarding the EDO's response to the February 23, 1999 ACRS report on the NRC's inspection and assessment program.

- Visit to the Susquehanna Nuclear Plant

Several Committee Members are scheduled to visit the Susquehanna nuclear plant on June 22 and meet with representatives of the NRC Region I Office on June 23 to discuss topics of mutual interest.

- Status of the Joint ACRS/ACNW Working Group on Risk-Informed, Performance-Based Regulations in NMSS

The Working Group consists of ACRS members, Drs. Apostolakis and Kress and ACNW members, Drs. Garrick and Hornberger. These four members discussed procedural issues related to the operation of the Working Group on April 22, 1999. The first meeting of the Working Group has been scheduled to be held on the morning of May 11, 1999.

- ACNW Report to the Commission on NRC-Sponsored Research and Technical Assistance

The ACNW has expressed a desire to report separately to the Commission in the future on the NRC-sponsored research/technical assistance related to its area of responsibility. This would facilitate the issuance of the ACNW report.

- Status of the Review of the Subcommittee Organization

The Subcommittee discussed Dr. Powers' proposal for reorganization of the ACRS subcommittees.

- Status of Quadripartite Working Group

The Planning and Procedures Subcommittee recommends that Dr. Uhrig and Dr. Apostolakis visit the Siemens Laboratories as representatives of the ACRS.

- Members' Issues

- The length of and the workload at the ACRS meetings has increased. One remedy would be to schedule more ACRS meetings in a year. The Committee Members should consider if they want to take this approach.
- A number of issues related to the performance of on-site dry storage casks are under evaluation. It is recommended that Dr. Shack, as Chairman of the Materials and Metallurgy Subcommittee, look into the matter and propose a course of action to the ACRS Chairman.
- Dr. Powers had provided the Committee with a draft letter to the Quadripartite Working Group on high burnup fuel during the March 1999 ACRS meeting. The draft letter urged that the Working Group participate in a proposed RES expert elicitation for phenomena identification and ranking in connection with the use of high burnup fuels. At the March ACRS meeting it

462nd ACRS Meeting
May 5-8, 1999

was proposed that the letter not be sent until the ACRS received an endorsement of its recommendation to the EDO for the participation of the Working Group in this effort. We have now received this endorsement.

C. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 463rd ACRS Meeting, June 2-4, 1999.

The 462nd ACRS meeting was adjourned at 1:00 p.m. on May 8, 1999.

official government statistics on employment and unemployment for over 50 years. Collection of labor force data through the CPS is necessary to meet requirements in Title 29, United States Code, Section 1 through 9. The Veterans Supplement meets the demands of Public Law 100-323, Section 9 (38 U.S.C. 2010A), as amended by Public Law 103-446, Section 701c, which mandates the Department of Labor to report on the labor force status of disabled veterans, Vietnam-theater veterans, and recently discharged veterans.

The Veterans Supplement provides information on the number and characteristics of disabled veterans, veterans who served in the Vietnam war theater, and recently separated veterans, including their employment status. The supplement provides data on veterans' participation in various employment training programs.

These data will be used by the Veterans Employment and Training Service and the Department of Veterans Affairs to determine policies that better meet the needs of our Nation's veteran population.

II. Current Actions

There are no substantial new actions in the September 1999 collection of the Veterans Supplement.

Type of Review: Reinstatement, with change, of a previously approved collection for which approval has expired.

Agency: Bureau of Labor Statistics.

Title: September 1999 CPS Veterans Supplement.

OMB Number: 1220-0102.

Affected Public: Individuals or households.

Total Respondents: 12,000.

Frequency: Biennially.

Total Responses: 12,000.

Average Time Per Response: 1 minute.

Estimated Total Burden Hours: 200 hours.

Estimated Total Burden Hours: 200 hours.

Total Burden Cost (Capital/startup): \$0.

Total Burden Cost (operating/maintenance): \$0.

Comments submitted in response to this notice will be summarized and/or included in the request for Office of Management and Budget approval of the information collection request; they also will become a matter of public record.

Signed at Washington, DC, this 15th day of April 1999.

W. Stuart Rust, Jr.,

Division of Management Systems, Bureau of Labor Statistics.

[FR Doc. 99-9877 Filed 4-19-99; 8:45 am]

BILLING CODE 4510-24-M

MERIT SYSTEMS PROTECTION BOARD

Announcement of Headquarters Closing on April 23, 1999

SUMMARY: In accordance with the Office of Personnel Management's recently announced guidelines for minimizing traffic in conjunction with the 50th Anniversary NATO Summit to be held in Washington, D.C., on April 23-25, 1999, the U.S. Merit Systems Protection Board (MSPB) office located at 1120 Vermont Avenue, N.W., Washington, D.C., will be closed on April 23, 1999. Documents that are due to be filed by April 23, 1999, with the Office of the Clerk or with the Office of the Administrative Law Judge will be accepted as timely if they are received or postmarked by April 26, 1999. Documents that are due to be filed with all other MSPB offices, including the Washington Regional Office, must be received or postmarked by April 23, 1999.

CONTACT PERSON FOR ADDITIONAL INFORMATION: Shannon McCarthy or Matthew Shannon, Office of the Clerk of the Board, (202) 653-7200.

Dated: April 15, 1999.

Robert E. Taylor,
Clerk of the Board.

[FR Doc. 99-9851 Filed 4-19-99; 8:45 am]

BILLING CODE 7400-01-M

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Meeting Notice

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on May 5-8, 1999, in Conference Room T-2B3, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the Federal Register on Wednesday, November 18, 1998 (63 FR 64105).

Wednesday, May 5, 1999

1:00 P.M.-1:15 P.M.: Opening Remarks by the ACRS Chairman

(Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

1:15 P.M.-1:45 P.M.: Electric Power Research Institute (EPRI) Application of Risk-Informed Methods to Inservice Inspection of Piping (Open)—The Committee will hear presentations by and hold discussions with representatives of EPRI and the NRC staff on the proposed application of risk-informed methods to inservice inspection of piping.

1:45 P.M.-2:45 P.M.: Proposed Final Revision to 10 CFR 50.59 (Changes, Test and Experiments) (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the staff's proposed final revision to 10 CFR 50.59.

3:00 P.M.-4:30 P.M.: Safety Evaluation for the Calvert Cliffs Nuclear Power Plant (CCNPP) License Renewal Application (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and of the Calvert Cliffs licensee on the CCNPP license renewal application.

4:30 P.M.-7:00 P.M.: Preparation of ACRS Reports (Open)—The Committee will discuss proposed ACRS reports, including a proposed report on the EPRI Proposed Application of Risk-Informed ISI of Piping, NRC Safety Research Program, proposed final revisions to 10 CFR 50.59, and Impact of high burnup or mixed oxide fuel on the revised source term.

Thursday, May 6, 1999

8:30 A.M.-8:35 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 A.M.-9:15 A.M.: Proposed Resolution of Generic Safety Issue (GSI) 158, "Performance of Safety Related Power-Operated Valves Under Design Bases Conditions" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed resolution of GSI 158.

9:15 A.M.-10:00 A.M.: Proposed Resolution of Generic Safety Issues (GSI) 165, "Spring-Actuated Safety Relief Valve Reliability" (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed resolution of GSI 165.

10:15 A.M.-11:45 A.M.: Fire Protection Functional Inspection Program (Open)—The Committee will

hear presentations by and hold discussions with representatives of the NRC staff regarding the staff work on the fire protection functional inspection program.

12:45 P.M.-2:15 P.M.: Westinghouse Owners Group (WOG) Proposal for Modification of Core Damage Assessment Guidelines (CDAG) and Post-Accident Sampling System (PASS) Requirements (Open/Closed)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and the WOG to discuss the WOG proposal to modify the CDAG and PASS requirements.

Note: A portion of this session may be closed to discuss Westinghouse Electric Company proprietary information.

2:15 P.M.-6:30 P.M.: Preparation of ACRS Reports (Open)—The Committee will discuss proposed ACRS reports.

Friday, May 7, 1999

8:30 A.M.-8:35 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 A.M.-9:30 A.M.: Tutorial on Instrument Setpoints (Open)—ACRS member, Dr. D. Miller, will provide a tutorial for the Committee on the issues and concerns associated with instrument setpoints for safety systems at nuclear power plants.

9:30 A.M.-9:45 A.M.: Reconciliation of ACRS Comments and Recommendations (Open)—The Committee will discuss the responses from the NRC Executive Director for Operations (EDO) to comments and recommendations included in recent ACRS reports and letters. The EDO responses are expected to be provided to the ACRS prior to the meeting.

9:45 A.M.-10:15 A.M.: Report of the Planning and Procedures Subcommittee (Open/Closed)—The Committee will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS. [Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this Advisory Committee, and information the release of which would constitute a clearly unwarranted invasion of personal privacy.]

10:30 A.M.-11:00 A.M.: Future ACRS Activities (Open)—The Committee will discuss the recommendations of the Planning and Procedures Subcommittee regarding items proposed for

consideration by the full Committee during future meetings.

11:00 A.M.-7:30 P.M.: Preparation of ACRS Reports (Open)—The Committee will continue its discussion of proposed ACRS reports.

Saturday, May 8, 1999

8:30 A.M.-2:00 P.M.: Preparation of ACRS Reports (Open)—The Committee will continue its discussion of proposed ACRS reports.

2:00 P.M.-2:30 P.M.: Miscellaneous (Open)—The Committee will discuss matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.

Procedures for the conduct of and participation in ACRS meetings were published in the Federal Register on September 29, 1998 (63 FR 51968). In accordance with these procedures, oral or written views may be presented by members of the public, including representatives of the nuclear industry. Electronic recordings will be permitted only during the open portions of the meeting and questions may be asked only by members of the Committee, its consultants, and staff. Persons desiring to make oral statements should notify Dr. Richard P. Savio, Associate Director for Technical Support, five days before the meeting, if possible, so that appropriate arrangements can be made to allow necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during this meeting may be limited to selected portions of the meeting as determined by the Chairman. Information regarding the time to be set aside for this purpose may be obtained by contacting the Associate Director for Technical Support prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the Associate Director for Technical Support if such rescheduling would result in major inconvenience.

In accordance with Subsection 10(d) P.L. 92-463, I have determined that it is necessary to close portions of this meeting noted above to discuss matters that relate solely to the internal personnel rules and practices of this Advisory Committee per 5 U.S.C. 552b(c)(2), to discuss Westinghouse proprietary information per 5 U.S.C. 552b(c)(4), and to discuss information the release of which would constitute a clearly unwarranted invasion of personal privacy per 5 U.S.C. 552b(c)(6).

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor, can be obtained by contacting Dr. Richard P. Savio, Associate Director for Technical Support (telephone 301/415-7363), between 7:30 a.m. and 4:15 p.m. EDT.

ACRS meeting agenda, meeting transcripts, and letter reports are available for downloading or viewing on the internet at <http://www.nrc.gov/ACRSACNW>.

Videoteleconferencing service is available for observing open sessions of ACRS meetings. Those wishing to use this service for observing ACRS meetings should contact Mr. Theron Brown, ACRS Audio Visual Technician (301-415-8066), between 7:30 a.m. and 3:45 p.m. EDT at least 10 days before the meeting to ensure the availability of this service. Individuals or organizations requesting this service will be responsible for telephone line charges and for providing the equipment facilities that they use to establish the videoteleconferencing link. The availability of videoteleconferencing services is not guaranteed.

Date: April 15, 1999.

Andrew L. Bates,
Advisory Committee Management Officer.
[FR Doc. 99-9838 Filed 4-19-99; 8:45 am]
BILLING CODE 7590-01-P

SECURITIES AND EXCHANGE COMMISSION

Submission for OMB Review; Comment Request

Upon Written Request, Copies Available From: Securities and Exchange Commission, Office of Filings and Information Services, Washington, DC 20549.

Extension:

Rule 17a-3, SEC File No. 270-27, OMB Control No. 3235-0035
Rule 11Ab2-1 and Form SIP, SEC File No. 270-23, OMB Control No. 3235-0043

Notice is hereby given that, pursuant to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.), the Securities and Exchange Commission (Commission) has submitted to the Office of Management and Budget (OMB) a request for the extension of the previously approved collections of information on the following:

Rule 17a-13(b) generally requires that at least once each calendar quarter, all registered brokers and dealers physically examine and count all



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

April 27, 1999

SCHEDULE AND OUTLINE FOR DISCUSSION
462nd ACRS MEETING
MAY 5-8, 1999

WEDNESDAY, MAY 5, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND

1:05 - 1:10
1) ~~1:00~~ - 1:15 P.M.

Opening Remarks by the ACRS Chairman (Open)

- 1.1) Opening statement (DAP/JTL/RPS)
- 1.2) Items of current interest (DAP/NFD/RPS)
- 1.3) Priorities for preparation of ACRS reports (DAP/JTL/RPS)

1:10 - 2:07
2) ~~1:15~~ - 1:45 P.M.

Electric Power Research Institute (EPRI) Application of Risk-Informed Methods to Inservice Inspection (Open) (WJS/GA/MTM)

- 2.1) Remarks by the Cognizant ACRS Member
- 2.2) Briefing by and discussions with representatives of EPRI and the NRC staff on the proposed application of risk-informed methods to inservice inspection of piping (ISI).

2:07 - 4:10
3) ~~1:45~~ - 2:45 P.M.

Proposed Final Revision to 10 CFR 50.59 (Changes, Tests and Experiments) (Open) (JJB/MTM)

- 3.1) Remarks by the Cognizant ACRS Member
- 3.2) Briefing by and discussions with representatives of the NRC staff regarding the staff's proposed final revision to 10 CFR 50.59.

Representatives of the nuclear industry will provide their views, as appropriate.

3:30 - 3:45
~~2:45~~ - 3:00 P.M.

BREAK

4:10 - 6:10
4) ~~3:00~~ - 4:30 P.M.

Safety Evaluation for the Calvert Cliffs Nuclear Power Plant (CCNPP) License Renewal Application (Open) (MHF/NFD)

- 4.1) Remarks by the Cognizant ACRS Member
- 4.2) Briefing by and discussions with representatives of the NRC staff and of the Calvert Cliffs licensee on the CCNPP license renewal application.

6:10 - 6:20
5) ~~4:30~~ - 7:00 P.M.
6:20 - 6:51

BREAK

Preparation of ACRS Reports (Open)

Discussion of proposed ACRS reports:

- 5.1) Proposed ACRS report on the NRC Safety Research Program (REU/MME)
- 5.2) EPRI Proposed Application of Risk-Informed ISI of Piping (WJS/GA/MTM)
- 5.3) Proposed final revisions to 10 CFR 50.59 (JJB/MTM)

- 5.4) Impact of High Burnup or Mixed Oxide Fuel on the Revised Source Term (DAP/MME)
- 5.5) Relationship and Balance Between PRA Results and Defense-in-Depth (TSK/PAB/JNS)

THURSDAY, MAY 6, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 6) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (DAP/RPS)
- 7) 8:35 - ^{10:00}~~9:15~~ A.M. Proposed Resolution of Generic Safety Issue (GSI) 158, "Performance of Safety Related Power-Operated Valves Under Design Bases Conditions" (Open) (MVB/AS)
 - 7.1) Remarks by the Cognizant ACRS Member
 - 7.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed resolution of GSI 158.
- 8) ^{10:00 - 10:10}~~9:15~~ - 10:00 A.M. Proposed Resolution of Generic Safety Issue (GSI) 165, "Spring-Actuated Safety Relief Valve Reliability" (Open) (MVB/AS)
 - 8.1) Remarks by the Cognizant ACRS Member
 - 8.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed resolution of GSI 165.
- ^{10:10 - 10:25}~~10:00~~ - 10:15 A.M. *****BREAK*****
- 9) ^{10:25 - 11:47}~~10:15~~ - 11:45 A.M. Fire Protection Functional Inspection Program (Open) (DAP/AS)
 - 9.1) Remarks by the Cognizant ACRS Members
 - 9.2) Briefing by and discussions with representatives of the NRC staff regarding the staff work on the fire protection functional inspection program.
- ^{11:47 - 12:50}~~11:45~~ - 12:45 P.M. *****LUNCH*****
- 10) ^{12:50 - 3:35}~~12:45~~ - 2:15 P.M. Westinghouse Owners Group (WOG) Proposal for Modification of Core Damage Assessment Guidelines (CDAG) and Post-Accident Sampling System (PASS) Requirements (Open/Closed) (TSK/PAB)
 - 10.1) Remarks by the Cognizant ACRS Member
 - 10.2) Briefing by and discussions with representatives of the NRC staff and WOG to discuss the WOG proposal for modification of CDAG and PASS requirements.

Representatives of the nuclear industry will provide their views, as appropriate.

[Note: A portion of this session may be closed to discuss Westinghouse Electric Company proprietary information.]

3:05-3:20 pm Break
 3:35-4:20 pm Revisions to the Maintenance Rule
 - Classifying high/low risk significance

4:20 - 6:45
 11) ~~2:45 - 3:00~~ P.M.
 (2:45-3:00 P.M. BREAK)

Preparation of ACRS Reports (Open)

Discussion of proposed ACRS reports:

- 11.1) ACRS Report on the NRC Safety Research Program(REU/MME)
- 11.2) Proposed Final Revision to 10 CFR 50.59 (JJB/MTM)
- 11.3) Interim Report on CCNPP License Renewal Application (MHF/NFD)
- 11.4) Proposed Resolutions of GSI 158 and GSI 165 (DWM/AS)
- 11.5) EPRI Proposed Application of Risk-Informed ISI of Piping (WJS/GA/MTM)
- 11.6) WOG Proposal for Modification of CDAG and PASS Requirements (TSK/PAB)
- 11.7) Impact of High Burnup or Mixed Oxide Fuel on the Revised Source Term (DAP/MME)
- 11.8) Relationship and Balance Between PRA Results and Defense-in-Depth (TSK/PAB/JNS)
- 11.9) Fire Protection Function Inspection Program (DAP/AS)

FRIDAY, MAY 7, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

8:37 - 8:40
 12) ~~8:30 - 8:35~~ A.M.

Opening Remarks by the ACRS Chairman (Open) (DAP/RPS)

8:40 - 9:55
 13) ~~8:35 - 9:30~~ A.M.

Tutorial on Instrument Setpoints (Open) (DWM/AS)

ACRS member, Dr. D. Miller, will provide a tutorial for the Committee on the issues and concerns associated with instrument setpoints for safety systems at nuclear power plants.

9:55 - 10:15 Break
 14) ~~9:30 - 9:45~~ A.M.

Reconciliation of ACRS Comments and Recommendations (Open) (DAP, et al./SD, et al.)

Discussion of the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.

1:05 - 1:10 pm

15) 9:45 - 10:15 P.M.

Report of the Planning and Procedures Subcommittee (Open/Closed) (DAP/JTL)

Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.

[Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this Advisory Committee, and information the release of which would constitute a clearly unwarranted invasion of personal privacy.]

~~10:15 - 10:30 A.M.~~ ***BREAK***

11:30 - 1:40 pm
16) 10:30 - 11:00 A.M.

Future ACRS Activities (Open) (DAP/JTL/RPS)

Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings.

10:15 - 8:30
17) 11:00 - 7:30 P.M.
(12:00-1:00 P.M.-LUNCH)
11:55-

Preparation of ACRS Reports (Open)

Continue discussion of proposed ACRS reports:

- 17.1) ACRS Report on the NRC Safety Research Program (REU/MME)
- 17.2) Proposed Final Revision to 10 CFR 50.59 (JJB/MTM)
- 17.3) Interim Report on CCNPP License Renewal Application (MHF/NFD)
- 17.4) Proposed Resolutions of GSI 158 and GSI 165 (DWM/AS)
- 17.5) EPRI Proposed Application of Risk-Informed ISI (WJS/GA/MTM)
- 17.6) WOG Proposal for Modification of CDAG and PASS Requirements (TSK/PAB)
- 17.7) Impact of High Burnup or Mixed Oxide Fuel on the Revised Source Term (DAP/MME)
- 17.8) Relationship and Balance Between PRA Results and Defense-in-Depth (TSK/PAB/JNS)
- 17.9) Fire Protection Function Inspection Program (DAP/AS)

SATURDAY, MAY 8, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

12:45
18) 8:30 - 2:00 P.M.
(12:00-1:00 P.M. LUNCH)

Preparation of ACRS Reports (Open)

Continue discussion of proposed ACRS reports listed under Item 17.

~~19) 2:00 - 2:30 P.M.~~

~~Miscellaneous (Open) (DAP/JTL/RPS)~~

~~Discussion of matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.~~

NOTE:

- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
- Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX III: MEETING ATTENDEES

462ND ACRS MEETING MAY 5-8, 1999

NRC STAFF (May 5, 1999)

S. Dinsmore, NRR
E. McKenna, NRR
F. Akstulewicz, NRR
D. Matthews, NRR
C. Carpenter, NRR
P. Shemanski, NRR
C. Grimes, NRR
S. Hoffman, NRR
R. Anand, NRR
S. Coffin, NRR
C. Gratton, NRR
A. Dromerick, NRR
B. Elliott, NRR
W. Burton, NRR
N. Yuki, NRR
P. T. Kuo, NRR
T. Liu, NRR
J. Fair, NRR
S. Lee, NRR
B. Prato, NRR
H. Graves, RES
R. Weisman, OGC

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

R. Gamble, Sartrex, Corp.	P. Riccardella, Structural Integrity Assoc.
J. Sieber, Self	B. Mrowca, BGE
J. Mitman, EPRI	W. Stuard, BGE
P. O'Regan, EPRI	M. Bowman, BGE
H. Fish, NYPA	P. Negus, GE
G. Smith, NYPA	C. Yoder, BGE
C. Amoriso, NUS	D. Ward, Ward Assoc.
K. Fleming, ERIN	P. Fulford, NUS
R. Huston, LSS	D. Walters, NEI
R. Bell, NEI	
B. Doroshuk, BGE	
R. Heibel, BGE	
V. Dimitrijevic, DE&S	

NRC STAFF (May 6, 1999)

J. Sharkey, OCM/EM	R. Palla, NRR
T. Tschiltz, OEDO	T. Essig, NRR
J. Craig, RES	F. Kantor, NRR
F. Cherny, RES	P. Wen, NRR
S. Bahadur, RES	K. Thomas, NRR
J. Flack, RES	S. LaVie, NRR
W. Gormley, RES	A. Levin, NRR
H. Ornstein, RES	P. Wilson, NRR
H. Vandermolen, RES	S. Wong, NRR
T. Y. Chang, RES	G. Thomas, NRR
N. Siu, RES	W. Scott, NRR
D. Terao, NRR	T. Quay, NRR
J. Colaccino, NRR	
G. Imbro, NRR	
S. West, NRR	
J. Hannon, NRR	
G. Hammer, NRR	
P. Madden, NRR	
L. Whitney, NRR	
G. Parry, NRR	
M. Rubin, NRR	
L. Lois, NRR	
K. Parczewski, NRR	
J. O'Brien, NRR	

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

P. McCabe, LMITCO-INEEL
J. Sieber, Self
O. Rothberg, INEEL
A. Wyche, Bechtel
F. Emerson, NEI
D. Modeen, NEI
H. Fontecilla, VA Power
S. Floyd, NEI
L. Collins, ABB
T. Garrett, WCNOC
W. Harrison, STPNOG/WOG
B. Bryan, WOG/TVA
K. Vavrek, Westinghouse
B. Lutz, Westinghouse



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

May 25, 1999 - REVISED

**SCHEDULE AND OUTLINE FOR DISCUSSION
463rd ACRS MEETING
JUNE 2-4, 1999**

**WEDNESDAY, JUNE 2, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND**

- 1) 8:30 - 8:45 A.M. Opening Remarks by the ACRS Chairman (Open)
 - 1.1) Opening statement (DAP/JTL/RPS)
 - 1.2) Items of current interest (DAP/NFD/SD)
 - 1.3) Priorities for preparation of ACRS reports (DAP/JTL/SD)

- 2) 8:45 - 10:15 A.M. Hydrogen Control Exemption Request for the San Onofre Nuclear
Generating Station Units 2 and 3 (Open) (TSK/PAB)
 - 2.1) Remarks by the Subcommittee Chairman
 - 2.2) Briefing by and discussions with representatives of the Southern California Edison (SCE) and NRC staff regarding the request by SCE for a license exemption to the hydrogen control requirements for the San Onofre Nuclear Generating Station Units 2 and 3, and the associated NRC staff's Safety Evaluation Report.

- 10:15 - 10:30 A.M. *****BREAK*****

- 3) 10:30 - 12:00 Noon Status of the Pilot Application of the Revised Inspection and Assessment
Programs (Open) (JJB/PAB)
 - 3.1) Remarks by the Subcommittee Chairman
 - 3.2) Briefing by and discussions with representatives of the NRC staff regarding the status of the pilot application of the revised inspection and assessment programs, and related matters.

Representatives of the nuclear industry will provide their views, as appropriate.

- 12:00 - 1:00 P.M. *****LUNCH*****

- 4) 1:00 - 2:30 P.M. Proposed Risk-Based Performance Indicators (Open) (GA/MTM)
 - 4.1) Remarks by the Subcommittee Chairman
 - 4.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed risk-based performance indicators (PIs), trial application of PIs, and identification of thresholds for regulatory action.

Representatives of the nuclear industry will provide their views, as appropriate.

2:30 - 2:45 P.M.

BREAK

5) 2:45 - 4:15 P.M.

Performance-Based Regulatory Initiatives and Related Matters (Open)
(GA/MTM)

- 5.1) Remarks by the Subcommittee Chairman
- 5.2) Briefing by and discussions with representatives of the NRC staff regarding performance-based regulatory initiatives and related matters.

Representatives of the nuclear industry will provide their views, as appropriate.

4:15 - 5:15 P.M.

Break and Preparation of ACRS Reports

Cognizant ACRS Members will prepare draft reports for consideration by the Committee

6) 5:15 - 7:00 P.M.

Discussion of ACRS Reports (Open)

Discussion of proposed ACRS reports on:

- 6.1) Performance-Based Regulatory Initiatives (GA/MTM)
- 6.2) Risk-Based Performance Indicators (GA/MTM)
- 6.3) Exemption Request to Hydrogen Control Requirements for San Onofre Units 2 and 3 (TSK/PAB)
- 6.4) Status of Pilot Application of the Revised Inspection and Assessment Programs (JJB/PAB)

THURSDAY, JUNE 3, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

7) 8:30 - 8:35 A.M.

Opening Remarks by the ACRS Chairman (Open) (DAP/RPS)

8) 8:35 - 10:00 A.M.

Use of Averted Onsite Costs and Voluntary Initiatives in Regulatory Analyses (Open) (TSK/NFD)

- 8.1) Remarks by the Subcommittee Chairman
- 8.2) Briefing by and discussions with representatives of the NRC staff regarding proposed options for treating averted onsite costs and voluntary initiatives in regulatory analyses.

Representatives of the nuclear industry will provide their views, as appropriate.

10:00 - 10:15 A.M.

BREAK

9) 10:15 - 11:15 A.M.

Development of a Low-Power and Shutdown Risk Program (Open)
(JJB/MME)

- 9.1) Remarks by the Subcommittee Chairman
- 9.2) Briefing by and discussions with representatives of the NRC staff regarding the status of development of a low-power and shutdown risk program.

Representatives of the nuclear industry will provide their views, as appropriate.

10) 11:15 - 11:45 A.M.

Strategy for ACRS Review of License Renewal Activities (Open)
(MVB/NFD)

Discussion of a proposed ACRS strategy for reviewing plant-specific license renewal applications, industry topical reports, and related matters.

11:45 - 12:45 P.M.

*****LUNCH*****

11) 12:45 - 2:15 P.M.

Options for Crediting Existing Programs for License Renewal (Open)
(RLS/NFD)

11.1) Remarks by the Cognizant ACRS Member

11.2) Briefing by and discussions with representatives of the NRC staff regarding proposed options for crediting existing NRC-approved programs for license renewal.

Representatives of the nuclear industry will provide their views, as appropriate.

2:15 - 2:30 P.M.

*****BREAK*****

12) 2:30 - 3:30 P.M.

Proposed Resolution of Generic Safety Issue (GSI)-165, "Spring-Actuated Safety and Relief Valve Reliability" (Open) (MVB/AS)

12.1) Remarks by the Subcommittee Chairman

12.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed resolution of GSI-165.

13) 3:30 - 4:00 P.M.

Report of the Joint ACRS/ACNW Working Group (Open)
(TSK/GA/NFD/JS)

Report of the Joint ACRS/ACNW Working Group regarding matters discussed at the May 11, 1999 meeting and procedures for reviewing and commenting on items of mutual interest between ACRS and ACNW.

4:00 - 5:00 P.M.

Break and Preparation of ACRS Reports

Cognizant ACRS Members will prepare draft reports for consideration by Committee.

14) 5:00 - 7:15 P.M.

Discussion of ACRS Reports (Open)

Discussion of proposed ACRS reports:

14.1) Proposed Resolution of GSI-165 (MVB/AS)

14.2) Use of Averted Onsite Costs and Voluntary Initiatives in Regulatory Analyses (TSK/NFD)

14.3) Options for Crediting Existing Programs for License Renewal (RLS/NFD)

14.4) Development of Low-Power and Shutdown Risk Program (JJB/MME)

FRIDAY, JUNE 4, 1999, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 15) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (DAP/RPS)
- 16) 8:35 - 9:30 A.M. Perspective on Nuclear Safety and the Regulatory Process (Open) (MVB/AS)
Briefing by and discussions with Dr. Bonaca, ACRS Member, regarding his perspective on nuclear safety and the regulatory process.
- 17) 9:30 - 10:00 A.M. Site Visit to the Susquehanna Steam Electric Station and Meeting with the NRC Region I Personnel (Open) (JJB/DAP/AS)
Discussion of the proposed schedule for touring the Susquehanna Steam Electric Station, specific plant areas to be visited, proposed issues for discussion with the licensee, and topics for meeting with representatives of the NRC Region I Office.
- 10:00 - 10:15 A.M. *****BREAK*****
- 18) 10:15 - 10:45 A.M. Future ACRS Activities (Open) (DAP/JTL/RPS)
Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings.
- 19) 10:45 - 11:15 A.M. Report of the Planning and Procedures Subcommittee (Open/Closed) (DAP/JTL)
Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.
- [Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this Advisory Committee, and information the release of which would constitute a clearly unwarranted invasion of personal privacy.]
- 20) 11:15 - 11:30 A.M. Reconciliation of ACRS Comments and Recommendations (Open) (DAP, et al./SD, et al.)
Discussion of the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.
- 11:30 - 12:30 P.M. *****LUNCH*****

21) 12:30 - 5:00 P.M.
(2:30-2:45 P.M. BREAK)

Discussion of ACRS Reports (Open)

Continue discussion of proposed ACRS reports on:

- 21.1) Risk-Based Performance Indicators (GA/MTM)
- 21.2) Performance-Based Regulatory Initiatives (GA/MTM)
- 21.3) Exemption Request to Hydrogen Control Requirements for San Onofre Units 2 and 3 (TSK/PAB)
- 21.4) Status of Pilot Applications of the Revised Inspection and Assessment Programs (JJB/PAB)
- 21.5) Development of Low-Power and Shutdown Risk Program (JJB/MME)
- 21.6) Options for Crediting Existing Programs for License Renewal (RLS/NFD)
- 21.7) Use of Averted Onsite Costs and Voluntary Initiatives in Regulatory Analyses (TSK/NFD)
- 21.8) Proposed Resolution of GSI -165 (MVB/AS)

22) 5:00 - 5:30 P.M.

Miscellaneous (Open) (DAP/JTL/RPS)

Discussion of matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit.

NOTE:

- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
- Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX V
LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA
ITEM NO.

DOCUMENTS

- 1 Opening Remarks by the ACRS Chairman
 1. Items of Interest dated May 5-8, 1999
- 2 Electric Power Research Institute (EPRI) Application of Risk-Informed Methods to Inservice Inspection
 2. EPRI Risk Informed - ISI Program to ACRS Presentation by EPRI [Viewgraphs]
 3. EPRI Methodology for Risk-Informed Inservice Inspection Programs for Piping Presentation by NRR, S. Ali, S. Dinsmore [Viewgraphs]
- 3 Proposed Final Revision to 10 CFR 50.59 (Changes, Tests and Experiments)
 4. Final Rulemaking for 10 CFR 50.59, 72.48, and Related Sections Presentation by NRR [Viewgraphs]
- 4 Safety Evaluation for the Calvert Cliffs Nuclear Power Plant (CCNPP) License Renewal Application
 5. Calvert Cliffs License Renewal Application, Part 54, Presentation by S. Hoffman, NRR [Viewgraphs]
 6. Calvert Cliffs Life Cycle Management/License Renewal Presentation by Barth Doroshuk, BGE [Viewgraphs]
- 5 Proposed Resolution of Generic Safety Issue (GSI) 158, "Performance of Safety Related Power-Operated Valves Under Design Bases Conditions"
 7. Closure of GSI 158 Performance of Safety-Related Power-Operated Valves (POVs) Under Design Basis Conditions Presentation by F. Cherny, RES [Viewgraphs]

- 6 Proposed Resolution of Generic Safety Issue (GSI) 165, "Spring-Actuated Safety Relief Valve Reliability"
- 7 Fire Protection Functional Inspection Program
 8. Fire Protection Functional Inspection Pilot Program Presentation by S. West, NRR [Viewgraphs]
 9. Fire Protection Risk Significance Screening Methodology Presentation by PSAB & PSB, NRR [Viewgraphs]
- 8 Westinghouse Owners Group (WOG) Proposal for Modification of Core Damage Assessment Guidelines (CDAG) and Post-Accident Sampling System (PASS) Requirements
 10. Revised Core Damage Assessment and Post Accident Sampling Presentation by B. Bryan, TVA, Westinghouse Owners Group [Viewgraphs]
 11. Core Damage Assessment Guideline Review Presentation by R. Palla, NRR [Viewgraphs]
 12. Diagram Post-Accident Sample System (PASS) Presentation by NRR [Viewgraphs]
- 8a. Revision to the Maintenance Rule
 13. Modification to the Revision to Paragraph (a)(3) of the Maintenance Rule, 10 CFR 50.65 Presentation by R. Correia, NRR [Viewgraphs]
- 9 Tutorial On Instrument Setpoints
 14. Instrument Setpoints Presentation by Professor Don W. Miller, Ohio State University & ACRS Member [Viewgraphs]
- 10 Reconciliation of ACRS Comments and Recommendations
 15. Reconciliation of ACRS Comments and Recommendations [Handout #14.1]
- 11 Future ACRS Activities
 - 16 Future ACRS Activities - 463rd ACRS Meeting, June 2-4, 199 [Handout #16-1]
- 11 Report of the Planning and Procedures Subcommittee

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

2 Electric Power Research Institute

1. Table of Contents
2. Proposed Schedule
3. Status Report, dated May 5, 1999
4. Letter dated April 16, 1999, from J. Mittman, EPRI, to M. Markley, ACRS Staff, Subject: EPRI TR-112657, "Revised Risk-Informed Inservice Inspection Procedure"
5. Report dated June 12, 1998, from R. L. Seale, Chairman, ACRS, to Shirley Ann Jackson, Chairman, NRC, Subject: Proposed Final Standard Review Plan Section 3.9.8 and Regulatory Guide 1.178 for Risk-Informed Inservice Inspection of Piping
6. Report dated November 20, 1998, from R. L. Seale, Chairman, ACRS, to William D. Travers, EDO, NRC, Subject: Safety Evaluation Report Related to Westinghouse Owners Group Application of Risk Informed Methods to Inservice Inspection of Piping, Topical Report (WCAP-14572, Revision 1)

3 Proposed Final Revision to 10 CFR 50.59 (Changes, Tests and Experiments)

7. Table of Contents
8. Proposed Schedule
9. Status Report dated May 5, 1999
10. Memorandum dated April 28, 1999, from Eileen McKenna, NRR, to Cynthia A. Carpenter, NRR, Subject: Summary of April 26, 1999, meeting with the Nuclear Energy Institute (NEI) Regarding Revisions to Evaluation Criteria in 10 CFR 50.59 (pre-decisional)
11. Staff Requirements Memorandum dated September 25, 1998, from John C. Hoyle, Secretary, NRC, to L. Joseph Callan, EDO, Subject: SECY-98-171
12. Report dated March 22, 1999, from Dana A. Powers, Chairman, ACRS, to Shirley Ann Jackson, Chairman, NRC, Subject: SECY-99-054, "Plans for Final Rule - Revisions to 10 CFR Parts 50, 52, and 72: Requirements Concerning Changes, Tests, and Experiments"
13. Report dated February 18, 1999, from D. Powers, Chairman, ACRS to Shirley Ann Jackson, Chairman, NRC, Subject: List of Key Issues Associated with the Proposed Revision to 10 CFR 50.59
14. Report dated July 16, 1998, from R. L. Seale, Chairman, ACRS, to Shirley Ann Jackson, Chairman, NRC, Subject: Proposed Revisions to 10 CFR 50.59 (Changes, Tests and Experiments)

- 4 Safety Evaluation for the Calvert Cliffs Nuclear Power Plant (CCNPP) License Renewal Application
 15. Table of Contents
 16. Proposed Schedule
 17. Status Report dated May 5, 1999
 18. Letter dated April 8, 1999, from Charles H. Cruse, Baltimore Gas and Electric Company to U.S. Nuclear Regulatory Commission, Subject: Calvert Cliffs Nuclear Power Plant Unit Nos. 1 & 2; Application for License Renewal
 19. U.S. Nuclear Regulatory Commission Safety Evaluation Report Related to the License Renewal of Calvert Cliffs Nuclear Power Plant, Unit 1 and 2, Chapter 1: "Introduction and General Discussion," issued March 1999

- 7 Proposed Resolution of Generic Safety Issue (GSI) 158, "Performance of Safety Related Power-Operated Valves Under Design Bases Conditions"
 20. Table of Contents
 21. Proposed Schedule
 22. Status Report dated May 6, 1999
 23. Memorandum to John T. Larkins, Executive Director, ACRS, from John W. Craig, Director, Division of Regulatory Applications, Office of Nuclear Regulatory Research, Subject: Resolution of Generic Safety Issue 158, "Performance of Safety-Related Power-Operated Valves Under Design Basis Conditions," dated April 5, 1999

- 8 Proposed Resolution of Generic Safety Issue (GSI) 165, "Spring-Actuated Safety Relief Valve Reliability"
 24. Table of Contents
 25. Proposed Schedule
 26. Status Report dated May 6, 1999
 27. Memorandum to John T. Larkins, Executive Director, ACRS, from John w. Craig, Director, Division of Generic Safety Issue 165, "Spring-Actuated Safety and Relief Valve Reliability," dated April 2, 1999

- 9 Fire Protection Functional Inspection Program
 28. Table of Contents
 29. Proposed Schedule
 30. Status Report dated May 6, 1999
 31. Flow Chart for Reactor Fire Protection Master Plan
 32. Commission Paper, Subject: Status of Reactor Fire Protection Projects and Recommendations for Fire Protection Inspections (Draft Predecisional Information - For ACRS Use Only)

33. Letter to William M. Dean, Chief, Inspection Program Branch, Division of Inspection Program Management, NRR, from David J. Modeen, Director, Nuclear Generation Division, NEI, Subject: Industry Comments on Proposed Fire Protection Inspection Module, dated April 14, 1999
 34. Draft Inspection Procedures for Fire Protection Triennial Inspection
10. Westinghouse Owners Group (WOG) Proposal for Modification of Core Damage Assessment Guidelines (CDAG) and Post-Accident Sampling System (PASS) Requirements
35. Table of Contents
 36. Project Status Report dated May 6, 1999
 37. NRC Transmittal: "Background and NRR Staff Preliminary Evaluation of WCAP-14696, Westinghouse Owners Group Core Damage Assessment Guidance," dated April 19, 1999 (Preliminary Evaluation)
 38. NRC Transmittal: "NRC Transmittal: "Background and NRR Staff Preliminary Evaluation of WCAP-14986-P, "Westinghouse Owners Group Post Accident Sampling System Requirements, A Technical Basis," dated April 21, 1999
 39. Memorandum to T. Kress, from P. Boehnert, NRR/WOG Meeting - Proposed Post Accident Sampling System Requirements, March 25, 1999, dated April 12, 1999
 40. Memorandum to T. Kress, from P. Boehnert, NRR/WOG Meeting - Proposed Revisions to Core Damage Assessment Guidelines, February 24, 1999, dated March 8, 1999.